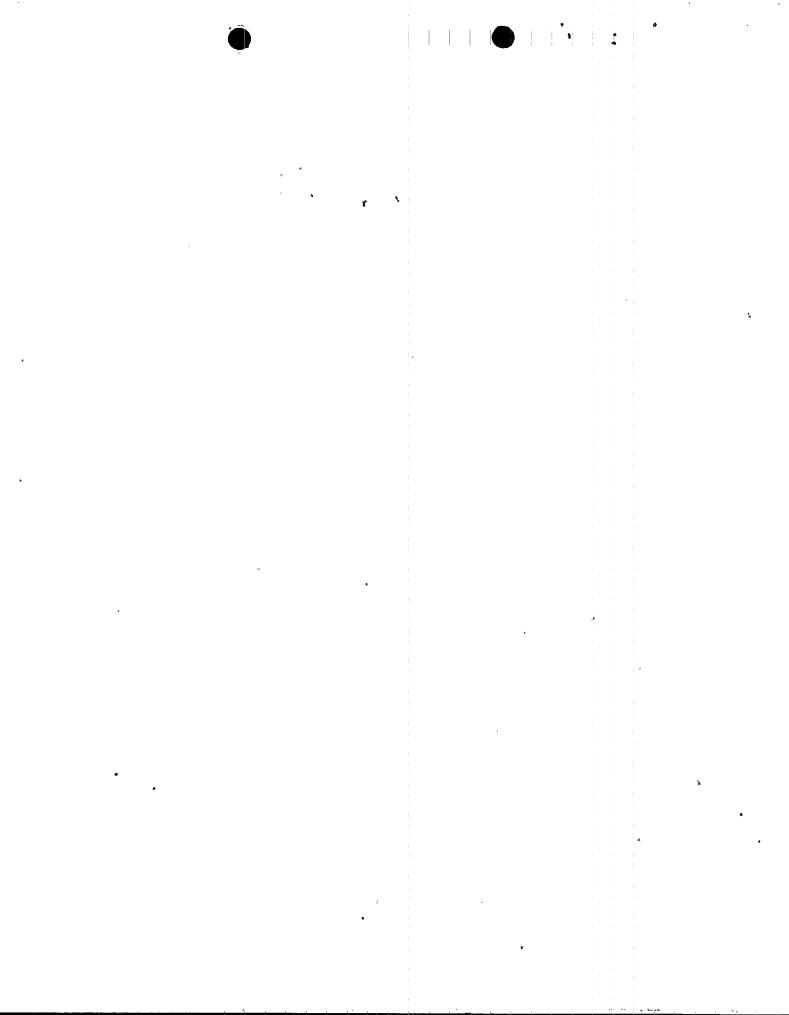
• / •	·····					•		
NRC FORM 195						50-250351		
NRC DISTRIBUT	TION FOR PAR	T 50 DOCKET	MA	TERIAL	F	ILE NUMBER		
TO: FROM: Florida Pov				ver & Light Company		DATE OF DOCUMENT 6/9/77		
Mr. Victor Stello Miami,			mi, Fla. ert E. Uhrig			DATE RECEIVED		
ELETTER ONOTORIZED PROP ORIGINAL SUNCLASSIFIED			IOP INPUT FORM			UMBER OF COPIES RECEIVED		
DESCRIPTION	<u>I</u>	ENCLOSURE						
				Amdt. to OL/change to Appendix A tech specsnotorized 6/10/77concerns the air filtration systems				
ACKNOWLEDGED DO NOT REMOVE (2-P)				40 cy (10-P)				
PLANT NAME: Turkey Point Units 3 & 4 RJL 6/17/77				40 encl.				
SAFETY		FOR ACTION/	INF			RONMENTAL		
ASSIGNED AD:	LEAR			ASSIGNED AD: V > BRANCH CHIEF:	• M	OORE (LTR)		
BRANCH CHIEF: (5				PROJECT MANAGER:	r,			
PROJECT MANAGER: ELLIOTT		54	LICENSING ASSISTANT:					
				B. HARLESS		• 		
		INTERNAL D	IST					
REG FILES	SY STEMS	A DESCRIPTION OF A DESC		PLANT SYSTEMS	T	SITE SAFETY &		
NRC PDR	HEINEMAN			TEDESCO		ENVIRON ANALYSIS		
T&E 12.1	SCHROEDER	>		BENAROYA		DENTON & MULLER		
		•		LAINAS	-	CRUTCHFIELD		
OELD .	ENOTHERI			IPPOLITO	-1			
GOSSICK & STAFF	ENGINEER			F. ROSA		ENVIRO TECH.		
MTPC	BOSNAK				-	ERNST		
CASE (VIV				OPERATING REACTORS	+	BALLARD		
BOYD	PAWLICK			- STELLO		YOUNGBLOOD		
		•	30	EISENHUT	-1			
PROJECT MANAGEMENT	REACTOR	CAFETY '	5	SHAQ	+			
		ORF BIL		BAER	+	SITE TECH.		
SKOVHOLT	NOVAK			BUTLER	+	GAMMILL (2)		
P. COLLINS	ROSZTOCZY	p	5	GRIMES	+	GORTILDE (2)		
HOUSTON	CHECK	·	-	GUTTIND	+	SITE ANALYSIS		
HELTEMES	- UNEUN	· · · · · · · · · · · · · · · · · · ·	l		+	VOLLMER		
SK	AT&I				-(BUNCH		
	SALTZMAN					J. COLLINS		
	RUTBERG				-1	KREGER		
EXTERNAL DISTRIBUTION						CONTROL NUMBER		
LPDR:			·			771710006		
TIC	NSIC				-			
NAT LAB REG IV (J. HANCHETT)					1	A		
16 CYS ACRS SENT CATE	GORY 3				4	1 as i		
					1	• 0		

-7



O. BOX 013100, MIAMI, FL 33101



June 9, 1977 L-77-174

Regulatory Docket File

Director of Nuclear Reactor Regulation Attention: Mr. Victor Stello, Director Division of Operating Reactors U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Dear Mr. Stello:

Re: Turkey Point Units 3 and 4 Docket Nos. 50-250 and 50-251 Proposed Amendment to Facility Operating Licenses DPR-31 and DPR-41

In accordance with 10 CFR 50.30, Florida Power & Light Company submits herewith three (3) signed originals and forty (40) copies of a request to amend Appendix A of Facility Operating Licenses DPR-31 and DPR-41.

In response to telephone requests from members of your staff, we have revised our proposal of May 3, 1976 on the subject of air filtration systems. The present proposal conforms as near as possible to the Standard Technical Specifications without our having to make hardware modifications. The proposed changes are described below and shown on the accompanying Technical Specification pages bearing the date of this letter in the lower right hand corner.

Pages ii and iii

Revised Table of Contents.

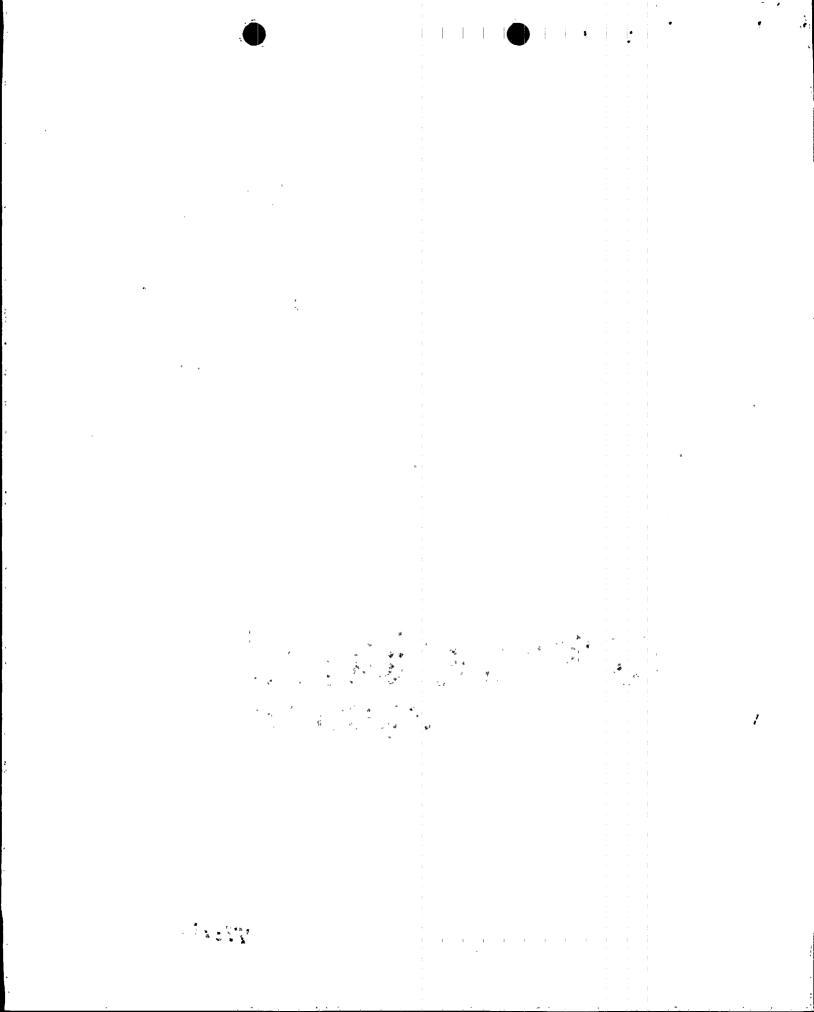
Page 3.4-4

Specification 3.4.3.b is revised. This specification concerns the limiting conditions for operation applicable to the Emergency Containment Filtering System during power operation.

Page 3.13-1

New specification 3.13 is added. This specification concerns the limiting conditions for operation applicable to the Control Room Ventilation System and the Post Accident Containment Vent System.

771710006



Director of Nuclear Reactor Regulation Page Two

Pages 4.7-1 through 4.7-3

The surveillance requirements of specifications 4.7.1 (Emergency Containment Filtering System) and 4.7.2 (Post Accident Containment Vent System) are revised. A new specification 4.7.3 (Control Room Ventilation System) is added.

Page B3.13-1

A new section B3.13-1 is added to provide bases for new specification 3.13.

Page B4.7-1

The bases for specification 4.7 are revised to include the Control Room Ventilation System.

The proposed amendment has been reviewed by the Turkey Point Plant Nuclear Safety Committee and the Florida Power & Light Company Nuclear Review Board and the conclusion reached that it does not involve an unreviewed safety question.

Very truly yours,

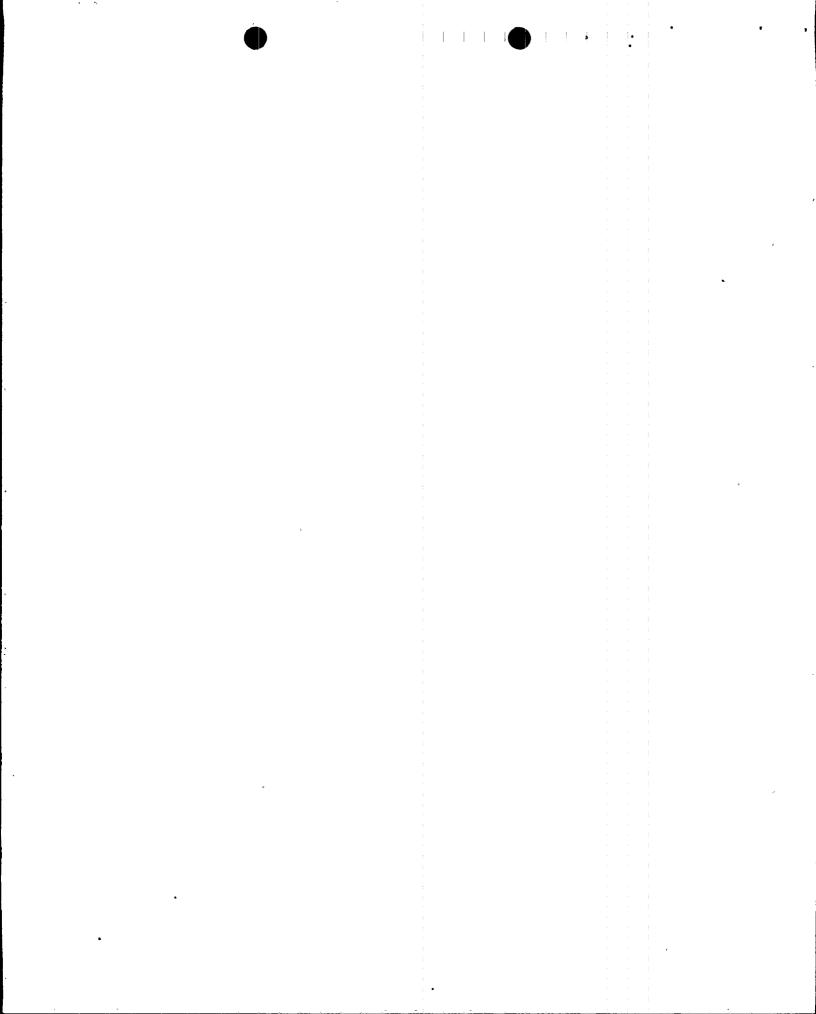
Robert E. Uhrig

Vice President

REU/MAS/cpc

Attachment

cc: Mr. Norman C. Moseley, Region II Robert Lowenstein, Esquire



E OF CONTENTS (continued) Т

Section	Title	Page
3.7 3.8 3.9	Electrical Systems Steam and Power Conversion Systems Radioactive Materials Release Liquid Wastes Gaseous Wastes Containerized Wastes	3.7-1 3.8-1 3.9-1 3.9-1 3.9-2 3.9-3
3.10 3.11 3.12 3.13	Refueling Miscellaneous Radioactive Materials Sources Cask Handling Air Treatment Sysyems	3.10-1 3.11-1 3.12-1 3.13-1
4 4.1 4.2 4.3 4.4	SURVEILLANCE REQUIREMENTS Operational Safety Review Reactor Coolant System In Service Inspection Reactor Coolant System Integrity Containment Tests Integrated Leakage Rate Test - Post Operational Local Panetration Tests Report of Test Results Isolation Valves Residual Heat Removal System	4.1-1 4.1-1 4.2-1 4.3-1 4.4-1 4.4-1 4.4-1 4.4-2 4.4-3 4.4-3 4.4-3
4.5 4.6	Tendon Surveillance End Anchorage Concrete Surveillance Liner Surveillance Safety Injection Emergency Containment Cooling Systems	4.4-4 4.4-6 4.4-7 4.5-1 4.6-1
4.7 4.8 4.9 4.10 4.11 4.12 4.13	Emergency Containment Filtering, Post Accident Containment Vent, and Control Room Ventilation Systems Emergency Power System Periodic Tests Main Steam Isolation Valves Auxiliary Feedwater System Reactivity Anomalies Environmental Radiation Survey Radioactive Materials Sources Surveillance	4.7-1 4.8-1 4.9-1 4.10-1 4.11-1 4.12-1 4.13-1
5 5.1 5.2 5.3 5.4	DESIGN FEATURES Site Reactor Containment Fuel Storage	5.1-1 5.1-1 5.2-1 5.3-1 5.4-1
6 6.1 6.2 6.3 6.4 6.5 6.6 6.7 6.8 6.9	ADMINISTRATIVE CONTROLS Responsibility Organization Facility Staff Qualifications Training Review and Audit Review and Audit Reportable Occurrence Action Safety Limit Violation Frozedures Reporting Requirements	6.1-1 6-1 6-5 6-5 6-5 6-14 6-14 6-14 6-16

ii

٠ . ſ ٢ ١

_ IADLE OF CONTENTS (continued)

ć

ļ

•		٩
• Section	Title .	Page
6-10	Record Retention	6–27
6-11 .	Radiation Protection Program	6-29
(-12, · · ·	Respiratory Protection Program	6-29
5-13	High Radiacion Area	6-33
10	- X	
B2.1	Bases for Safety Limit, Reactor Core	B2.1-1
B2.2	Bases for Safety Limit, Reactor Coolant System	
,	Pressure	B2.2-1
B2.3	Bases for Limiting Safety System Settings,	
	Protective Instrumentation	B2:3-1
B3.1 ,	Bases for Limiting Conditions for Operation,	• -
•	Reactor Coolant System	B3.1-L
B3.2	Bases for Limiting Conditions for Operation, Con-	
	trol Rod and Power Distribution Limits	B3.2-1
B3.3 .	Bases for Limiting Conditions for Operation,	50
DO 1	Containment	B3.3-1
B3.4	Bases for Limiting Conditions for Operation,	NA 4 - 1
D 2 C	Engineered Safety Features	B3.4-1
B3.5	Bases for Limiting Conditions for Operation,	na
D2 <i>L</i>	Instrumentation Recease for Limiting Conditions for Operation	B3.5-1
B3.6	Bases for Limiting Conditions for Operation,	D2 6 7 '
B3.7	Chemical and Volume Control System . Bases for Limiting Condition for Operation,	. B3.6-1
· · · ·	Electrical Systems	B3.7-1
B3.8	Bases for Limiting Conditions for Operation,	JJ • 1 L
	Steam and Power Conversion Systems	B3.8-1
B3.9	Bases for Limiting Conditions for Operation,	یکر ^س اله می مو ب
	Radicactive Materials Release	B3.9-1
B3.10	Bases for Limiting Conditions for Operation,	
* *	Refueling	B3.10-1
B3.11	Bases for Limiting Conditions for Operation.	•
ana an a an an an	Miscellaneous Radioactive Materials Sources	B3.11-1
B3.12 · ·	Bases for Limiting Conditions for Operation.	B3.12-1
	Cask Handling	D3+T7-T
B3.13	Bases for Limiting Conditions for Operation,	• _
CT + CC	Air Treatment Systems	B3.13-1
B4.1	Bases for Operational Safety Review	B4.1-1
B4.2	Bases for Reactor Coolant System In Service	
	Inspection	B4-2-1
B4.3	Bases for Reactor Coolant System Integrity'	. B4.3-1
B4.4	Bases for Containment Tests	B4.4-1
B4.5	Bases for Safety Injection Tests	B4.5-1
B4.6	Bases for Emergency Containment Cooling System Tests	B4.6-1
B4.7	Bases for Emergency Containment Filtering, Post	
	Accident Containment Vent, and Control Room	
	Ventilation Systems Tests	B4.7-1
B4.3	Zaras for Emergency Power System Periodic Tests	B4.8-1
B4.9	Easis for Main Steam Isolation Valve Tests	B4.9-1
B4.10	Essas for Auxiliary Feedwater System Tests	B4.10-1
B4.11	Bises for Reactivity Anomalies	B4.11-1
B4.12	Eases for Environmental Radiation Survey	B4.12-1
	· · · · ·	

. 6/10/77.

iii ·

•

\$

• •

- 1. ONE emergency containment cooling unit may be out of service for a period of 24 hours. Prior to initiating maintenance, the other TWO units shall be tested to demonstrate operability.
- 2. ONE containment spray pump may be out of service provided it is restored to operable status within 24 hours. The remaining containment spray pump shall be tested to demonstrate operability before initiating maintenance on the inoperable pump.
- 3. Any value in the system may be inoperable provided repairs are completed within 24 hours. Prior to initiating repairs, all values that provide the duplicate function shall be tested to demonstrate operability.

3. EMERGENCY CONTAINMENT FILTERING SYSTEM

During times when containment integrity is required:

- a. The three Emergency Containment Filtering Units shall be operable at all times or the reactor shall be placed in hot shutdown within 12 hours and in cold shutdown within the following 30 hours, with the following exception:
 - 1) If one of the three emergency containment filtering trains is made or found to be inoperable for any reason, reactor operation is permissable only during the succeeding seven (7) days provided that all active components of the other two trains shall be demonstrated to be operable within two hours and daily thereafter.
- b. Any value in the system may be inoperable provided repairs are completed within 24 hours except as stated in 3.a.l above. Prior to initiating maintenance, all values that provide the duplicate function shall be tested to demonstrate operability.

4. COMPONENT COOLING SYSTEM

a. The reactor shall not be made critical, except for low power physics tests, unless the following conditions are met.

• s

÷

3.13 AIR TREATMENT SYSTEMS

- Applicability: Applies to the operating status of the Control Room Ventilation System and the Post Accident Containment Vent System.
- <u>Objectives</u>: To define those limiting conditions for operation that are necessary:
 - 1. To remove airborne activity from the control room if detected.
 - To reduce containment hydrogen concentration and to reduce containment pressure during post-accident conditions.
- Specification: 1. CONTROL ROOM VENTILATION SYSTEM
 - a. The Control Room Ventilation (Emergency Internal Cleanup) System may be inoperable for a period of up to seven days. If the system cannot be made operable within seven days, both reactors shall be shut down.
 - 2. POST ACCIDENT CONTAINMENT VENT SYSTEM
 - a. The Post Accident Containment Vent System may be inoperable for a period up to thirty days. If the system cannot be made operable within 30 days, both reactors shall be shutdown.

6/10/77

 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 .
 . • ,

\$

٠

4.7 EMERCENCY CONTAINMENT FILTER SYSTEM, POST ACCIDENT CONTAINMENT VENT SYSTEM

AND CONTROL ROOM VENTILATION SYSTEM.

Applicability: Applies to the Emergency Containment Filter System, the

Post Accident Containment Vent System, the Control Room

Ventilation System.

Objectives:

To verify that these systems and their components will be able to perform their design functions.

In the event that painting, fire, or chemical release occurs such that the filters are exposed to the effluents of these events, the system will be tested to verify its performance

of design features.

Specification:

. Operating Tests

System tests shall be performed once per operating cycle

or once per 18 months, whichever comes first.

4.7.1 EMERGENCY CONTAINMENT FILTERING SYSTEM

The tests shall consist of pressure drop and flow measurements across all filters in the plenum. Less than 6" of water pressure drop at design flow (± 10%) across the combined HEPA filters and charcoal adsorbers shall constitute acceptable performance. Once per operating cycle, the Emergency Containment Filtering System shall be tested to demonstrate automatic initiation upon receipt of a safety injection signal. The Emergency Containment Filtering System shall be cperated monthly for at least 15 minutes to demonstrate

cperability. .

6/10/77

4.7-1

•

le .

2. Performance Tests

- At least once per 18 months or after every 720 hours of system a. operation, in-place DOP and halogenated hydrocarbon tests on each Emergency Containment Filter plenum shall be performed. In addition, in-place DOP and halogenated hydrocarbon tests shall be performed after (1) any structural maintenance on system housings, (2) after any testing which might have affected filter bank efficiency, (3) after complete or partial replacement of a filter bank, or (4) after exposure of the filters to effluents from painting, fire, or chemical release. Removal of >99% DOP and >99% halogenated hydrocarbon shall constitute acceptable performance. Once per operating cycle, a charcoal surveillance specimen from one of the emergency containment filters shall be analyzed for elemental iodine removal capability. The results of the laboratory carbon sample analysis shall show >99.9% removal efficiency. Failing this, the charcoal shall be replaced with charcoal which meets or exceeds original specifications.
- b. An air distribution test shall be performed at least once, and thereafter only if there is indication that the air distribution has changed.

4.7.2 POST ACCIDENT CONTAINMENT VENT SYSTEM

1. Operating Tests

Operating tests shall be performed annually. The tests shall consist of visual inspection of the system, operation of all valves, and pressure drop and air flow measurements. Visual inspection shall include search for any foreign materials and gasket deterioration in the HEPA filters and charcoal adsorbers. Less than 6" of water pressure drop at 55 cfm flow shall constitute acceptable performance.

2. Performance Tests

At least once per 18 months or after 720 hours of system operation, in-place DOP and halogenated hydrocarbon tests on the Post Accident Containment Vent filters shall be performed. In addition, in-place DOP and halogenated hydrocarbon tests shall be performed after (1) any structural maintenance on system housings, (2) after any testing which might have affected filter bank efficiency, (3) after complete or partial replacement of a filter bank, or (4) after exposure of the filters to effluents from painting, fire or chemical release. Removal of >99% DOP and >99% halogenated hydrocarbon shall constitute acceptable performance.

The hydrogen concentration measurement instrument shall be calibrated with proper consideration for humidity.

6/10/77

* • • •

4.7.3 CONTROL ROOM VENTILATION (EMERGENCY INTERNAL CLEARNUP) SYSTEM

1. Operating Tests

The Control Room Ventilation System shall be operated monthly for at least 15 minutes to demonstrate operability. Auto initiation of the systems operations shall be checked annually. Pressure drop measurements across the filter bank shall be made annually. Less than 6" of water pressure drop at designed flow (±10%) across the combined HEPA filter and charcoal adsorbers shall constitute acceptable performance.

2. Performance Tests

At least once per 18 months or after 720 hours of system operation, in-place DOP and halogenated hydrocarbon tests on the filters shall be performed. In addition, in-place DOP and halogenated hydrocarbon tests shall be performed after (1) any structural maintenance on system housings, (2) after any testing which might have affected filter bank efficiency, (3) after complete or partial replacement of a filter bank, or (4) after exposure of the filters to effluents from painting, fire, or chemical release. Removal of >99% DOP and >99% halogenated hydrocarbon shall constitute acceptable performance.

A charcoal surveillance specimen from one of the charcoal adsorbers shall be removed and analyzed annually for methyl iodide removal capability. The results of the laboratory carbon sample analysis shall show >85% removal efficiency with >95% relative humidity and >130°C. Failing this, the charcoal shall be replaced with charcoal which meets or exceeds original specifications.

6/10/77

· | | | • • • • • • . • ۲ • ~ •

B3.13 BASES FOR LIMITING CONDITIONS FOR OPERATION, AIR TREATMENT SYSTEMS

The Control Room Ventilation (Emergency Internal Cleanup) System is designed to partially filter the Control Room atmosphere for recirculation during Control Room isolation conditions. If the system is inoperable, there is no immediate threat to the Control Room and reactor operation may continue for a limited time while repairs are being made. If the system cannot be repaired within seven days, both reactors are shutdown in accordance with 10 CFR 50.36(c).

The Post Accident Containment Vent System is designed to facilitate controlled venting of either containment through HEPA and charcoal filters to reduce combustible gas concentration following a loss of coolant accident. If the system is inoperable, there is no significant threat caused by continued reactor operation because the system would only be used seven to ten days after an accident. Reactor operation may, therefore, continue for up to thirty days while repairs are being made. If the system cannot be repaired in that time, both reactors are shutdown in accordance with 10 CFR 50.36(c).

6/10/77

• * : • : • ٠ ÷ , e

٠

١

B 4.7

BASES FOR EMERGENCE CONTAINMENT FILTERING SYSTEM, VENT SYSTEM, CONTROL ROOM VENTILATION SYSTEM.

System components are not subject to rapid deterioration, having lifetimes of many years, even under continuous flow conditions. Visual inspection and operating tests provide assurance of system reliability and will insure early detection of conditions which could cause the system to fail or operate improperly. The performance tests prove conclusively that filters have been properly installed, that no deterioration or damage has occurred, and that all components and subsystems operate properly. The tests are performed in accordance with the methodology and intent of ANSI 410, 1975, and provide assurance that filter performance has not deteriorated below original specification values due to aging, contamination, or other effects.

6/10/77

'n

a

STATE OF FLORIDA)) COUNTY OF DADE)

.ss.

Robert E. Uhrig, being first duly sworn, deposes and says:

That he is a Vice President of Florida Power & Light Company, the Licensee herein;

That he has executed the foregoing document; that the statements made in this said document are true and correct to the best of his knowledge, information, and belief, and that he is authorized to execute the document on behalf of said Licensee.

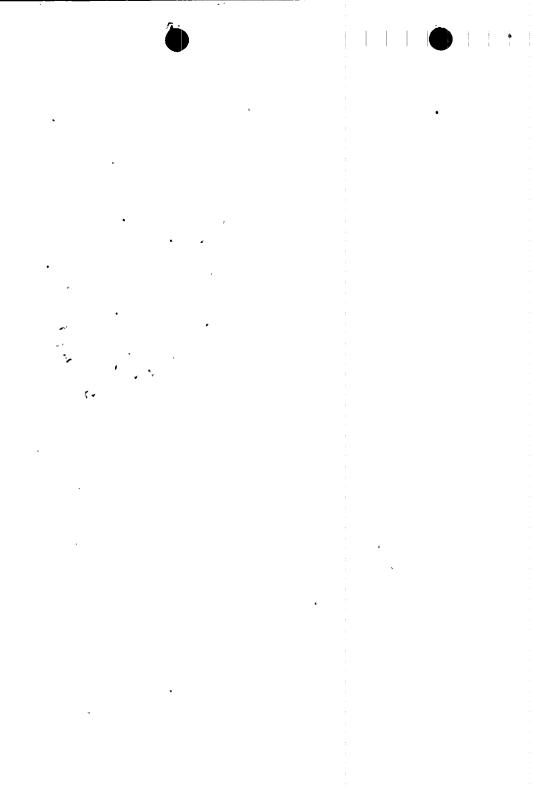
Robert Ε. Uhrio

Subscribed and sworn to before me this

, 19 77 day of

NOTARY PUBLIC, in and for the County of Dade, State of Florida

My commission expires: My commission expires: My commission expires:



•

.

· ·

.

•

FLORIDA POWER & LIGHT COMPANY

June 9, 1977 L-77-174

Director of Nuclear Reactor Regulation Attention: Mr. Victor Stello, Director Division of Operating Reactors U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Dear Mr. Stello:

Re: Turkey Point Units 3 and 4 Docket Nos. 50-250 and 50-251 Proposed Amendment to Facility Operating Licenses DPR-31 and DPR-41



In accordance with 10 CFR 50.30, Florida Power & Light Company submits herewith three (3) signed originals and forty (40) copies of a request to amend Appendix A of Facility Operating Licenses DPR-31 and DPR-41.

In response to telephone requests from members of your staff, we have revised our proposal of May 3, 1976 on the subject of air filtration systems. The present proposal conforms as near as possible to the Standard Technical Specifications without our having to make hardware modifications. The proposed changes are described below and shown on the accompanying Technical Specification pages bearing the date of this letter in the lower right hand corner.

Pages ii and iii

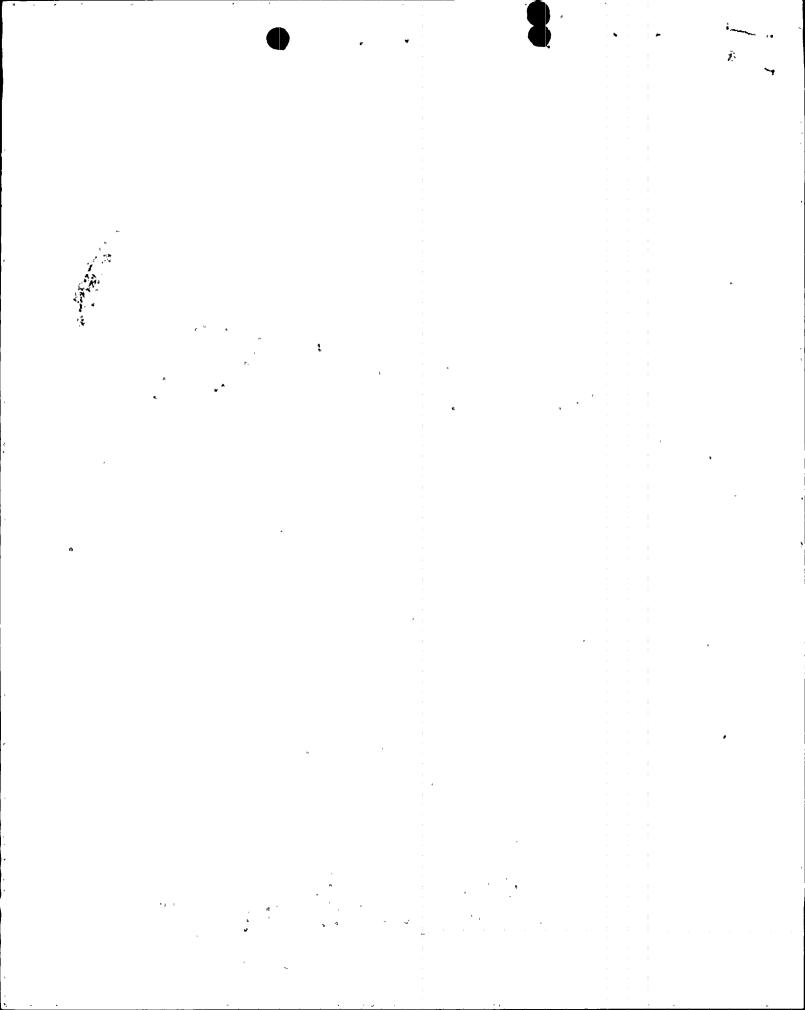
Revised Table of Contents.

Page 3.4-4

Specification 3.4.3.b is revised. This specification concerns the limiting conditions for operation applicable to the Emergency Containment Filtering System during power operation.

Page 3.13-1

New specification 3.13 is added. This specification concerns the limiting conditions for operation applicable to the Control Room Ventilation System and the Post Accident Containment Vent System.



Director of Nuclear Reactor Regulation Page Two

Pages 4.7-1 through 4.7-3

The surveillance requirements of specifications 4.7.1 (Emergency Containment Filtering System) and 4.7.2 (Post Accident Containment Vent System) are revised. A new specification 4.7.3 (Control Room Ventilation System) is added.

Page B3.13-1

A new section B3.13-1 is added to provide bases for new specification 3.13.

Page B4.7-1

The bases for specification 4.7 are revised to include the Control Room Ventilation System.

The proposed amendment has been reviewed by the Turkey Point Plant Nuclear Safety Committee and the Florida Power & Light Company Nuclear Review Board and the conclusion reached that it does not involve an unreviewed safety question.

Very truly yours,

Robert E. Uhrig Vice President

REU/MAS/cpc

Attachment

cc: Mr. Norman C. Moseley, Region II Robert Lowenstein, Esquire



(2.70)		4						50-250/251	
	NRC DISTRIBUT	101	V FOR	T 50 DOCKET	MAT	TERIAL	TELL	5-NUMUCA	
TO: Mr. Victor Stello			FROM: Florida Power & Light Company			DA	DATE OF DOCUMENT		
			Miami, Fla.	•		DA	TE RECEIVED		
			Mr. Robert E. Uhrig			<u> </u>	6/13/77		
DETTER DINOTORIZED DORIGINAL DUNCLASSIFIED DCOPY			PROP INPUT FORM			NU	MBER OF COPIES RECEIVED		
				•		1	3 SIGNER		
DESCRIPTI	DESCRIPTION:					CLOSURE			
	*	**		4	-		~	•	
· · · · · ·	L.		· • •		•	Amdt. to OL/change to		pendix A tech	
					•	specssummitted as a result of a \cdot , \cdot			
	*) 		ł			re-evaluation_of_ECCS notorized 6/8/77.	co	oling performance	
				• •			:,		
	• •		• •	,					
	· · ·		· · ·	•••		· DO NO	ጋፐ	REMOVE' 1	
	۰ ۰		-	(1-P)		(28-P)		TOTATATO A HI, 1	
DT ANT	NAME:		•	· · ·		(20-1) · · · · · · ·		· · · · · · · · · · · · · · · · · · ·	
	•		•••		•	ACKN			
Turke	y Point Units 3	& 4	• •	*		440477A	v) vi	/LEDGED	
RJL	6/14/77	,n	• •		.				
			•	•		•			
	a "			•		• * .	•	<u> </u>	
- i Acerc	SAFETY		<u> </u>	. FOR ACTION/	INFO		<u>) </u>		
	SNED AD:		Lease	(5)		ASSIGNED AD:			
	CT MANAGER:			F+,	BRANCH_CHIEF: PROJECT_MANAGER:				
a Shere a subscript of the second se	LIC. ASST.:			rrish	LIC. ASST.:				
	• • •								
-			·					•	
RES_F	T.I.P		SYSTEMS	INTERNAL D					
NRC P			HEINEMAN				<u> </u>	SITE SAFETY & ENVIRO ANALYSIS	
I&E	721		SCHROEDE			TEDESCO BENAROYA		DENTON & MULLER	
OELD				•		LAINAS			
	CK & STAFF		ENGINEER			IPPOLITO ·		ENVIRO TECH	
MIPC			MACCARRY			KIRKWOOD	- <u> </u>	_ERNST	
CASE HANAU		├	KNIGHT ·SIHWEI			(NDIST & CONTA DE LA COMAN A			
JIARLE			PAWLICKI	· · ·	┝╶╾╋	OPERATING REACTORS STELLO		SPANGLER	
								SITE_TECH	
	CT MANAGEMENT		REACTOR	SAFETY		OPERATING TECH.		GAMMILL	
BOYD	· · · · · · · · · · · · · · · · · · ·		ROSS	•	4	EISENHUT		STEPP	
	I.LINS	<u> </u>	NOVAK	v	4	SIIAO		_HUILMAN	
HOUST PETER			ROSZTOCZ CHECK	<u>I</u>	·/-	BAER	;	CTTE ANA 1070 '	
MELTZ	a and at a subscription of the second se		,	,		<u>BUTLER</u> CRIMES		SITE ANALYSIS	
HELTE	مى مى خىلى بىلى تى		AT & I		ř-ł	XXALUX	1	BUNCH	
SKOVI			SALTZMAN			* •	Z	_J. COLLINS	
			RUTBERG				4	KREGUR	
- An mini-	MIAM Plu	1		DISTRIBUTION	······			CONTROL NUMBER	
TIC:	YIIIAM, KIN		NAT LAB:			BROOKHAVEN NAT LAB	-	· ·	
NSIC:	- 		REG. VIE			ULRIKSON(ORM.)	1.		
ASLB			CONSULTA	NTS · A·		میں ہوتی ہوتی ہے۔ 1	-	171650242	
ACRS	(CCYS -HOLDING!	(i:N	ASC	ATD			-] .	1	
	- - 14 - 24 - 10 - 10 - 10 - 10 - 10 - 10 - 10 - 1	1	ي ال عدد الا مالية مارد. معاد الدينية المالية المالية الم	******		لاين التي المحمد ال الما المحمد ال]		
HIG FORM	10n (2-74)				_	•		•	

