

ATTACHMENT TO LICENSE AMENDMENT NO. 139

TO FACILITY COMBINED LICENSE NO. NPF-92

DOCKET NO. 52-026

Replace the following pages of the Facility Combined License No. NPF-92 with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Combined License No. NPF-92

REMOVE

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Appendix C to Facility Combined License No. NPF-92

REMOVE

C-255

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C-255

(7) Reporting Requirements

- (a) Within 30 days of a change to the initial test program described in UFSAR Section 14, Initial Test Program, made in accordance with 10 CFR 50.59 or in accordance with 10 CFR Part 52, Appendix D, Section VIII, "Processes for Changes and Departures," SNC shall report the change to the Director of NRO, or the Director's designee, in accordance with 10 CFR 50.59(d).
- (b) SNC shall report any violation of a requirement in Section 2.D.(3), Section 2.D.(4), Section 2.D.(5), and Section 2.D.(6) of this license within 24 hours. Initial notification shall be made to the NRC Operations Center in accordance with 10 CFR 50.72, with written follow up in accordance with 10 CFR 50.73.

(8) Incorporation

The Technical Specifications, Environmental Protection Plan, and ITAAC in Appendices A, B, and C, respectively of this license, as revised through Amendment No. 139, are hereby incorporated into this license. |

(9) Technical Specifications

The technical specifications in Appendix A to this license become effective upon a Commission finding that the acceptance criteria in this license (ITAAC) are met in accordance with 10 CFR 52.103(g).

(10) Operational Program Implementation

SNC shall implement the programs or portions of programs identified below, on or before the date SNC achieves the following milestones:

- (a) Environmental Qualification Program implemented before initial fuel load;
- (b) Reactor Vessel Material Surveillance Program implemented before initial criticality;
- (c) Preservice Testing Program implemented before initial fuel load;
- (d) Containment Leakage Rate Testing Program implemented before initial fuel load;
- (e) Fire Protection Program
 - 1. The fire protection measures in accordance with Regulatory Guide (RG) 1.189 for designated storage building areas (including adjacent fire areas that could affect the storage area) implemented before initial receipt

Table 2.3.10-4 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
440	2.3.10.05b	Not used per Amendment No. 85		
441	2.3.10.06a	Not used per Amendment No. 85		
442	2.3.10.06b	Not used per Amendment No. 85		
443	2.3.10.07a.i	Not used per Amendment No. 112		
444	2.3.10.07a.ii	7.a) The WLS provides the nonsafety-related function of detecting leaks within containment to the containment sump. 7.b) The WLS provides the nonsafety-related function of controlling releases of radioactive materials in liquid effluents. 8. Controls exist in the MCR to cause the remotely operated valve identified in Table 2.3.10-3 to perform its active function. 9. The check valves identified in Table 2.3.10-1 perform an active safety-related function to change position as indicated in the table. 10. Displays of the parameters identified in Table 2.3.10-3 can be retrieved in the MCR.	i) Inspection will be performed for retrievability of the displays of containment sump level channels WLS-034, WLS-035, and WLS-036 in the MCR. ii) Testing will be performed by adding water to the sump and observing display of sump level. Tests will be performed to confirm that a simulated high radiation signal from the discharge radiation monitor, WLS-RE-229, causes the discharge isolation valve WLS-PL-V223 to close. Stroke testing will be performed on the remotely operated valve listed in Table 2.3.10-3 using controls in the MCR. Exercise testing of the check valves with active safety functions identified in Table 2.3.10-1 will be performed under pre-operational test pressure, temperature and flow conditions. Inspection will be performed for retrievability of the displays identified in Table 2.3.10-3 in the MCR.	i) Nonsafety-related displays of WLS containment sump level channels WLS-034, WLS-035, and WLS-036 can be retrieved in the MCR. ii) A report exists and concludes that sump level channels WLS-034, WLS-035, and WLS-036 can detect a change of 1.34 ± 0.5 inches. A simulated high radiation signal causes the discharge control isolation valve WLS-PL-V223 to close. Controls in the MCR operate to cause the remotely operated valve to perform its active function. Each check valve changes position as indicated on Table 2.3.10-1. Displays identified in Table 2.3.10-3 can be retrieved in the MCR.
445	2.3.10.07b	Not used per Amendment No. 112		
446	2.3.10.08	Not used per Amendment No. 112		
447	2.3.10.09	Not used per Amendment No. 112		