

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

June 10, 1993

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D. C. 20555

Serial No. 93-365  
NO/RPC:vlh  
Docket Nos. 50-280  
50-281  
License Nos. DPR-32  
DPR-37

Gentlemen:

**VIRGINIA ELECTRIC AND POWER COMPANY**  
**SURRY POWER STATION UNITS 1 AND 2**  
**MONTHLY OPERATING REPORT**

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of May 1993.

Very truly yours,



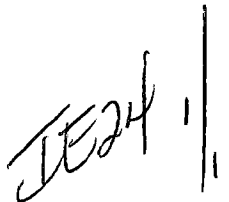
M. L. Bowling, Manager  
Nuclear Licensing & Programs

Enclosure

cc: U. S. Nuclear Regulatory Commission  
Region II  
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Suite 2900  
Atlanta, Georgia 30323

Mr. M. W. Branch  
NRC Senior Resident Inspector  
Surry Power Station

9306170374 930531  
PDR ADOCK 05000280  
R PDR



**VIRGINIA ELECTRIC AND POWER COMPANY  
SURRY POWER STATION  
MONTHLY OPERATING REPORT  
REPORT NO. 93-05**

Approved:

  
Station Manager

6-8-93  
Date

**TABLE OF CONTENTS**

<b>Section</b>	<b>Page</b>
Operating Data Report - Unit No. 1 .....	3
Operating Data Report - Unit No. 2 .....	4
Unit Shutdowns and Power Reductions - Unit No. 1.....	5
Unit Shutdowns and Power Reductions - Unit No. 2.....	6
Average Daily Unit Power Level - Unit No. 1.....	7
Average Daily Unit Power Level - Unit No. 2.....	8
Summary of Operating Experience - Unit No. 1 .....	9
Summary of Operating Experience - Unit No. 2 .....	9
Facility Changes That Did Not Require NRC Approval.....	11
Procedure or Method of Operation Changes That Did Not Require NRC Approval .....	14
Tests and Experiments That Did Not Require NRC Approval .....	16
Chemistry Report .....	17
Fuel Handling - Unit No. 1 .....	18
Fuel Handling - Unit No. 2 .....	18
Description of Periodic Test(s) Which Were Not Completed Within the Time Limits Specified in Technical Specifications .....	19

**OPERATING DATA REPORT**

Docket No.: 50-280  
 Date: 06-07-93  
 Completed By: D. Mason  
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 1
- 2. Reporting Period:..... May, 1993
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):..... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

\_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_

10. Reasons For Restrictions, If Any: \_\_\_\_\_

\_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

	This Month	YTD	Cumulative
11. Hours In Reporting Period .....	744.0	3623.0	179183.0
12. Number of Hours Reactor Was Critical .....	744.0	3516.4	118891.4
13. Reactor Reserve Shutdown Hours .....	0.0	0.0	3774.5
14. Hours Generator On-Line.....	744.0	3498.0	116773.4
15. Unit Reserve Shutdown Hours.....	0.0	0.0	3736.2
16. Gross Thermal Energy Generated (MWH).....	1811241.0	8295732.5	271915011.6
17. Gross Electrical Energy Generated (MWH)....	608020.0	2798010.0	88816263.0
18. Net Electrical Energy Generated (MWH).....	582758.0	2666448.0	84264308.0
19. Unit Service Factor.....	100.0%	96.5%	65.2%
20. Unit Availability Factor.....	100.0%	96.5%	67.3%
21. Unit Capacity Factor (Using MDC Net).....	100.3%	94.2%	60.7%
22. Unit Capacity Factor (Using DER Net).....	99.4%	93.4%	59.7%
23. Unit Forced Outage Rate.....	0.0%	3.5%	18.0%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

\_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

25. If Shut Down at End of Report Period, Estimated Date of Start-up: \_\_\_\_\_

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

**OPERATING DATA REPORT**

Docket No.: 50-281  
 Date: 06-07-93  
 Completed By: D. Mason  
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 2
- 2. Reporting Period:..... May, 1993
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):..... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

---



---

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_

10. Reasons For Restrictions, If Any: \_\_\_\_\_

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	This Month	YTD	Cumulative
11. Hours In Reporting Period .....	744.0	3623.0	176063.0
12. Number of Hours Reactor Was Critical .....	651.4	2190.7	115877.6
13. Reactor Reserve Shutdown Hours .....	0.0	0.0	328.1
14. Hours Generator On-Line.....	606.0	2145.0	114076.0
15. Unit Reserve Shutdown Hours.....	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH).....	1317613.0	4846947.5	266178021.3
17. Gross Electrical Energy Generated (MWH)....	435405.0	1618540.0	86814444.0
18. Net Electrical Energy Generated (MWH).....	415224.0	1537952.0	82328365.0
19. Unit Service Factor.....	81.5%	59.2%	64.8%
20. Unit Availability Factor.....	81.5%	59.2%	64.8%
21. Unit Capacity Factor (Using MDC Net).....	71.5%	54.4%	60.0%
22. Unit Capacity Factor (Using DER Net).....	70.8%	53.9%	59.3%
23. Unit Forced Outage Rate.....	2.1%	0.6%	14.1%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

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25. If Shut Down at End of Report Period, Estimated Date of Start-up: June 3, 1993

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

**UNIT SHUTDOWN AND POWER REDUCTION**  
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: May, 1993

Docket No.: 50-280  
 Unit Name: Surry Unit 1  
 Date: 06-07-93  
 Completed by: Anthony Xenakis  
 Telephone: (804) 365-2145

(1) Date	(1) Type	(1) Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	LER No.	(4) System Code	(5) Component Code	Cause & Corrective Action to Prevent Recurrence
930513	S	0	B	4	N/A	TA	V	Unit power was reduced to 80% in order to perform 1-OSP-TM- 001, turbine valve freedom test.

(1)  
 F: Forced  
 S: Scheduled

(2)  
 REASON:  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & Licensing Examination  
 F - Administrative  
 G - Operational Error (Explain)

(3)  
 METHOD:  
 1 - Manual  
 2 - Manual Scram.  
 3 - Automatic Scram.  
 4 - Other (Explain)

(4)  
 Exhibit G - Instructions for Preparation of Data Entry Sheets  
 for Licensee Event Report (LER) File (NUREG 0161)

(5)  
 Exhibit 1 - Same Source.

**UNIT SHUTDOWN AND POWER REDUCTION  
(EQUAL TO OR GREATER THAN 20%)**

REPORT MONTH: May, 1993

Docket No.: 50-281  
 Unit Name: Surry Unit 2  
 Date: 06-07-93  
 Completed by: Anthony Xenakis  
 Telephone: (804) 365-2145

(1) Date	(1) Type	(1) Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	LER No.	(4) System Code	(5) Component Code	Cause & Corrective Action to Prevent Recurrence
930531	F	12.9	B	4	N/A	TA	TRB	Ramped the Unit off line to perform balance shot on main turbine to correct high vibration condition. The reactor remained critical.

(1)  
 F: Forced  
 S: Scheduled

(2)  
 REASON:  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & Licensing Examination  
 F - Administrative  
 G - Operational Error (Explain)

(3)  
 METHOD:  
 1 - Manual  
 2 - Manual Scram.  
 3 - Automatic Scram.  
 4 - Other (Explain)

(4)  
 Exhibit G - Instructions for Preparation of Data Entry Sheets  
 for Licensee Event Report (LER) File (NUREG 0161)

(5)  
 Exhibit 1 - Same Source.

**AVERAGE DAILY UNIT POWER LEVEL**

Docket No.: 50-280  
Unit Name: Surry Unit 1  
Date: 06-07-93  
Completed by:  
Telephone:

Month: May, 1993

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	782	17	782
2	793	18	783
3	790	19	782
4	789	20	789
5	789	21	787
6	786	22	785
7	788	23	787
8	790	24	784
9	790	25	782
10	788	26	787
11	785	27	782
12	779	28	781
13	743	29	779
14	782	30	779
15	781	31	782
16	779		

**INSTRUCTIONS**

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.



**AVERAGE DAILY UNIT POWER LEVEL**

Docket No.: 50-281  
Unit Name: Surry Unit 2  
Date: 06-07-93  
Completed by:  
Telephone:

Month: May, 1993

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	0	17	693
2	0	18	691
3	0	19	694
4	0	20	726
5	0	21	766
6	125	22	768
7	277	23	770
8	445	24	768
9	677	25	777
10	715	26	783
11	725	27	781
12	714	28	783
13	686	29	785
14	695	30	785
15	698	31	279
16	695		

**INSTRUCTIONS**

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

## SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: May, 1993

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

### UNIT ONE:

05/01/93	0000	The reporting period began with the Unit operating at 100% power, 825 MWe.
05/13/93	0502	Started ramp down to perform 1-OSP-TM-001, turbine valve freedom test; 100% power, 820 MWe.
	0745	Stopped ramp; 80% power, 660 MWe.
	1253	Started ramp up; 80% power, 660 MWe.
	1410	Stopped ramp; 100% power, 820 MWe.
05/31/93	2400	The reporting period ended with the Unit operating at 100% power, 820 MWe.

### UNIT TWO:

05/01/93	0000	The reporting period began with the Unit at hot shutdown (HSD) preparing to start up from a scheduled refueling outage.
05/04/93	2035	Reactor critical.
05/05/93	1946	Unit at 2% power and stabilized on the main steam dumps.
05/06/93	0510	Unit on line.
	0545	Unit at 30% power on chemistry hold.
05/07/93	0820	Started ramp up from 30% power.
	1910	Stopped ramp; 61% power, 460 MWe.
05/08/93	1210	Started ramp up; 60% power, 465 MWe.
	2213	Stopped ramp for calorimetric; 70% power, 560 MWe.
05/09/93	0055	Started ramp up; 70% power, 560 MWe.
	0700	Stopped ramp; 90% power, 740 MWe.
05/10/93	0657	Started ramp up; 90% power, 740 MWe.
	0727	Stopped ramp; 92% power; reactor coolant system loop "A" delta T indicated 100% power.
	1202	Started ramp up; 92% power, 760 MWe.
	1333	Stopped ramp due to "C" steam generator feedwater flow oscillations; 95.5% power.
	1335	Reduced power to 93% power to stabilize feedwater flow on "C" steam generator.

**SUMMARY OF OPERATING EXPERIENCE**

**MONTH/YEAR:** May, 1993

**UNIT TWO (Continued):**

05/11/93	1208	Started ramp down to test No. 4 turbine governor valve; 91% power, 750 MWe.
	1212	Stopped ramp; 89.7% power, 740 MWe.
	1335	Started ramp up; 89.7% power, 740 MWe.
	1540	Stopped ramp at 98.5% power.
	1603	Started ramp down to stabilize Unit from 2% power swings resulting from feedwater flow oscillations on "C" steam generator.
	1614	Stopped ramp; 95% power, 780 MWe.
	1702	Started ramp down to place "C" feedwater regulating valve in automatic control mode; 92% power, 760 MWe.
	1841	Stopped ramp; 89.5% power, 740 MWe.
05/12/93	2107	Removed first point feedwater heaters from service for maintenance.
05/20/93	1438	Started ramp up; 91% power, 730 MWe.
	1919	Stopped ramp; 100% power, 825 MWe.
	2050	Power reduced to 98% to stabilize unit from "C" steam generator feedwater flow oscillations.
05/25/93	0437	Started ramp up; 98% power, 800 MWe.
	0511	Stopped ramp; 100% power, 820 MWe.
05/31/93	0552	Started ramp down due to turbine vibrations; 100% power, 820 MWe.
	1109	Unit off line.
	1115	Reactor power at 2%.
05/31/93	2400	The reporting period ended with the Unit off line and the reactor at 7% power.

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR: May, 1993**

- |  |   |          |
|--|---|----------|
| TM S2-93-21  | <b>Temporary Modification</b><br>(Safety Evaluation No. 93-105) | 05-01-93 |
| <p>Temporary Modification S2-93-21 installed electrical jumpers to maintain power to certain Unit 2 reactor protection (RP) system train A circuit relays during the replacement of a RP system relay FC3-XA.</p> <p>The activity was performed while Unit 2 was shutdown with the reactor trip breakers open. The operation of other systems was not affected. Double verification of jumper installation/removal and post maintenance testing were performed. Therefore, an unreviewed safety question does not exist.</p> |   |          |
| TM S2-93-22  | <b>Temporary Modification</b><br>(Safety Evaluation No. 93-106) | 05-01-93 |
| <p>Temporary Modification S2-93-22 installed electrical jumpers to maintain power to certain Unit 2 reactor protection (RP) system train B circuit relays during the replacement of a RP system relay FC3-XB.</p> <p>The activity was performed while Unit 2 was shutdown with the reactor trip breakers open. The operation of other systems was not affected. Double verification of jumper installation/removal and post maintenance testing were performed. Therefore, an unreviewed safety question does not exist.</p> |   |          |
| TM S2-93-23  | <b>Temporary Modification</b><br>(Safety Evaluation No. 93-107) | 05-01-93 |
| <p>Temporary Modification S2-93-23 installed electrical jumpers to maintain power to certain Unit 2 reactor protection (RP) system train B circuit relays during the replacement of a RP system relay P10-XB.</p> <p>The activity was performed while Unit 2 was shutdown with the reactor trip breakers open. The operation of other systems was not affected. Double verification of jumper installation/removal and post maintenance testing were performed. Therefore, an unreviewed safety question does not exist.</p> |   |          |
| FS 93-12   | <b>UFSAR Change</b><br>(Safety Evaluation 93-110)               | 05-06-93 |
| <p>Updated Final Safety Analysis Report Change 93-12 revised section 2.2.1, "Meteorological Program," to reflect previously approved changes to the meteorological monitoring system equipment and to include historical data.</p> <p>This was an administrative change that did not affect the control functions, data collection, or reporting requirements of the system. Therefore, an unreviewed safety question does not exist.</p>  |   |          |

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR: May, 1993**

- TM S2-93-31**                      **Temporary Modification**                      05-13-93  
(Safety Evaluation No. 93-113)
- Temporary Modification S2-93-31 installed a temporary 120 volt AC power strip from the radiation monitor cabinet 2-1 power supply to radiation monitor cabinet 2-2.
- This modification returned cabinet 2-2 to service following the failure of its Sola transformer. This alignment does not constitute any additional emergency bus loading since both cabinets are normally powered from the 2H1-1 motor control center. Therefore, an unreviewed safety question does not exist.
- TM S1-93-06**                      **Temporary Modification**                      05-22-93  
(Safety Evaluation No. 93-115)
- Temporary Modification S1-93-06 installed temporary blind flanges and pipe supports to facilitate the replacement of service water system strainer 1-VS-S-1A and valves 1-SW-301 and 1-SW-302.
- The blind flanges met the criteria for the piping class for which they were used and an in-service leak test was performed to verify the adequacy of their installation. The applicable Technical Specification limiting conditions for operation were adhered to during this activity. Therefore, an unreviewed safety question does not exist.
- TM S2-93-32**                      **Temporary Modification**                      05-24-93  
(Safety Evaluation No. 93-117)
- Temporary Modification S2-93-32 removed a control fuse to prevent a spurious Unit 2 Hi Hi Consequence Limiting Safeguards (CLS) train B signal while trouble shooting to identify a DC ground.
- This modification affected only Hi Hi CLS train B. The Hi Hi CLS train A remained capable of performing its design function. Technical Specification action statements were adhered to during this activity. Therefore, an unreviewed safety question does not exist.
- TM S2-93-33**                      **Temporary Modification**                      05-24-93  
(Safety Evaluation No. 93-116)
- Temporary Modification S2-93-32 de-terminated an electrical lead to prevent a spurious Unit 2 Hi Consequence Limiting Safeguards (CLS) train B safety injection (SI) actuation signal while trouble shooting to identify a DC ground.
- This modification affected only the Hi CLS train B. The Hi CLS train A remained capable of performing its design function. Technical Specification action statements were adhered to during this activity. Therefore, an unreviewed safety question does not exist.

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR: May, 1993**

TM S1-93-07

**Temporary Modification**  
(Safety Evaluation No. 93-119)

05-25-93

Temporary Modification S1-93-07 installed a temporary ventilation duct and configured dampers to redirect cooling air from the Protective Clothing Issue and Dressing area to supply cooling air to the Health Physics Count Room. This modification will remain in place until repairs can be made to the air conditioner (1-VS-AC-3) that normally supplies the Health Physics Count Room.

This modification will improve the reliability of the radiation monitoring equipment used in post accident analysis (while 1-VS-AC-3 is out of service) and will not affect any safety-related systems or components. Therefore, an unreviewed safety question does not exist.

EWR 90-275

**Engineering Work Request**  
(Safety Evaluation No. 90-242)

05-26-93

Engineering Work Request 90-275 installed a manually operated drain valve in Unit 1 waste gas system pipe spool 2"-GW-24-154.

This modification provides a means for draining accumulated condensate downstream of relief valve 01-RV-GW-103. This will improve the reliability of the relief valve since an accumulation of condensate could impair its operation (through corrosion) and increase the potential for water hammer upon valve actuation. This change does not affect the function or performance characteristics of the waste gas system. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES  
THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** May, 1993

**SE 93-109                      Safety Evaluation                      05-04-93**

Safety Evaluation 93-109 assessed continued operation of Units 1 and 2 with the containment sump sampling capability of the high radiation sampling sink (HRSS) out of service.

The assessment concluded that the Emergency Response Procedures provide alternate methods for making core damage assessments. In addition, the HRSS containment sump sampling capability is not assumed in the basis for any Technical Specification. Therefore, an unreviewed safety question does not exist.

**2-OP-5.1.5                      Operating Procedure                      05-05-93**  
**(Safety Evaluation No. 93-103)**

Operating Procedure 2-OP-5.1.5, "Venting the PRT," was revised to provide direction for continuously venting the Unit 2 pressurizer relief tank (PRT) to the containment atmosphere via PRT pressure transmitter PT-2472. This change was made in order to maintain PRT pressure low and to allow stabilization of pressurizer safety and power operated relief valve tailpipe conditions.

The containment atmosphere will be monitored by the containment particulate and gaseous radiation monitors during the venting process. Activity samples will be obtained every 12 hours and weekly grab samples will be obtained on a weekly basis in the event these monitors become inoperable. The venting process will be performed at a slow rate and will be terminated if the containment radiation monitors alarm or if samples indicate adverse (explosive) gas concentrations. The release of the PRT contents to the containment atmosphere does not pose any design or operational concerns and has no impact on the UFSAR accident analyses. Therefore, an unreviewed safety question does not exist.

**1-OP-5.1.5                      Operating Procedure                      05-11-93**  
**(Safety Evaluation No. 93-112)**

Operating Procedure 1-OP-5.1.5, "Venting the PRT," was revised to provide direction for continuously venting the Unit 1 pressurizer relief tank (PRT) to the containment atmosphere via PRT pressure transmitter PT-1472. This change was made in order to maintain PRT pressure low and to allow stabilization of pressurizer safety and power operated relief valve tailpipe conditions.

The containment atmosphere will be monitored by the containment particulate and gaseous radiation monitors during the venting process. Activity samples will be obtained every 12 hours and weekly grab samples will be obtained on a weekly basis in the event these monitors become inoperable. The venting process will be performed at a slow rate and will be terminated if the containment radiation monitors alarm or if samples indicate adverse (explosive) gas concentrations. The release of the PRT contents to the containment atmosphere does not pose any design or operational concerns and has no impact on the UFSAR accident analyses. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES  
THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** May, 1993

1-PT-17.7

**Periodic Test Procedure**  
(Safety Evaluation No. 93-114)

05-14-93

Periodic Test Procedure 1-PT-17.7, "Recirculating Spray HX Service Water Radiation Monitor Pump Test," was temporarily changed to install electrical jumpers to allow the consequence limiting safeguards (CLS) automatic start features of Unit 1 recirculating spray heat exchanger service water radiation monitor sample pumps 1-SW-P-5A, 1-SW-P-5B, 1-SW-P-5C, and 1-SW-P-5D to be tested.

Installation of the electrical jumpers will not affect other systems or components. The applicable Emergency Operating Procedures (EOP) will be revised to require verification that the subject pumps have automatically started upon receipt of a Hi Hi CLS signal. The EOPs will also direct appropriate actions to minimize the potential for a radioactive release in the event the pumps do not start. Therefore, an unreviewed safety question does not exist.

0-ECM-1403-01

**Electrical Corrective Maintenance Procedure**  
(Safety Evaluation No. 93-121)

05-27-93

Electrical Corrective Maintenance Procedure 0-ECM-1403-01, "RCP Motor Disconnect and Connect," was revised to provide instructions for implementing temporary modifications to defeat the Units 1 and 2 reactor coolant pump (RCP) breaker interlocks. These modifications are needed to facilitate the performance of an uncoupled RCP motor run without relying on bearing lift oil pressure switches and loop stop valve position.

The breaker interlocks that are defeated by this procedure are not needed during an uncoupled RCP motor run since there is no flow path requirements. The performance of this procedure will not affect the reactor coolant system and does not affect other accident mitigating equipment. Therefore, an unreviewed safety question does not exist.



**TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR: May, 1993**

None During This Reporting Period

**CHEMISTRY REPORT**

**MONTH/YEAR: May, 1993**

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioact., $\mu\text{Ci/ml}$	5.01E-1	3.17E-1	3.79E-1	1.61E-1	7.43E-4	1.02E-1
Suspended Solids, ppm	$\leq 0.1$	$\leq 0.1$	$\leq 0.1$	$\leq 0.1$	$\leq 0.1$	$\leq 0.1$
Gross Tritium, $\mu\text{Ci/ml}$	2.18E-1	1.99E-1	2.08E-1	1.55E-1	2.08E-2	9.37E-2
$\text{I}^{131}$ , $\mu\text{Ci/ml}$	1.13E-3	5.99E-4	8.56E-4	5.05E-4	3.49E-5	1.07E-4
$\text{I}^{131}/\text{I}^{133}$	0.12	0.07	0.09	0.22	0.07	0.14
Hydrogen, cc/kg	41.7	28.9	35.2	45.0	24.5	37.2
Lithium, ppm	2.27	2.07	2.19	3.54	1.25	2.39
Boron - 10, ppm*	102.3	84.7	93.3	469.4	257.0	325.2
Oxygen, (DO), ppm	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$
Chloride, ppm	0.006	0.001	0.002	0.025	0.003	0.013
pH at 25 degree Celsius	7.08	6.82	6.95	6.52	5.53	6.21

\* Boron - 10 = Total Boron x 0.196

Comments:

Unit 2 On-line 5/6/93.

**FUEL HANDLING  
UNITS 1 & 2**

**MONTH/YEAR: May 1993**

<u>New or Spent Fuel Shipment Number</u>	<u>Date Stored or Received</u>	<u>Number of Assemblies per Shipment</u>	<u>Assembly Number</u>	<u>ANSI Number</u>	<u>Initial Enrichment</u>	<u>New or Spent Fuel Shipping Cask Activity</u>
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No Fuel Received or Stored During this Reporting Period.

**DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED  
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS**

**MONTH/YEAR: May, 1993**

None During This Reporting Period.