REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS) DISTRIBUTION FOR INCOMING MATERIAL 50-316

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DOCDATE: 08/02/78 DATE RCVD: 08/07/78

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SUBJECT:

LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-316/78-048) ON 07/06/78 CONCERNING AFTER REACTOR TRIPS ON MARCH 25, JUNE 14, 19 , JULY 6, 14, 1978, ROD POSITION INDICATION FOR CONTROL ROD H-8 DID NOT INDICATE THAT H-8 WAS FULLY

INSERTED...W/ATT LER 78-029.

PLANT NAME: COOK - UNIT 2

REVIEWER INITIAL: MLX DISTRIBUTOR INITIAL: ~

********** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS ***********

NOTES:

1. SEND 3 COPIES OF ALL MATERIAL TO 1&E

INCIDENT REPORTS (DISTRIBUTION CODE A002)

FOR ACTION:

BR CHIEF ORB#1 BC**W/4 ENCL

INTERNAL: .

REG FILE***W/ENCL I & E**W/3 ENCL

I & C SYSTEMS BR**W/ENCL

NOVAK/CHECK**W/ENCL AD FOR ENG**W/ENCL HANAUER**W/ENCL

AD FOR SYS & PROJ**W/ENCL ENGINEERING BR**W/ENCL KREGER/J. COLLINS***W/ENCL

K SEYFRIT/IE**W/ENCL

EXTERNAL:

LPDR'S

ST. JOSEPH, MI**W/ENCL TIC, LIZ CARTER**W/ENCL

NSIC**W/ENCL

ACRS CAT B**W/16 ENCL

NRC PDR**W/ENCL MIPC**W/3 ENCL

EMERGENCY PLAN BR**W/ENCL

EEB**W/ENCL

PLANT SYSTEMS BR**W/ENCL

AD FOR PLANT SYSTEMS**W/ENCL

REACTOR SAFETY BR**W/ENCL VOLLMER/BUNCH**W/ENCL POWER SYS BR**W/ENCL

DISTRIBUTION: SIZE: 1P+1P+2P LTR 45

ENCL 45

THE END

4 2 5 6 3 8

D. Janhanc

Power INDIANA & MICHIGAN POWER COMPANY
System INDIANA & MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT P.O. Box 458, Bridgman, Michigan 49106

August 2, 1978

Mr. J.G. Keppler, Regional Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

NO 10 TH 8 4;

Operating License DPR-74 Docket No. 50-316

Dear Mr. Keppler:

Pursuant to the requirements of the Appendix A Technical Specifications the following reports are submitted:

R0 78-029/03L-1 R0 78-048/03L-0

Sincerely,

For D.V. Shaller Plant Manager

/sjk

cc: J.E. Dolan

R.W. Jurgensen

R.F. Kroeger

Lucussa

R. Kilburn

R.J. Vollen BPI

K.R. Baker RO:III

R.C. Callen MPSC

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R. Walsh, Esq.

G. Charnoff, Esq.

G. Olsen

J.M. Hennigan

PNSRC

J.F. Stietzel

R.S. Keith

Dir., IE (30 copies)

Dir., MIPC (3 copies)

AUG 4 1978

AU 4 A003/11

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'A ' A'	LICENSEE EVENT REPORT
<u>.</u> .	CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1	M I D C C 2 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0
CON'T 0 1 7 8	REPORT L 6 0 5 0 0 0 0 3 1 6 7 0 5 1 1 7 8 8 0 8 0 2 7 8 9 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
0 2	DURING NORMAL OPERATION, ON MAY 11 AND 14, 1978, THE MAIN TURBINE STOP
0 3	VALVE "D" CLOSURE LIMIT SWITCH BECAME INOPERABLE. THIS SWITCH PROVIDES A TRIP
04	SIGNAL LISTED IN T. S. TABLE 3.3-1 ITEM 18.B. IN BOTH OCCASIONS, THE APPLICABLE
0 5	ACTION REQUIREMENTS WERE FULFILLED., THEN THE LIMIT SWITCH WAS REPAIRED AND
06	RETURNED TO OPERATION.
0 7	
08	
0 9	SYSTEM CAUSE SUBCODE COMPONENT CODE SUBCODE SU
	17) REPORT NUMBER 21 22 23 24 26 27 28 29 30 31 31 32
	ACTION FUTURE ON PLANT SHUTDOWN HOURS 22 ATTACHMENT FORM SUB. PRIME COMP. COMPONENT MANUFACTURER SUBMITTED FORM SUBMITTED FOR
110	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) 1 THE FAILURE OF THE MAIN TURBINE STOP VALVE LIMIT SWITCHES IS DUE TO SEVERE
	VIBRATION OF THE SWITCH MOUNTING BRACKETS AND OPERATING DISCS INDUCED BY
12	VIBRATION OF THE COMBINED STOP AND CONTROL VALVES. THE MOUNTING BRACKETS
1 3	AND THE OPERATING DISCS HAVE BEEN REDESIGNED AND THE MODIFICATION IS PRESENTLY
1 4	BEING INSTALLED. DESIGN CHANGE NO. RFC-DC-02-2248.
1 5	FACILITY SPOWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32 B 31 OPERATOR OBSERVATION .
	OCTIVITY CONTENT ELEASED OF RELEASE AMOUNT OF ACTIVITY (35) NA PERSONNEL ASSOCIATED AND NA PERSONNEL ASSOCIATED AND NA PERSONNEL ASSOCIATED AND NA A44 45 46 NA NA NA NA NA NA NA NA NA N
17/	NUMBER OF TYPE DESCRIPTION (39) NA NA
18	PERSONNEL INJURIES 13 NA NA NA NA
7 8	11 12 10 10 10 10 10 10 10 10 10 10 10 10 10
,[.1 ⁷] 9 7 ,8	NA NBC USE ONLY
20	SSUED DESCRIPTION (45) NA OBSERVATION OF THE PROPERTY OF THE
7	NAME OF A PARE Jack Rischling PHONE: 616-465-5901

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(3,77)	LICENSEE EVENT REPORT
	CCNTROL BLOCK:
0 1 8	M I D C C 2 2 0 0 0 0 0 0 0 0 0 0 3 4 1 1 1 1 1 4 5 5 CAT 58
CON'T 0 1 7 8	REPORT L 6 0 5 0 0 0 3 1 6 7 0 7 0 6 7 8 8 0 18 0 2 7 8 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT, DATE 74 75 REPORT DATE 80
0 2	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) AFTER REACTOR TRIPS ON MARCH 25, JUNE 14,19, JULY 6, 14, 1978, ROD POSITION
0 3	INDICATION FOR CONTROL ROD H-8 DID NOT INDICATE THAT H-8 WAS FULLY INSERTED. THE
0 4	ACTION REQUIREMENT OF T.S.3.1.1.1 WAS FULFILLED BY EMERGENCY BORATING FOR
0 5	5 MINUTES. THE MARCH AND JUNE EVENTS WERE DISCOVERED WHILE GATHERING OPERATING
0 6	INFORMATION, CONCERNING OTHER EVENTS. OCCURRENCES OF A SIMILAR NATURE HAVE BEEN
0 7	IDENTIFIED ON RO-050-0316/78-40.
08	9 80
0 9	SYSTEM CAUSE CODE SUBCODE COMPONENT CODE SUBCODE SUBCO
	LER/RO EVENT YEAR REPORT NO. O 4 8 0 3 L
	ACTION FUTURE EFFECT SHUTDOWN HOURS (22) ATTACHMENT NPRD-4 PRIME COMP. COMPONENT TAKEN ACTION ON PLANT METHOD HOURS (22) SUBMITTED FORM SUB. SUPPLIER MANUFACTURER X 139 X
	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10	THE ROOT CAUSE OF THESE EVENTS IS UNKNOWN AT THIS TIME. AT THE TIME OF THE
11	REACTOR TRIPS, ROD POSITION INDICATION (RPI) INDICATES THAT ROD H-8 IS STUCK
1 2	APPROXIMATELY 25-30 STEPS OUT, RPI SECONDARY COIL VOLTAGE MEASUREMENTS HAVE
1 3	ALSO INDICATED THAT H-8 IS NOT FULLY INSERTED. HOWEVER, H-8 DOES DRIFT IN
1 4 7 8	SLOWLY AFTER 10-15 MINUTES. SEE ATTACHED SUPPLEMENT.
1578	TACILITY SPOWER OTHER STATUS OF DISCOVERY DESCRIPTION (32) D (28) 0 0 0 (29) NA
	CTIVITY CONTENT ELEASED OF RELEASE AMOUNT OF ACTIVITY 35 NA 44 PERSONNEL EXPOSURES AMOUNT OF ACTIVITY 35 NA 45 NA 4
17	NUMBER 0 0 0 0 37 Z 38 NA
1 8	PERSONNEL INJURIES NUMBER DESCRIPTION 41 NA 9 11 12 2
1 9	LOSS OF OR DAMAGE TO FACILITY 43 TYPE DESCRIPTION NA NA NA
7 8	9 10 80 PUBLICITY SSUED DESCRIPTION 45 NRC USE ONLY N 44
7 • 8	SSUED DESCRIPTION NA

Contract of the Se Carried Se ATTACHMENT TO LER # 78-048/03L-0

SUPPLEMENT TO CAUSE DESCRIPTION

THE PROBLEM APPEARS TO BE PERIODIC IN NATURE AND DOES NOT OCCUR AFTER EVERY REACTOR TRIP, BUT GENERALLY ONLY AFTER REACTOR TRIPS FROM HIGH POWER LEVELS.

INVESTIGATIVE TESTING INCLUDES ROD DROP TESTS AND A CONTROLLED REACTOR TRIP FROM 8% POWER. REACTOR TRIP TEST DATA INCLUDED TAKING VISICORDER TRACES OF H-8 RPI PRIMARY COIL VOLTAGE WITH BRUSH RECORDERS MONITORING THE SIGNAL CONDITIONING MODULE FOR H-8 RPI AND ANOTHER ROD USED AS A COMPARATOR. TEST DATA OBTAINED FROM THE REACTOR TRIP WAS INCONCLUSIVE SINCE CONTROL ROD H-8 DID NOT HANG-UP. THIS IS AN INTERIM REPORT AND A REVISION WILL BE SUPPLIED LATER WHEN A ROOT CAUSE HAS BEEN DETERMINED.