

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL  
(TEMPORARY FORM)

CONTROL NO: 1146

FILE: INCIDENT FILE

FROM: Indiana & Michigan Power Bridgman, Michigan R.W. Jurgensen			DATE OF DOC 1-29-76	DATE REC'D 2-5-76	LIB XXX	TWX	RPT	OTHER
TO: J.G. Keppler			ORIG 1 Signed	CC 0	OTHER	SENT NRC PDR SENT LOCAL PDR		XXX XXX
CLASS XXX	UNCLASS	PROP INFO	INPUT	NO CYS REC'D 1		DOCKET NO: -50-315		

DESCRIPTION:  
Letter trans the following.....

PLANT NAME: Cook # 1

ENCLOSURES:  
Reportable Occurrence # 76-03, on 1-2-76,  
Concerning Damaged Containment Crane Wall...

(1 Copy Received) **ACKNOWLEDGED**

**DO NOT REMOVE**

FOR ACTION/INFORMATION

SAB 2-9-76

BRANCH CHIEF Kniel W/3

LIC. ASST. Service W/16 cys ACRS

INTERNAL DISTRIBUTION

**REG FILE**  
NRC PDR  
I&E (2)  
MIPC (3)  
SCHRODER/IPPOLITO  
HOUSTON  
NOVAK/CHECK  
GRIMES/SCHWENCER  
CASE  
F. WILLIAMS  
HANAUER  
TEDESCO/MACCARY  
EISENHUT  
BAER  
SHAO

VOLLMER/BUNCH  
KREGER/J. COLLINS

NOTE: IF PERSONEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

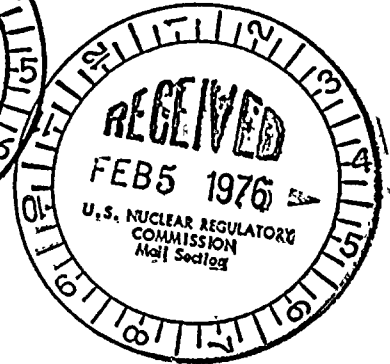
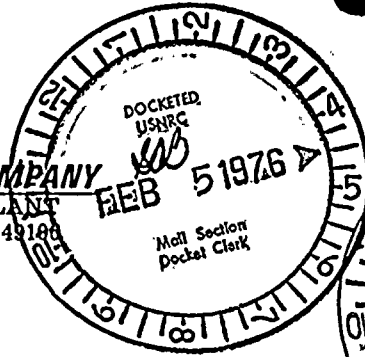
EXTERNAL DISTRIBUTION

LOCAL PDR St. Joseph, Michigan  
TIC  
NSIC



[The page contains extremely faint and illegible text, likely bleed-through from the reverse side of the document. The text is scattered across the page and does not form any recognizable words or sentences.]


**INDIANA & MICHIGAN POWER COMPANY**

 DONALD C. COOK NUCLEAR PLANT  
 P.O. Box 458, Bridgman, Michigan 49119


January 29, 1976

 Mr. J.G. Keppler, Regional Director  
 Office of Inspection and Enforcement  
 United States Nuclear Regulatory Commission  
 Region III  
 799 Roosevelt Road  
 Glen Ellyn, IL 60137

 Operating License DPR-58  
 Docket No. 50-315

Dear Mr. Keppler:

Pursuant to the requirements of Appendix "A" Technical Specifications and the United States Nuclear Regulatory Commission Regulatory Guide 1.16, Revision 3, Section 2a 9c, the attached report is submitted:

RO 50-315/76-03

This event was discovered on January 2, 1976. It was verbally reported to RO: III Resident Inspector, Mr. K.R. Baker, on January 3, 1976. At the time of the verbal report we had determined that the formal written report would have been required to be submitted within 30 days of the occurrence of the event as required by Regulatory Guide 1.16, Revision 3, Section 2.b. Subsequent review of Regulatory Guide 1.16, Revision 3, Section 2a 9c revealed this report should have been submitted prior to January 16, 1976.

This report is late in being submitted due to our misinterpretation of the requirements of Regulatory Guide 1.16, Revision 3.

Sincerely,

  
 R.W. Jurgensen  
 Plant Manager

/bab

 cc: R.S. Hunter  
 J.E. Dolan  
 G.E. Lien  
 R. Kilburn  
 R.J. Vollen BPI

Continued...

FEB 2 1976

1146

1941-1942

1941-1942

R.C. Callen MPSC  
K.R. Baker RO: III  
P.W. Steketee, Esq.  
R. Walsh, Esq.  
G. Charnoff, Esq.  
G. Olson  
J.M. Hennigan  
PNSRC  
R.S. Keith  
Dir., IE (30 copies)  
Dir., MIPC (3 copies)





)

EVENT DESCRIPTION - Continued

the common reservoir for the two 1-FWS-10 snubbers was slightly bent. A third snubber at this location, 1-FWS-11 was found with the feedpipe from the reservoir to the control valve assembly snapped off at the reservoir.

The two 1-FWS-10 hydraulic snubbers were replaced with identical units and a replacement reservoir and feedpipe were installed on 1-FWS-11.

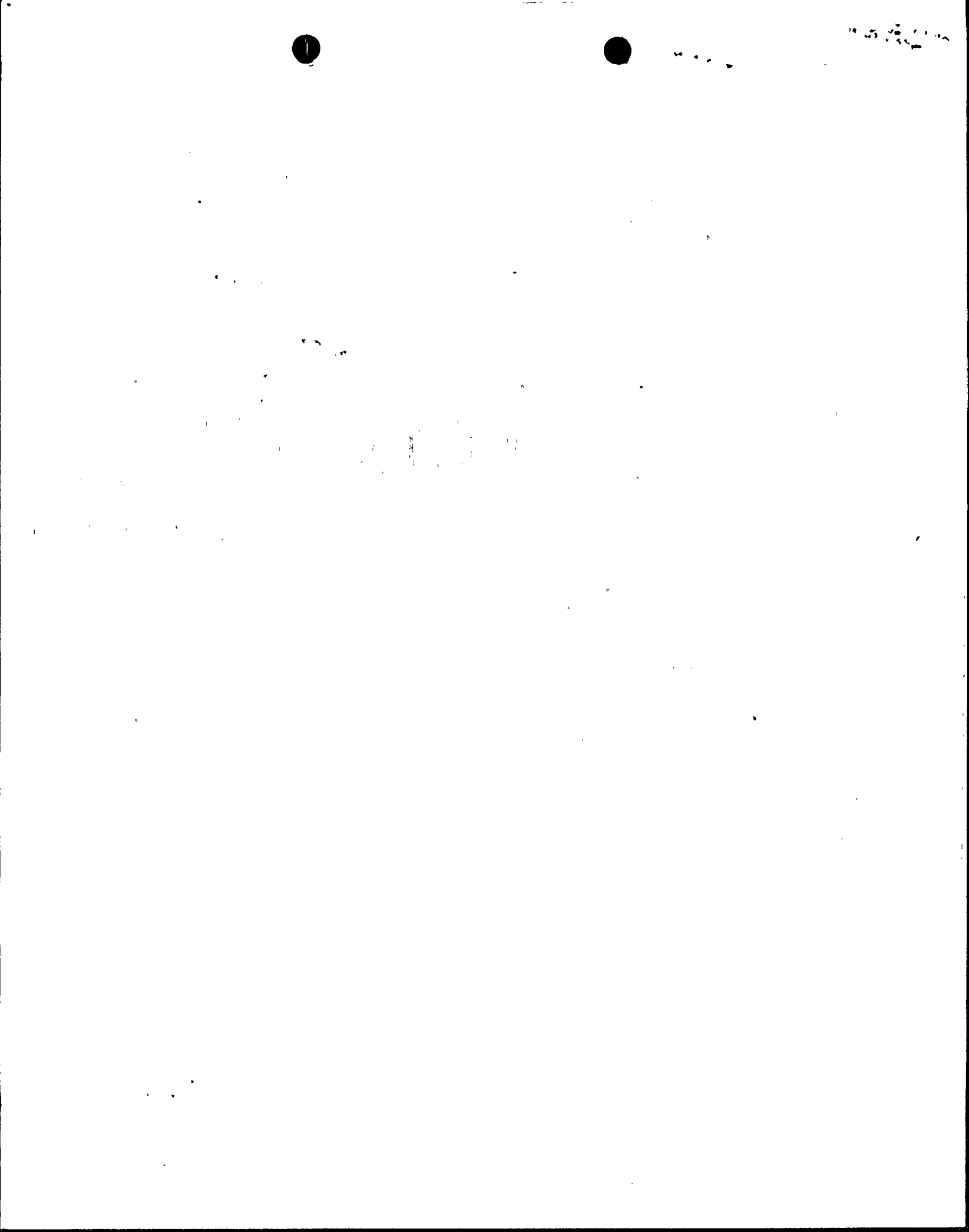
CAUSE DESCRIPTION - Continued

Prior to the November 14, 1975, startup, an extensive containment inspection was made which would have readily revealed the observed snubber damage had it occurred prior to that date. Inspection of the feedwater line from the steam generator to the containment wall penetration was performed. All welds were either dye penetrant or magnetic particle tested. Piping found in satisfactory condition.

ADDITIONAL FACTORS - Continued

water hammer shock loading.





NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL  
(TEMPORARY FORM)

CONTROL NO: 1036

FILE: INCIDENT FILE

FROM: END & MICH POWER COMPANY BRIDGMAN, MICH R W JURGENSEN		DATE OF DOC 1-29-76	DATE REC'D 2-3-76	LTR XXXXXX	TWX	RPT	OTHER
TO: MR J G KEPPLER		ORIG	CC 1	OTHER	SENT NRC PDR <u>xxx</u>		SENT LOCAL PDR <u>xxx</u>
CLASS	UNCLASS XXXXXX	PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: 50-315		

DESCRIPTION:  
LTR TRANS THE FOLLOWING.....

PLANT NAME: BRUNSWICK UNIT 2

ENCLOSURES:  
LICENSE EVENT REPORT, RO-50-315/76-03  
ON 1-2-76 RE DAMAGE HYDRAULIC SNUBBERS  
ON MAIN WATERFEED TO #4 SG.....

(1 CY OF ENCL REC'D)

**DO NOT REMOVE  
ACKNOWLEDGED**

FOR ACTION/INFORMATION RKB 2-4-76

BRANCH CHIEF PURPLE (3)

LIC. ASST. SHEPPARD W/16 cys ACRS

INTERNAL DISTRIBUTION

- REG FILE**
- NRC PDR
- I&E (2)
- MIPC (3)
- SCHRODER/IPPOLITO
- HOUSTON
- NOVAK/CHECK
- GRIMES/SCHWENCER
- CASE
- F. WILLIAMS
- HANAUER
- TEDESCO/MACCARY
- EISENHUT
- BAER
- SHAO

- VOLLMER/BUNCH
- KREGER/J. COLLINS

NOTE: IF PERSONEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

LOCAL PDR SOUTHPORT, NC

TIC

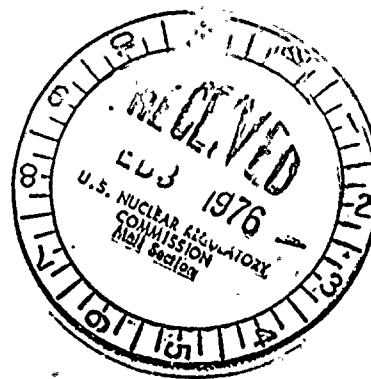
NSIC





**INDIANA & MICHIGAN POWER COMPANY**

DONALD C. COOK NUCLEAR PLANT  
P.O. Box 458, Bridgman, Michigan 49106



January 29, 1976

**Regulatory Docket File**

Mr. J.G. Keppler, Regional Director  
Office of Inspection and Enforcement  
United States Nuclear Regulatory Commission  
Region III  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Operating License DPR-58  
Docket No. 50-315

Dear Mr. Keppler:

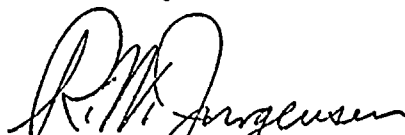
Pursuant to the requirements of Appendix "A" Technical Specifications and the United States Nuclear Regulatory Commission Regulatory Guide 1.16, Revision 3, Section 2a 9c, the attached report is submitted:

RO 50-315/76-03

This event was discovered on January 2, 1976. It was verbally reported to RO: III Resident Inspector, Mr. K.R. Baker, on January 3, 1976. At the time of the verbal report we had determined that the formal written report would have been required to be submitted within 30 days of the occurrence of the event as required by Regulatory Guide 1.16, Revision 3, Section 2.b. Subsequent review of Regulatory Guide 1.16, Revision 3, Section 2a 9c revealed this report should have been submitted prior to January 16, 1976.

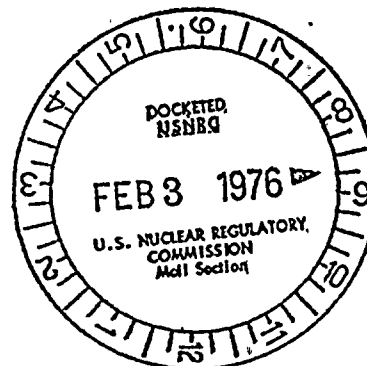
This report is late in being submitted due to our misinterpretation of the requirements of Regulatory Guide 1.16, Revision 3.

Sincerely,

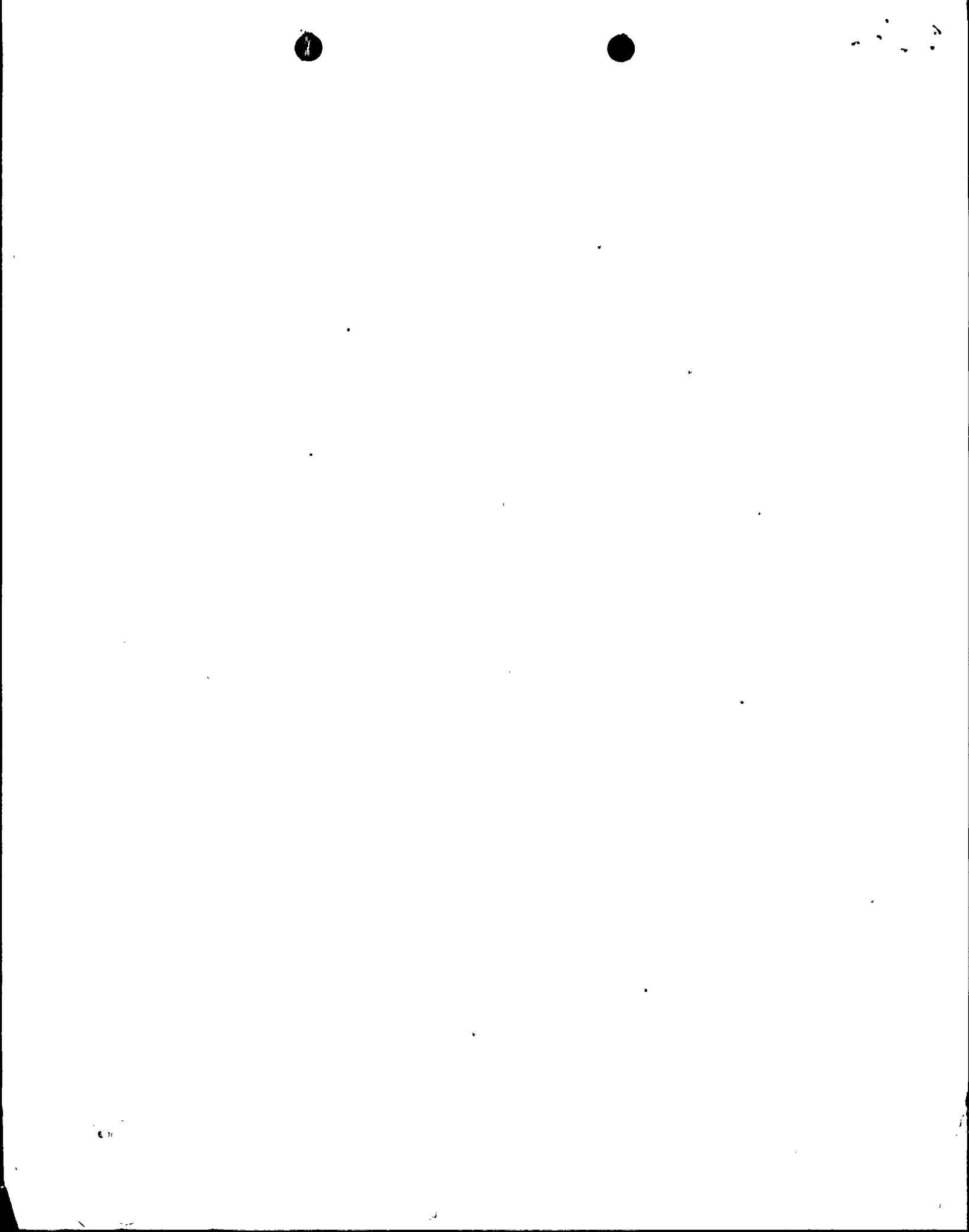
  
R.W. Jurgensen  
Plant Manager

/bab

- cc: R.S. Hunter
- J.E. Dolan
- G.E. Lien
- R. Kilburn
- R.J. Vollen BPI



Continued...



R.C. Callen MPSC  
K.R. Baker RO: III  
P.W. Steketee, Esq.  
R. Walsh, Esq.  
G. Charnoff, Esq.  
G. Olson  
J.M. Hennigan  
PNSRC  
R.S. Keith  
Dir., IE (30 copies)  
Dir., MIPC (3 copies)



2  
3  
4  
5

8

10  
11  
12

13

# LICENSEE EVENT REPORT

CONTROL BLOCK: 

--	--	--	--	--	--

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME						LICENSE NUMBER										LICENSE TYPE					EVENT TYPE			
01	M	I	D	C	C	1	0	0	-	0	0	0	0	0	0	0	0	4	1	1	1	1	0	1
7	8	9				14	15											26				30	31	32
CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER					EVENT DATE				REPORT DATE											
01	CONT	T	L	0	5	0	-	0	3	1	1	5	0	1	0	2	7	6	0	1	2	9	7	6
7	8			57	58	59	60	61					68	69				74	75					80

## EVENT DESCRIPTION

02 | Hydraulic snubbers on the 14" main feedwater line to #4 Steam Generator inside the  
03 | containment crane wall were found to be damaged. The upper snubber on 1-FWS-10 was  
04 | found broken. Breakage occurred in the hydraulic fittings from the control valve  
05 | assembly to the cylinder end blocks. The control valve assembly on the lower  
06 | snubber for 1-FWS-10 was intact but the hydraulic tubing from (Continued on next page)

SYSTEM CODE		CAUSE CODE		COMPONENT CODE			PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER			VIOLATION			
07	C	H	E	H	A	N	G	E	R	0	G	2	5	5	N
7	8	9	10	11	12			17		43	44			47	48

## CAUSE DESCRIPTION

08 | An apparent water hammer in the feedwater line to #4 Steam Generator is the probable  
09 | cause of this problem. There have been no known water hammers on #4 Steam Generator  
10 | since the unit was returned to service on November 14, 1975. (Continued on next page)

FACILITY STATUS		% POWER			OTHER STATUS			METHOD OF DISCOVERY		DISCOVERY DESCRIPTION			
11	G	0	0	0	NA			B	NA				
7	8	9	10	12	13			44	45	46			80
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY			LOCATION OF RELEASE						
12	Z	Z	NA				NA						
7	8	9	10	11			44	45					80

## PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION	
13	0	0	0	Z	NA
7	8	9	11	12	13

## PERSONNEL INJURIES

NUMBER		DESCRIPTION		
14	0	0	0	NA
7	8	9	11	12

## PROBABLE CONSEQUENCES

15 | NA

## LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION	
16	Z	NA	
7	8	9	10

## PUBLICITY

17 | NA

## ADDITIONAL FACTORS

18 | We are presently in the process of performing a dynamic analysis to determine the  
19 | piping movements that could occur in this system when subject to (Continued on next page)

NAME: D. V. SMALLER

PHONE: 616-465-5901



EVENT DESCRIPTION - Continued

the common reservoir for the two 1-FWS-10 snubbers was slightly bent. A third snubber at this location, 1-FWS-11 was found with the feedpipe from the reservoir to the control valve assembly snapped off at the reservoir.

The two 1-FWS-10 hydraulic snubbers were replaced with identical units and a replacement reservoir and feedpipe were installed on 1-FWS-11.

CAUSE DESCRIPTION - Continued

Prior to the November 14, 1975, startup, an extensive containment inspection was made which would have readily revealed the observed snubber damage had it occurred prior to that date. Inspection of the feedwater line from the steam generator to the containment wall penetration was performed. All welds were either dye penetrant or magnetic particle tested. Piping found in satisfactory condition.

ADDITIONAL FACTORS - Continued

water hammer shock loading.

