



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION II
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ATLANTA, GEORGIA 30303-1257

June 20, 2018

Mr. Mano Nazar
President and Chief Nuclear Officer
Nuclear Division
Florida Power & Light Co.
Mail Stop: EX/JB
700 Universe Blvd.
Juno Beach, FL 33408

**SUBJECT: ST. LUCIE PLANT – NUCLEAR REGULATORY COMMISSION PROBLEM
IDENTIFICATION AND RESOLUTION INSPECTION REPORT
05000335/2018010 AND 05000389/2018010**

Dear Mr. Nazar:

On May 25, 2018, the U.S. Nuclear Regulatory Commission (NRC) completed a problem identification and resolution inspection at your St. Lucie Plant, Units 1 and 2. On that date, the NRC inspectors discussed the results of this inspection with Mr. Robert Coffey, Southern Regional Vice President, and other members of your staff. The results of this inspection are documented in the enclosed inspection report.

The NRC inspection team reviewed the station's corrective action program and the station's implementation of the program to evaluate its effectiveness in identifying, prioritizing, evaluating, and correcting problems, and to confirm that the station was complying with NRC regulations and licensee standards for corrective action programs. Based on the samples reviewed, the team determined that your staff's performance in each of these areas adequately supported nuclear safety.

The team also evaluated the station's processes for use of industry and NRC operating experience information and the effectiveness of the station's audits and self-assessments. Based on the samples reviewed, the team determined that your staff's performance in each of these areas adequately supported nuclear safety.

Finally, the team reviewed the station's programs to establish and maintain a safety-conscious work environment, and interviewed station personnel to evaluate the effectiveness of these programs. Based on the team's observations and the results of these interviews, the team found no evidence of challenges to your organization's safety-conscious work environment. Your employees appeared willing to raise nuclear safety concerns through at least one of the several means available.

The NRC inspectors did not identify any finding or violation of more than minor significance.

M. Nazar

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This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with 10 CFR 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Randall A. Musser, Chief
Reactor Projects Branch 3
Division of Reactor Projects

Docket Nos.: 50-335, 50-389
License Nos.: DPR-67, NPF-16

Enclosure:
IR 05000335/2018010 and 05000389/2018010

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IDENTIFICATION AND RESOLUTION INSPECTION REPORT
05000335/2018010 AND 05000389/2018010 June 20, 2018

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U.S. NUCLEAR REGULATORY COMMISSION

REGION II

Docket Nos.: 50-335, 50-389

License Nos.: DRP-67, NPF-16

Report Nos.: 05000335/2018010, 05000389/2018010

Enterprise Identifier: I-2018-010-0045

Licensee: Florida Power & Light Company (FPL)

Facility: St. Lucie Plant, Units 1 and 2

Location: 6501 South Ocean Drive
Jensen Beach, FL 34957

Dates: May 7 through May 25, 2018

Inspectors: A. Wilson, Project Engineer, Team Leader
L. Pressley, Senior Project Engineer
S. Roberts, Resident Inspector
J. Rotton, Senior Resident Inspector

Approved by: Randall A. Musser, Chief
Reactor Projects Branch 3
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a Problem Identification and Resolution Inspection at St. Lucie Plant, Units 1 and 2 in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings were identified.

INSPECTION SCOPE

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met, consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71152B – Problem Identification and Resolution (1 Sample)

The inspectors performed a biennial assessment of the licensee's corrective action program, use of operating experience, self-assessments and audits, and safety-conscious work environment. The assessment is documented below.

- (1) Corrective Action Program Effectiveness: Problem Identification, Problem Prioritization and Evaluation, and Corrective Actions – The inspection team reviewed the station's corrective action program and the station's implementation of the program to evaluate its effectiveness in identifying, prioritizing, evaluating, and correcting problems, and to confirm that the station was complying with NRC regulations and licensee standards for corrective action programs.
- (2) Operating Experience and Self-Assessments and Audits – The team evaluated the station's processes for use of industry and NRC operating experience information and the effectiveness of the station's audits and self-assessments.
- (3) Safety Conscious Work Environment - The team reviewed the station's programs to establish and maintain a safety-conscious work environment, and interviewed station personnel to evaluate the effectiveness of these programs.

INSPECTION RESULTS

Corrective Action Program Effectiveness Observations	71152—Problem Identification and Resolution
<p>Based on the samples reviewed, the team determined that the licensee's corrective action program (CAP) complied with regulatory requirements and self-imposed standards. The licensee's implementation of the CAP adequately supported nuclear safety.</p> <p>Effectiveness of Problem Identification: The inspectors determined that the licensee was effective in identifying problems and entering them into the CAP and there was a low threshold for entering issues into the CAP. This conclusion was based on a review of the requirements for initiating Condition Reports (CRs) as described in licensee procedure PI-AA-104-1000, "Condition Reporting," and management's expectation that employees were</p>	

encouraged to initiate CRs for any reason. Additionally, site management was actively involved in the CAP and focused appropriate attention on significant plant issues.

Based on reviews and walkdowns of accessible portions of selected systems, the inspectors determined that deficiencies were being identified and placed in the CAP.

Effectiveness of Prioritization and Evaluation of Issues: Based on the review of CRs sampled by the inspection team during the onsite period, the inspectors concluded that problems were generally prioritized and evaluated in accordance with the CR significance determination guidance in procedure PI-AA-104-1000. The inspectors determined that in general, adequate consideration was given to system or component operability and associated plant risk.

The inspectors determined that plant personnel had conducted root cause and apparent cause analyses in compliance with the licensee’s CAP procedures and cause determinations were appropriate, and considered the significance of the issues being evaluated. A variety of formal causal-analysis techniques were used to evaluate CRs depending on the type and complexity of the issue consistent with the applicable cause evaluation procedures.

The inspection team identified one issue of concern regarding the failure to properly consider mission time of Emergency Diesel Generators (EDGs) in operability evaluations in accordance with Procedure EN-AA-203-1001, “Operability Determinations / Functionality Assessments,” Revision 29. This issue was entered into the licensee’s CAP as CR 2264338 and is documented as a minor performance deficiency in the section below.

Effectiveness of Corrective Actions: Based on a review of corrective action documents, interviews with licensee staff, and verification of completed corrective actions, the inspectors determined that overall, corrective actions were timely, commensurate with the safety significance of the issues, and effective, in that conditions adverse to quality were corrected. For significant conditions adverse to quality, the corrective actions directly addressed the cause and effectively prevented recurrence. The team reviewed performance indicators, CRs, and effectiveness reviews, as applicable, to verify that the significant conditions adverse to quality had not recurred. Effectiveness reviews for corrective actions to prevent recurrence (CAPRs) were sufficient to ensure corrective actions were properly implemented and were effective.

Minor Performance Deficiency	71152—Problem Identification and Resolution
<p>Minor Performance Deficiency: The station failed to properly consider mission time of EDGs in operability evaluations in accordance with Procedure EN-AA-203-1001. Specifically, as documented in several CRs, the conclusions provided within the operability analysis sections could not be supported without evaluating the conditions against the system mission time.</p> <p>The inspectors reviewed CRs 2132695, 2132704, and 2132705, which were all initiated to document oil leaks on the EDGs. Each CR contained a statement that, “there is no impact on the ability of the EDG to meet its mission time,” under the operability notes.</p> <p>The inspectors reviewed the EDG Design Basis Documents (DBD-EDG-1, -2) for the defined mission time and determined that an official mission time had not been established for the</p>	

EDGs. The inspectors interviewed a sample of senior reactor operators and determined that the mission time used for operability determinations had been a loosely based time derived from EDG placards and day tanks and diesel oil storage tank capacities.

Procedure EN-AA-203-1001 governed operability determinations at St. Lucie Plant. Step 4.2.2.2 directed that the screener of a condition report shall perform the Operability/Functionality Review screening using guidance in Attachment 1. Step 6, Table C of Attachment 1 required the screener to evaluate that the condition did not impair completion of design mission time.

For issues that required immediate operability determinations, Step 4.3.6 directed the Shift Manager to prepare the Immediate Operability Determination using the guidance of Attachment 2, Immediate Operability Determination Guidelines. Attachment 2, Step 2 stated: "In order to be considered operable, a SSC must be capable of performing the specified safety functions of its design, within the required range of physical conditions, initiation times, and mission times assumed in the CLB. For operability determination purposes, the mission time was the duration of SSC operation that was credited in the design basis for the SSC to perform its specified safety function."

In addition, Attachment 2, Step 8, stated: "If oil leakage is detected from either a pump bearing housing, a motor bearing housing, or a governor, then leakage rate and mission time must be addressed."

Contrary to the above, because a mission time had not been established in the EDG Design Basis Documents (DBD-EDG-1, -2) for the EDGs, the ability of the system to remain operable for the duration of its mission time had not been appropriately evaluated for several conditions associated with EDGs.

CR 2264338 was originated to address the issue. The licensee had initiated actions to establish an official EDG mission time within the Design Basis Documents and perform an extent of condition to determine if a mission time needed to be established for other plant systems.

Screening: The inspectors screened the issue in accordance with Inspection Manual Chapter 0612, Appendix B, "Issue Screening," and determined that the performance deficiency did not meet the criteria in Block 3 to be a More-than-Minor issue. The inspectors determined that the CRs reviewed for this instance were minor in nature and therefore would unlikely change the outcome of the operability determinations of the examples reviewed. The inspectors did not identify a violation of regulatory requirements associated with this minor performance deficiency.

Operating Experience and Self-Assessments and Audits Observations	71152—Problem Identification and Resolution
<p>Based on the samples reviewed, the team determined that station’s processes for the use of industry and NRC operating experience information and for the performance of audits and self-assessments were effective and complied with all regulatory requirements and licensee standards. The implementation of these programs adequately supported nuclear safety. Overall, the team concluded that operating experience was adequately evaluated for applicability and that appropriate actions were implemented to address lessons learned as needed. In general, the inspectors determined that the licensee was effective at performing self-assessments and audits to identify issues at a low level, properly evaluated those issues, and resolved them commensurate with their safety significance.</p>	

Safety Conscious Work Environment Observations	71152—Problem Identification and Resolution
<p>Based on interviews with plant staff and reviews of the latest safety culture survey results to assess the safety-conscious work environment on site, the team found no evidence of challenges to safety-conscious work environment. Employees appeared willing to raise nuclear safety concerns through at least one of the several means available.</p>	

EXIT MEETINGS AND DEBRIEFS

The inspectors confirmed that proprietary information was controlled to protect from public disclosure. No proprietary information was documented in this report.

- On May 25, 2018, the inspectors presented the problem identification and resolution inspection results to Mr. Robert Coffey, Southern Regional Vice-President, and other members of the licensee staff.

DOCUMENTS REVIEWED

Condition Reports

- 1893969; Replace All 9 Bus Vent Covers on 1A UAT; 8/2/2013
- 1920696; Unit 2 Manual Reactor Trip; 11/14/2013
- 1920742; Found Degraded Relays During Extent of Condition; 11/15/2013
- 1937045; HCV-14-11A1 Seal Cooler Valve Failed Closed; 1/29/2014
- 1948469; V381 Over Thrust During Disk Pull Out; 3/15/2014
- 1951584; Degraded Contacts Prevented 2B EDG Breaker From Closing; 3/25/2014
- 1973521; LPM Channel 1 Below Minimum Threshold; 6/20/2014
- 2054489 'B' RPS CNTMT Press Bistable Trip & Pre-Trip Lights Inop; 06/16/2015
- 2077661; 2C ICW Pp Motor Tripped On Overcurrent; 9/30/2015
- 2083954; Indicator Stuck At The Top Of The Gauge; 10/23/2015
- 2084836; 2C AFW Pump Governor Oil Leak; 10/23/2015
- 2090225; Unauthorized entry into a High Radiation area; 5/3/2016
- 2095560; U1 Circ Water DFS Filter DF-21-1B1 High DP Condition; 12/5/2015
- 2108156; EDG 1A One Engine Fail To Start Alarm Locks In During Surv; 02/06/2016
- 2108868; 2016 CDBI Functionality of EER Sup Fan Discharge Dampers, GD-1/2; 2/9/2016
- 2109116; 2016 CDBI HVAC Action in 1-EOP-09; 2/10/2016
- 2109310; Hydrogen Purity Local Meter Not Indicating Correct Purity; 2/11/2016
- 2111358; Failure to verify adequacy of Unit 1 Electrical Equipment room ventilation system; 04/11/2016
- 2112128; Reverse Flow & Backward Rotation Observed on HVS-5A/B
- 2114645; 10 CFR Part 21 issue for Masterpack NT & NW breakers; 1/18/2017
- 2117193; Failure to verify adequacy of Unit 1 Electrical Equipment room ventilation system; 04/11/2016
- 2125717; ACE to Address Green NCV on CAQ for EER Supply Fans; 7/14/2016
- 2128716; Loose Parts Monitoring Alarm For Channel 4; 4/29/2016
- 2129351; Control Room Shift Communicator failure on simulator; 5/2/2016
- 2131209; "A" EDG Conduit Covers Loose; 5/11/2016
- 2131211; "A" EDG Conduit Termination Pulled Loose; 5/11/2016
- 2131215; 1A EDG Set Instrument Tubing In Contact With Each Other; 5/11/2016
- 2131219; 1A1 EDG Machine Screw Found Out Of Place; 5/11/2016
- 2132695; Valve Gallery Cover On 1A2 EDG Leaking Oil; 5/18/2016
- 2132704; 1A1 EDG Turbocharger Flange Oil Leak; 5/18/2016
- 2132705; 1A1 EDG Camshaft Covers Leaking Oil; 5/18/2016
- 2133018; UNIT 2 2A2 EDG Fuel Pressure Low; 5/19/2016
- 2133422; 2B MSIV accumulator corrosion; 5/22/2016
- 2133504; 2D Instrument air compressor not loading; 5/23/2016
- 2133774; 2D Instrument air compressor trip on high cylinder temperature; 5/24/2016
- 2134435; NOS Finding – Radioactive material stored outdoors; 5/26/2016
- 2135100; 1A EDG MVAR Readings Off Normal; 5/31/2016
- 2135780; NRC EQ Inspection – Containment High Range Radiation Monitors; 6/2/2016
- 2136211; TR-1112 Failed Low Affecting DCS and 'A' QSPDS; 6/6/2016
- 2136427; ROTORK Controls, Inc. and Electroswitch Part 21 issues; 6/7/2016
- 2137373; DCS Point 2CSFM 10:P8023a A.PNT; 6/10/2016
- 2140294; 2D Instrument air compressor trip during performance run; 6/24/2016
- 2140595; 1A EDG MVAR Swings; 6/27/2016
- 2143046; U2 RCS Zinc concentration dropped below threshold limit; 7/12/2016
- 2143242; Evaluation of U2 ACC-3C refrigeration migration issue; 7/12/2016

- 2144201; Shortfall in observations of Radiological work practices; 7/17/2016
- 2144412; 1A Fire pump breaker, 1-40209, 10 CFR Part issue; 7/18/2016
- 2144416; 2AB to 2B2 bus tie breaker, 2-40706, 10 CFR Part 21 issue; 7/18/2016
- 2145831; Condition of RABS unsat; 7/25/2016
- 2146175; U1 RCS Zinc concentration less than admin limit; 7/27/2016
- 2148252; V3217 Backleakage During Shutdown Cooling Flush; 8/6/16
- 2149500; Mv-09-11 Only Strokes Closed Intermittently; 8/12/2016
- 2150868; Work hours control violation; 8/18/2016
- 2151322; Inadequate Modification Of The Unit 2 DOST, EC274456; 8/22/2016
- 2151502; High Radiation Area and contamination area created due to RES; 8/23/2016
- 2152647; Instrument air system pressure degrading trend; 8/28/2016
- 2154938; All NRC ENS phone lines OOS; 9/9/2016
- 2155627; Untimely classification resulting in missed DEP-PI; 9/13/2016
- 2156690; Recovery plan for exercise and drill performance; 9/19/2016
- 2156817; DEP miss due to inaccurate state notification during SIMEVAL; 9/20/2016
- 2157214; 1B DC Bus Ground Locked In; 9/21/2016
- 2157980; Core Exit Thermocouple 16 Q-3, Continues To Drop Out Low; 09/25/2016
- 2158314; U1 CCW Tritium greater than MDA; 9/27/2016
- 2160390; 1B EDG Room Floor Drain Backing Up; 10/5/2016
- 2165042; Foamy oil in U1 ACC-3C, control room air conditioner; 10/25/2016
- 2165690; 1A EDG MVAR Swings During Adjustments; 10/27/2016
- 2166772; Industry OE – Harris NOUE (LOOP); 11/1/2016
- 2166950; EDG LOOP Response Time Safeguards Acceptance Criteria; 11/1/2016
- 2167267; Industry OE – Farley Alert due to toxic gas; 11/3/2016
- 2168328; Leakage Past V09248; 11/9/2016
- 2168775; Industry OE – Dresden U3 Alert- ICES Report# 323383; 11/11/2016
- 2168864; WO: 40499100-01 Inadvertently Closed In EWP; 11/11/2016
- 2174203; Loose Parts Monitor Alarming; 12/12/2016
- 2175061; EDG Starting Air Leakage Test Margin Recovery; 12/15/2016
- 2175530; During SI-1-27, V09280 Was Worked And PMT Was Not Completed; 12/15/2016
- 2175641; L-31, Boron Concentration Low Channel 1 Alarming; 12/17/2016
- 2176118; Inappropriate long term storage of radioactive material; 12/21/2016
- 2179806; 2D Instrument air compressor excessive cycling; 1/14/2017
- 2181204; Declining RCA Radiation Worker observation trend; 1/23/2017
- 2183774; ALARA near miss due to inadequate scaffold; 2/4/2017
- 2184494; PSL Siren S-70 test failure; 2/8/2017
- 2185542; Waste processing challenges due to U1 AWST Activity; 2/14/2017
- 2186438; HVA/ACC-3B Freon leak; 2/17/2017
- 2187133; 2B EDG Jacket Water Temp Low During Safeguards; 2/21/17
- 2187169; OE regarding a loss of instrument air (NRC IN 2016-05); 2/22/2017
- 2187670; AFW PP 1C-PP OTBD Brg Wear-Increased Particle Concentration; 2/23/2017
- 2188004; Personnel contamination event / PCE #17-013; 2/25/2017
- 2188010; The Wrong Pc Level Oil Was Used In The 1c Auxiliary FW Pump; 2/25/2017
- 2188020; SE-07-5F Solenoid Valve PEN P-51, Field Wires Damaged; 3/1/2017
- 2188171; Personnel contamination event / PCE #17-015; 2/26/2017
- 2188184; EDG Fuel Oil Cross-Connect Surveillance Conundrum; 2/27/17
- 2188751; Engineered Safeguard Logic Cabinet Sa Volt Meter Failed; 03/01/2017
- 2189467; 2A Instrument air dryer blowing down continuously; 3/4/2017
- 2189538; 1C Instrument air compressor low oil level; 3/5/2017
- 2190852; 1C Instrument air compressor excessive oil consumption; 3/11/2017

- 2190895; 20 Gallons Of Oil Added To 2A1 Engine On 2A EDG; 3/11/2017
- 2190896; 12 Gallons Of Oil Added To 2A2 Engine On 2A EDG; 3/11/2017
- 2192588; "A" Feedwater Header Mfiv,HCV-09-1,Hydraulic Actuator Issue; 3/21/2017
- 2193112; 2B2 RCP Lower Guide BRG Temp Inconsistent Output; 3/22/2017
- 2193490; 1C AFW Pp OTBD Oil Reservoir Drain/Flush Before Next Run; 3/24/2017
- 2193884; 1C Instrument air compressor oil leak; 3/25/2017
- 2194614; 1D Instrument air compressor low oil level; 3/29/2017
- 2202422; HVA/ACC-3A Low Freon level while running; 4/30/2017
- 2203125; Ladder laying across contaminated area boundary; 5/3/2017
- 2203233; Request Conditional Release Of R&O 150 Oil For 2B EDG; 5/4/2017
- 2204967; Industry OE – LaSalle Anchor Darling Part 21; 5/15/2017
- 2205200; Unexpected Loss of the 2A3 4160V Safety Bus; 5/16/2017
- 2205687; U1 CCW shows signs of Tritium activity; 5/17/2017
- 2205761; 2A2 EDG Tubing Found Degraded; 5/18/2017
- 2206260; HCV-09-2B Oil Leak; 5/21/2017
- 2206286; 2A1 EDG Water Immersion Heater Tripping; 5/21/2017
- 2207340; Level 1 Assessment – Trending Program at PSL; 5/26/2017
- 2208096; Unit 1 TSC plant computer room air conditioning not cooling; 5/31/2017
- 2223692; Incomplete actions from Hurricane Matthew assessment; 9/6/2017
- 2210364; HVA 3A trip on motor temperature overload; 6/14/2017
- 2212311; MCC AC-DC starter mechanical interlock reliability review; 6/26/2017
- 2217392; Engineering to perform OE review Robinson Plane OE 241354; 7/27/2017
- 2217631; U1 Hot Leg Injection Procedures; 7/29/2017
- 2219154; HCV-09-2B Reservoir Oil Level Low In Spec Not An Active Leak; 8/8/2017
- 2220259; 2A1 EDG Fuel Pressure Reads Below Min Value; 8/14/2017
- 2221186; HVA/ACC-3B failure due to ground on condensing unit; 8/20/2017
- 2222623; 1B2 EDG Needs Oil Added; 8/29/2017
- 2222743; U1 ACC-3 thermostat not responding; 8/29/2017
- 2224199; MFIV Oil Reservoir At The Low Level Mark; 9/10/2017
- 2224325; Unacceptable performance for post-hurricane accountability; 9/11/2017
- 2226606; 2B EDG Load Transient During Performance Of 2-OSP-59.01B; 9/25/17
- 2227429; Fire Protection Program Implementation Cognitive Trend; 9/28/2017
- 2229920; 2B EDG Fuel supply; 10/12/2017
- 2230410; Elevated contamination in room; 10/14/2017
- 2230610; Level 1 Assessment – Fire Protection Program Implementation; 10/16/2017
- 2230758; OE- Industry White finding for EDG Diode Failure Catawba; 10/17/2017
- 2232779; U2 C AFW Flow Transmitters Isolated; 10/25/2017
- 2232782 1A MSL Rad Monitor Frequently Spiking Into Alarm. Inop. 10/25/17
- 2233080; Multiple CBCS Deficiencies Create Operator Burdens; 10/27/2017
- 2233516; Work Request Cancelled w/o Any Work Performed on Active Leak; 10/29/2017
- 2233669; Recommended Actions From Previous AR 02110464 Not Addressed; 10/30/2017
- 2234317; 2-FSG-10, Safety Injection Tank Isolation – Rev 2; 11/2/2017
- 2235711; U1 ACC-3B low oil level; 11/10/2017
- 2236131; Tritium detected in catch basins; 11/14/2017
- 2237096; U1 HVA-3C compressor going into pump down; 11/20/2017
- 2238413; Part 21 Report, Fisher Information Notice 17-05; 11/30/2017
- 2238617; 1b AFW Pp THRST BRN Preload/Early Stage Wear – Contingency; 12/1/2017
- 2242181; Increased Flow Coming From 2B EDG OIL LEAK; 12/27/2017
- 2243677; Nuc. Assurance Escalation. L1: Inadequate use of GFCI protection; 1/9/2018

- 2243798; Water Leaking from Cell Jar Lids; 1/9/2018
- 2245101; 2D Instrument air compressor motor overload; 1/18/2018
- 2245821; Low Level In Reservoir; 1/23/2018
- 2246135; Inappropriate Use of MA Assignments for CAQ CRs; 1/24/2018
- 2249170; One-Hour Notification; 2/13/2018
- 2249174; Contamination in a clean area; 2/12/2018
- 2249292; 1b AFW Pp Outboard Cover Guard Corroded; 02/13/2018
- 2249873; NRC evaluated drill 2018SAE not performed IAW drill scenario; 2/15/2018
- 2249999; Nuc. Assurance Escalation. L1: Metal shavings on plant equipment; 2/16/2018
- 2250930; PI&R Self Assessment; 2/22/2018
- 2250938; PI&R Self Assessment; 2/22/2018
- 2253212; Siren #1 failure; 3/8/2018
- 2253859; 1A EDG Failed Response Time Testing; 3/13/18
- 2254424; Alternate Source Supplying Load Light Will Not Illuminate; 3/15/2018
- 2254769; Higher than expected dose rates on refuel bridge; 3/19/2018
- 2255082; Inadequate commercial grade dedication resulted in Part 21; 3/20/2018
- 2256602; NRC Ident: Metal Shavings, Unit 1 Vital AC SUPS and 1C HPSI; 3/28/2018
- 2257037; Ris-26-13-1 Reading High Alarm; 03/30/2018
- 2257077; 2B2 EDG Engine Needs Oil Added; Unsafe To Do So Manually; 3/30/2018
- 2257159; Through wall leak on SW line reducer weld; 3/30/2018
- 2257515; Personnel contamination event; 4/2/2018
- 2257712; RM-26-12 failure causing 2-AOP- 26.01 entry; 4/3/2018
- 2258467; MV-09-9 'A' AFW To 'A' S/G Failed To Control; 4/8/2018
- 2258467; Mv-09-9 'A' AFW To 'A' S/G Failed To Control; 4/8/2018
- 2258467; MV-09-9 'A' AFW To 'A' S/G Failed To Control; 4/8/2018
- 2259242; Level 1 Assessment of Part 21 Process; 4/12/2018
- 2259520; Metallic Debris on floor of U2 switchgear rooms; 4/18/2018
- 2263320; PI Processbook Unable to Calculate RCS Leak Rate; 5/9/2018
- 2264338; 2018 NRC PI&R Inspection EDG Mission Time; 5/15/2018
- 32 Security-related CRs were reviewed during the inspection

Procedures

- 1-GMM-80.41; Atwood & Morrill 12 Inch Swing Check Valve Maintenance; Revision 7
- 1-OSP-63.13A; ESF - Staggered 36 Month Surveillance For SIAS/CIS/CSAS - Train A; Revisions 32 & 33
- 2-FSG-10; Safety Injection Tank Isolation; Revision 2
- 2-GMM-03.02; Westinghouse 12 Inch Swing Check Valve Maintenance; Revision 2
- 2-GMM-03.04; Westinghouse 3 Inch Swing Check Valve Maintenance; Revision 4
- ADM-17.32; Structures Monitoring Program; Revision 6
- EN-AA-203-1001; Operability Determinations / Functionality Assessments; Revision 29
- ER-AA-100-2002; Maintenance Rule Program Administration; Revision 6
- ER-AA-124; Check Valve Program, Revision 1
- NA-AA-200-1000; Employee Concerns Program; Revision 1
- PI-AA-100; Condition Assessment and Response; Revision 10
- PI-AA-100-1005; Root Cause Analysis; Revision 17
- PI-AA-100-1006; Common Cause Evaluation; Revision 13
- PI-AA-100-1007; Issue Investigation; Revision 20
- PI-AA-101; Assessment and Improvement Programs; Revision 24
- PI-AA-102-1002; Internal Operating Experience; Revision 8
- PI-AA-102-1003; Sharing Operating Experience With Nuclear Industry; Revision 12

- PI-AA-104-1000; Condition Reporting; Revision 17
- PI-AA-207; Trend Coding and Analysis; Revision 13
- PI-AA-207-1000; Station Self-Evaluation and Trending Analysis; Revision 8
- SY-AA-102-1000; Safeguards Information Program; Revision 15

Self-Assessments, Audits, and Trend Reports

- 4Q17 PSL Self-Evaluation and Trending Analysis Report
- 4Q17 PSL/Engineering Self-Evaluation and Trending Analysis Report
- 4Q17 PSL/Maintenance Self-Evaluation and Trending Analysis Report
- 4Q17 PSL/Operations Self-Evaluation and Trending Analysis Report
- 4Q17 PSL/Radiation Protection Self-Evaluation and Trending Analysis Report
- 4Q17 PSL/Security Self-Evaluation and Trending Analysis Report
- Pre-NRC Problem Identification & Resolution (PI&R) Inspection Assessment
- PSL 16-002 Radiation Protection and Radwaste Audit Report; 5/26/2016
- PSL 16-005 EP Audit Report; 8/10/2016
- PSL 16-006; Operations; 9/7/2016
- PSL 16-009 Chemistry Audit Report; 1/04/2017
- PSL 17-003 QA Programs Audit Report; /18/2017
- PSL 17-005; Engineering Audit; 6/30/2017
- PSL 17-007 Emergency Preparedness Audit Report; 8/10/2017
- PSL 17-010; Performance Improvement; 1/25/2018
- PSL 17-012; Time Sensitive Action Review; 12/21/2017

Work Orders

- 40276910-01
- 40276911-01
- 40284284
- 40374092
- 40374094
- 40398730
- 40410089
- 40411087
- 40469413
- 40471113
- 40472171
- 40477208
- 40477338
- 40490422
- 40521769
- 40538193
- 40591898
- 94141676
- 94143832
- 94156308
- 94172111

Other

- Alternative Analysis Unit 2 DOST NFPA 30 Section 2348 Compliance AR 2151322-10; 9/25/16
- Apparent Cause Evaluation; EDG Speed Switch Failure, CR 02069073; 1/28/2016

- AR 2035539; Maintenance Rule (a)(1) Action Plan; V18279 Check Valve for the 1A Maintenance Hatch Door Seal Accumulator; 6/19/2015
- AR 2074774; Maintenance Rule (a)(1) Action Plan; 4kV and 6.9kV, System 53; 6/21/2017
- AR 2110122; Maintenance Rule (a)(1) Action Plan; Low Accumulator Pressure; Revision 1; 10/12/2017
- AR 2148252; Maintenance Rule (a)(1) Action Plan; V3217, SI Loop 1A2 Check Valve; 10/17/2016
- AR 2151217; Maintenance Rule (a)(1) Action Plan; Main Generator System 53a; 12/2/2016
- AR 2213017; Maintenance Rule (a)(1) Action Plan; 2B Component Cooling Water; Revision 1; 11/3/2017
- AR 2218382; Maintenance Rule (a)(1) Action Plan; control Room Ventilation System; Revision 1
- EDG System; Design Basis Document DBD-EDG-1; Revision 10
- EDG System; Design Basis Document DBD-EDG-2; Revision 9
- Equipment Apparent Cause Evaluation; 1A EDG Abnormal Sounds and Indications After Start; AR 02053060; 1/21/2016
- NEESLX138-PR-01; NFPA 30 Code Compliance Independent Review; Revision 1
- Nuclear Safety Culture Monitoring Panel; 4Q17 Data; 1/11/2018
- Nuclear Safety Culture Sub-Committee; 3Q17 Data
- Nuclear Safety Culture Sub-Committee; 2Q17 Data
- PSL-ENG-SEMS-17-007; St. Lucie Unit 1 Hot Leg Injection Failure Modes and Effects Analysis; Rev. 0; 9/25/2017
- Quality Assurance Topical Report FPL-1; Revision 21
- System Health Report, Unit 1 Auxiliary Feedwater System, Q2-2018
- System Health Report, Unit 2 Auxiliary Feedwater System, Q2-2018
- System Health Report; Unit 1 EDG; Q2-2018
- System Health Report; Unit 1 EDG; Q2-2018
- Unit 1 System 25b (Heating, Ventilation & Air Conditioning – Non-Risk significant) System Health Report; 4/18/2018
- Unit 2 System 25b (Heating, Ventilation & Air Conditioning – Non-Risk significant) System Health Report; 4/18/2018
- Unit 1 System 18 (Instrument & Service Air) System Health Report; 4/18/2018
- Unit 2 System 18 (Instrument & Service Air) System Health Report; 4/18/2018