

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

July 12, 1993

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D. C. 20555

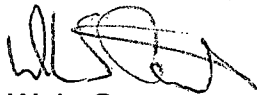
Serial No. 93-421  
NO/PC:vlh  
Docket Nos. 50-280  
50-281  
License Nos. DPR-32  
DPR-37

Gentlemen:

**VIRGINIA ELECTRIC AND POWER COMPANY**  
**SURRY POWER STATION UNITS 1 AND 2**  
**MONTHLY OPERATING REPORT**

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of June 1993.

Very truly yours,



W. L. Stewart  
Senior Vice President - Nuclear

Enclosure

cc: U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, N. W.  
Suite 2900  
Atlanta, Georgia 30323

Mr. M. W. Branch  
NRC Senior Resident Inspector  
Surry Power Station

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**OPERATING DATA REPORT**

Docket No.: 50-280  
 Date: 07-07-93  
 Completed By: D. Mason  
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 1
- 2. Reporting Period: ..... June, 1993
- 3. Licensed Thermal Power (MWt): ..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):.... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

\_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_

10. Reasons For Restrictions, If Any: \_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

	This Month	YTD	Cumulative
11. Hours In Reporting Period .....	720.0	4343.0	179903.0
12. Number of Hours Reactor Was Critical .....	720.0	4236.4	119611.4
13. Reactor Reserve Shutdown Hours .....	0.0	0.0	3774.5
14. Hours Generator On-Line.....	720.0	4218.0	117493.4
15. Unit Reserve Shutdown Hours.....	0.0	0.0	3736.2
16. Gross Thermal Energy Generated (MWH).....	1751251.0	10046983.5	273666262.6
17. Gross Electrical Energy Generated (MWH)....	580910.0	3378920.0	89397173.0
18. Net Electrical Energy Generated (MWH).....	556470.0	3222918.0	84820778.0
19. Unit Service Factor .....	100.0%	97.1%	65.3%
20. Unit Availability Factor.....	100.0%	97.1%	67.4%
21. Unit Capacity Factor (Using MDC Net).....	99.0%	95.0%	60.8%
22. Unit Capacity Factor (Using DER Net).....	98.1%	94.2%	59.8%
23. Unit Forced Outage Rate.....	0.0%	2.9%	18.0%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Scheduled Outage - July 16, 1993, 11 days

\_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

25. If Shut Down at End of Report Period, Estimated Date of Start-up: \_\_\_\_\_

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

**OPERATING DATA REPORT**

Docket No.: 50-281  
 Date: 07-07-93  
 Completed By: D. Mason  
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 2
- 2. Reporting Period:..... June, 1993
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):..... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

\_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_

10. Reasons For Restrictions, If Any: \_\_\_\_\_

\_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

	This Month	YTD	Cumulative
11. Hours In Reporting Period.....	720.0	4343.0	176783.0
12. Number of Hours Reactor Was Critical.....	705.4	2896.1	116583.0
13. Reactor Reserve Shutdown Hours.....	0.0	0.0	328.1
14. Hours Generator On-Line.....	686.1	2831.1	114762.1
15. Unit Reserve Shutdown Hours.....	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH).....	1455871.2	6302818.7	267633892.5
17. Gross Electrical Energy Generated (MWH)....	477515.0	2096055.0	87291959.0
18. Net Electrical Energy Generated (MWH).....	455278.0	1993230.0	82783643.0
19. Unit Service Factor.....	95.3%	65.2%	64.9%
20. Unit Availability Factor.....	95.3%	65.2%	64.9%
21. Unit Capacity Factor (Using MDC Net).....	81.0%	58.8%	60.1%
22. Unit Capacity Factor (Using DER Net).....	80.2%	58.2%	59.4%
23. Unit Forced Outage Rate.....	4.7%	1.6%	14.1%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

\_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

25. If Shut Down at End of Report Period, Estimated Date of Start-up: \_\_\_\_\_

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

**UNIT SHUTDOWN AND POWER REDUCTION**  
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: June, 1993

Docket No.: 50-280  
 Unit Name: Surry Unit 1  
 Date: 07-07-93  
 Completed by: Anthony Xenakis  
 Telephone: (804) 365-2145

	(1)		(2)	(3)		(4)	(5)	
Date	Type	Duration Hours	Reason	Method of Shutting Down Rx	LER No.	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
930611	S	0	B	4	N/A	RC	HCV	Started ramp down to permit containment entry for isolation of 1-RC-HCV-1556B.

(1)  
 F: Forced  
 S: Scheduled

(2)  
 REASON:  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & Licensing Examination  
 F - Administrative  
 G - Operational Error (Explain)

(3)  
 METHOD:  
 1 - Manual  
 2 - Manual Scram.  
 3 - Automatic Scram.  
 4 - Other (Explain)

(4)  
 Exhibit G - Instructions for Preparation of Data Entry Sheets  
 for Licensee Event Report (LER) File (NUREG 0161)

(5)  
 Exhibit 1 - Same Source.

**UNIT SHUTDOWN AND POWER REDUCTION**  
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: June, 1993

Docket No.: 50-281  
 Unit Name: Surry Unit 2  
 Date: 07-07-93  
 Completed by: Anthony Xenakis  
 Telephone: (804) 365-2145

	(1)		(2)	(3)		(4)	(5)	
Date	Type	Duration Hours	Reason	Method of Shutting Down Rx	LER No.	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
930601	F	15.6	B	1	N/A	TA	TRB	Unit maintained at 7% power for main turbine balance shot.
930620	F	18.3	A	3	S2-93-002-00	SJ	P	Automatic reactor trip occurred due to a steam flow/feed flow mismatch coincident with a low steam generator level on "A" steam generator. This condition was directly related to the overcurrent trip of 2-FW-P-1A ("A" Main Feed Pump). The overcurrent trip was the result of a motor ground.

(1)  
 F: Forced  
 S: Scheduled

(2)  
 REASON:  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & Licensing Examination  
 F - Administrative  
 G - Operational Error (Explain)

(3)  
 METHOD:  
 1 - Manual  
 2 - Manual Scram.  
 3 - Automatic Scram.  
 4 - Other (Explain)

(4)  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5)  
 Exhibit 1 - Same Source.

**AVERAGE DAILY UNIT POWER LEVEL**

Docket No.: 50-280  
Unit Name: Surry Unit 1  
Date: 07-07-93  
Completed by: Pat Kessler  
Telephone: 365-2790

Month: June, 1993

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	787	17	771
2	779	18	771
3	785	19	772
4	784	20	766
5	786	21	771
6	787	22	761
7	787	23	774
8	785	24	770
9	782	25	772
10	780	26	774
11	767	27	771
12	689	28	770
13	778	29	769
14	782	30	763
15	781		
16	771		

**INSTRUCTIONS**

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.



**AVERAGE DAILY UNIT POWER LEVEL**

Docket No.: 50-281  
Unit Name: Surry Unit 2  
Date: 07-07-93  
Completed by: Pat Kessler  
Telephone: 365-2790

Month: June, 1993

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	126	17	777
2	763	18	775
3	764	19	780
4	769	20	143
5	770	21	399
6	784	22	510
7	782	23	481
8	779	24	472
9	776	25	482
10	777	26	478
11	776	27	482
12	776	28	484
13	778	29	477
14	781	30	475
15	777		
16	777		

**INSTRUCTIONS**

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

## SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: June, 1993

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

### UNIT ONE:

06/01/93 0000 This reporting period began with the Unit operating at 100% power, 815 MW.

06/11/93 2213 Started ramp down to permit containment entry for isolation of 1-RC-HCV-1556B.

06/12/93 0054 Stopped ramp; 60% power, 440 MW.

0315 Started ramp up; 60% power, 440 MW.

0758 Stopped ramp; 100% power, 810 MW.

06/30/93 2400 This reporting period ended with the Unit operating at 100% power, 810 MW.

### UNIT TWO:

06/01/93 0000 This reporting period began with the Reactor critical at 7% power to permit performance of a balance shot on the Main Turbine-Generator.

1536 Unit on line.

06/02/93 0223 Unit at 100% power, 825 MW.

0327 Reduced Unit to 98% power due to the receipt of high loop Delta T. on "A" Loop.

06/03/93 0046 Started ramp down to perform 2-PT-6 (Control Rod Partial Movement Test); 97% power, 800 MW.

0118 Stopped ramp; 95% power, 780 MW.

0310 Started ramp up; 95% power, 780 MW.

0345 Stopped ramp; 98% power, 805 MW.

06/05/93 1950 Started ramp up after "A" Loop Delta T. was calibrated; 98 % power, 800 MW.

2001 Stopped ramp; 100% power, 815 MW.

06/20/93 0413 2-FW-P-1A ("A" Main Feed Pump) tripped on overcurrent.

0414 Automatic Reactor trip occurred due to steam flow/feed flow mismatch in coincidence with low steam generator level on "A" steam generator.

1847 Reactor critical.

2233 Unit on line.

06/21/93 1420 Unit power at 67% power, 540 MW; power being maintained at 67% until return of "A" Main Feed Pump to service.

06/30/93 2400 This report period ended with the Unit operating at 67% power, 505 MW awaiting the return to service of "A" Main Feed Pump and a High Pressure Heater Drain Pump.

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** June, 1993

- |             |   |          |
|-------------|---|----------|
| TM S2-93-34 | <b>Temporary Modification</b><br>(Safety Evaluation No. 93-122)   | 06-02-93 |
|             | <p>This Temporary Modification (TM) installed electrical jumpers to bypass test switch FC-494-TA on the Train "A" Safety Injection (SI) isolation circuitry to allow its replacement without rendering the circuits inoperable.</p> <p>This TM continued to allow Train "A" SI isolation actuation on a valid signal and did not affect train "B," thus maintaining the design SI isolation functions. Double verification of jumper installation/removal and post maintenance testing were performed. Therefore, an unreviewed safety question does not exist.</p> |          |
| TM S2-93-35 | <b>Temporary Modification</b><br>(Safety Evaluation No. 93-123)   | 06-02-93 |
|             | <p>This Temporary Modification (TM) installed electrical jumpers to bypass test switch FC-474-TA on the Train "A" Safety Injection (SI) isolation circuitry to allow its replacement without rendering the circuits inoperable.</p> <p>This TM continued to allow Train "A" SI isolation actuation on a valid signal and did not affect train "B," thus maintaining the design SI isolation functions. Double verification of jumper installation/removal and post maintenance testing were performed. Therefore, an unreviewed safety question does not exist.</p> |          |
| TM S2-93-36 | <b>Temporary Modification</b><br>(Safety Evaluation No. 93-124)   | 06-02-93 |
|             | <p>This Temporary Modification (TM) installed electrical jumpers to bypass test switch FC-484-TA on the Train "A" Safety Injection (SI) isolation circuitry to allow its replacement without rendering the circuits inoperable.</p> <p>This TM continued to allow Train "A" SI isolation actuation on a valid signal and did not affect train "B," thus maintaining the design SI isolation functions. Double verification of jumper installation/removal and post maintenance testing were performed. Therefore, an unreviewed safety question does not exist.</p> |          |

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** June, 1993

TM S2-93-37

**Temporary Modification**  
(Safety Evaluation No. 93-126)

6-13-93

This Temporary Modification (TM) installed jumpers on Unit 2 Condensate Polishing Panel A to permit the use of 2-CP-AOV-217A until out of stock replacement solenoids for 2-CP-OV-217A and 2-CP-CV-217A can be obtained.

Electrical and mechanical jumpers were used to place identical installed spares in panel 2-CP-PNL-A in service. Normal functions and controls of 2-CP-AOV-217 are available. Consequently, operation of the Condensate Polishing System is maintained as described in section 10.3.5.2 of the SPS UFSAR. Also, double verification of jumper installation/removal and post maintenance testing were performed. Therefore, an unreviewed safety question does not exist.

TM S2-93-38

**Temporary Modification**  
(Safety Evaluation No. 91-145-R1)

6-17-93

This Temporary Modification (TM) was required to install a Temporary Vibration Monitoring System to the Unit 2 main turbine. New vibrational pickups were installed and are utilized for new monitors which are located locally on the turbine floor.

The turbine vibration monitors are not described in the UFSAR or a TS LCO and do not interface with a safety system. Therefore, an unreviewed safety question does not exist.

TM S2-93-39

**Temporary Modification**  
(Safety Evaluation No. 93-138)

6-22-93

This Temporary Modification (TM) disconnected the defective bearing temperature detector on the 555 ton mechanical refrigeration unit (1-CD-REF-1B) until repairs can be made. This TM was developed to allow continued operation of 1-CD-REF-1B without falsely tripping on high bearing oil temperature.

Bearing oil temperature is monitored as part of the normal turbine building operator rounds. If bearing oil temperature should increase, the operator would promptly notify maintenance to investigate.

The 555 ton mechanical chiller is not safety related and is not used to mitigate the consequences of a UFSAR Chapter 14 accident. The provisions outlined in TS 3.8 regarding containment temperature and pressure will be adhered to. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES  
THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** June, 1993

TSI-017                      **Technical Specification Interpretation**                      06-11-93  
(Safety Evaluation No. 93-125)

Technical Specification Interpretation TSI-017 was developed to clarify the power supply requirements for an operable residual heat removal (RHR) loop with the Unit at cold or refueling shutdown. (Re: Technical Specifications 1.0.D, 3.0.2, 3.1.A.1.d, 3.10.A.6, 3.10.A.7, 3.16).

The TSI concludes that: 1) a reactor coolant loop or RHR loop is operable if it is capable of removing decay heat; 2) the power supply requirements of Technical Specification (TS) 3.16 do not apply and, therefore, the RHR pumps need to have only one of their power sources operable; and 3) at least one of the two decay heat removal loops must be operating. This interpretation was discussed with on-site and Region II NRC personnel. The NRC agreed that this TSI is consistent with the applicable TS. Therefore, an unreviewed safety question does not exist.

JCO C-93-003                      **Justification For Continued Operation**                      06-15-93  
(Safety Evaluation No. 93-127)

Justification For Continued Operation (JCO) C-93-003 assessed continued operation with a postulated single failure in the rod control system which results in the potential for inadvertent withdrawal of a single rod control cluster assembly.

The assessment concluded that adequate indications and alarms are available to alert the operator to a potential rod control system malfunction and adequate procedures exist to address such a condition. Furthermore, Technical Specification operability requirements and limiting conditions for operation continue to be fully met. Therefore, an unreviewed safety question does not exist.

AC S-93-0615                      **Administrative Control**                      06-15-93  
(Safety Evaluation No. 93-128)

Administrative control of the main control room/emergency switchgear room boundary door was established to allow the door to be blocked open to permit temporary cables and ventilation ducts to be routed through the doorway.

A continuous fire watch was stationed at the door while it was open. The fire watch was responsible for disconnecting the temporary cables and ducts and closing the door in the event of a safety injection, fire, high energy line break, or turbine building flooding. This control action ensured the capability of rapidly restoring the main control room/emergency switchgear room pressure envelope to its required accident condition. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES  
THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** June, 1993

2-PT-15.8                      **Periodic Test Procedure**                      06-17-93  
(Safety Evaluation No. 93-129)

Periodic Test Procedure 2-PT-15.8, "Steam Generator Low-Low Level Test," was temporarily changed to provide instructions for verifying the operation of the Steam Generator "A" relays that are required to initiate an auxiliary feedwater pump start signal.

This change involves the lifting of electrical leads to actuation relays and cycling each one (one at a time) to verify contact continuity. This is a conservative action with regard to the logic for turbine driven auxiliary feedwater pump start and steam generator blowdown isolation. Only one motor driven auxiliary feedwater pump train will be in pull-to-lock at a time. The appropriate Technical Specification limiting conditions for operation will be entered, as required. Therefore, an unreviewed safety question does not exist.

FS 92-141                      **UFSAR Change**                      06-21-93  
(Safety Evaluation 93-137)

Updated Final Safety Analysis Report Change 93-141 revised section 9.1.2.6.7, "Resin Fill Tank," and Table 9.1-2, "Chemical and Volume Control System Principal Component Data Summary," to reflect the current method of adding resin to the chemical and volume control system (CVCS) demineralizers.

This change involves the use of a small funnel in lieu of the portable resin fill tank. The funnel is connected to the CVCS only during the resin fill process when the demineralizer is tagged out of service. This is a routine evolution that is not discussed in the Technical Specifications and will not affect the operation of the CVCS system. Therefore, an unreviewed safety question does not exist.

1-NSP-CC-001                      **Engineering Surveillance Procedures**                      06-24-93  
1-NSP-CC-002                      (Safety Evaluation No. 93-140)

Engineering Surveillance Procedures 1-NSP-CC-001, "Performance Test of Component Cooling Heat Exchanger 1-CC-E-1A," and 1-NSP-CC-002, "Performance Test of Component Cooling Heat Exchanger 1-CC-E-1B," were revised to install a Rosemount pressure transmitter in parallel with the component cooling heat exchanger (CCHX) service water (SW) delta pressure indicator to provide performance data.

The temporary transmitters were installed using fittings and tubing rated for system design pressures. The reliability or integrity of the system was not impacted. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES  
THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** June, 1993

QA Topical Report      **Quality Assurance Topical Report**      06-25-93  
(Safety Evaluation No. 93-141)

The QA Topical Report was revised to satisfy the requirement for an annual update.

The changes are primarily of an administrative or editorial nature and serve to enhance administrative controls and improve the effectiveness of the quality control program. The changes do not reflect or affect changes to the plant. Therefore, an unreviewed safety question does not exist.

TSI-014B      **Technical Specification Interpretation**      06-25-93  
(Safety Evaluation No. 93-142)

Technical Specification Interpretation TSI-014B was developed to describe the actions that need to be taken if a main feedwater regulating valve (MFRV) must be placed on its jack. (Re: Technical Specifications 3.7, Table 3.7-3).

The TSI directed controls permit no more than one MFRV on its jack at one time and for a period not to exceed eight hours. The TSI also requires that the motor operated valve (MOV) associated with the affected main Feedwater line be stroked partially closed to ensure it can be manually closed, if required. In addition, the controls require a dedicated control room operator to close the isolation MOV on a safety injection signal and a dedicated operator (located at the MFRV) to close the MFRV as required. These measures do not affect other accident mitigation systems and ensure that feedwater isolation is achieved within the main steam line break accident analysis assumptions. Accident probability and consequences are not increased. The proposed limited duration and controls to provide alternative isolation assure probability of equipment malfunction has not increased. The margin of safety is assured by diverse and redundant isolation control features. Therefore, an unreviewed safety question does not exist.

**TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** June, 1993

None During This Reporting Period



**CHEMISTRY REPORT**

**MONTH/YEAR:** June, 1993

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioact., $\mu\text{Ci/ml}$	4.31E-1	2.68E-1	3.73E-1	1.74E-1	4.59E-2	1.32E-1
Suspended Solids, ppm	$\leq 0.1$	$\leq 0.1$	$\leq 0.1$	$\leq 0.1$	$\leq 0.1$	$\leq 0.1$
Gross Tritium, $\mu\text{Ci/ml}$	2.10E-1	1.77E-1	1.91E-1	3.33E-1	2.41E-1	2.89E-1
$\text{I}^{131}$ , $\mu\text{Ci/ml}$	1.25E-3	7.29E-4	9.22E-4	2.31E-4	2.86E-5	7.89E-5
$\text{I}^{131}/\text{I}^{133}$	0.13	0.07	0.09	0.18	0.07	0.11
Hydrogen, cc/kg	40.5	20.6	31.8	42.9	15.6	33.0
Lithium, ppm	2.32	1.95	2.18	2.56	2.10	2.26
Boron - 10, ppm*	86.4	67.8	76.5	311.6	254.2	269.4
Oxygen, (DO), ppm	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$
Chloride, ppm	$\leq 0.050$	$\leq 0.001$	0.004	$\leq 0.050$	0.010	0.014
pH at 25 degree Celsius	7.11	7.00	7.06	6.38	6.16	6.28

\* Boron - 10 = Total Boron x 0.196

Comments:

None.

**FUEL HANDLING  
UNITS 1 & 2**

**MONTH/YEAR:** June, 1993

<u>New or Spent Fuel Shipment Number</u>	<u>Date Stored or Received</u>	<u>Number of Assemblies per Shipment</u>	<u>Assembly Number</u>	<u>ANSI Number</u>	<u>Initial Enrichment</u>	<u>New or Spent Fuel Shipping Cask Activity</u>
--	------------------------------------	--	----------------------------	------------------------	-------------------------------	---

No Fuel Received or Stored During this Reporting Period.

**DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED  
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS**

**MONTH/YEAR:** June, 1993

None During This Reporting Period