

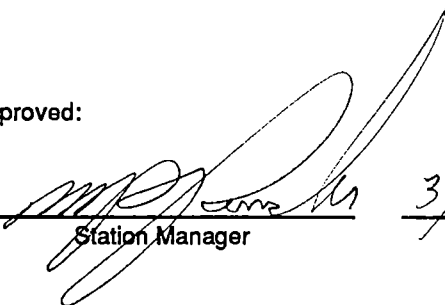
ATTACHMENT 1

**SURRY POWER STATION
MONTHLY OPERATING REPORT
FEBRUARY 1993**

9303160291 930270
PDR ADOCK 05000280
R PDR

**VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION
MONTHLY OPERATING REPORT
REPORT NO. 93-02**

Approved:



Station Manager

3/5/93

Date

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OPERATING DATA REPORT

Docket No.: 50-280
 Date: 03-05-93
 Completed By: D. Mason
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 1
- 2. Reporting Period:..... February 1993
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):..... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting Period	672.0	1416.0	176976.0
12. Number of Hours Reactor Was Critical	626.8	1309.4	116684.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	3774.5
14. Hours Generator On-Line.....	616.5	1291.0	114566.4
15. Unit Reserve Shutdown Hours.....	0.0	0.0	3736.2
16. Gross Thermal Energy Generated (MWH).....	1490967.3	3128331.5	266747610.6
17. Gross Electrical Energy Generated (MWH)....	503805.0	1057270.0	87075523.0
18. Net Electrical Energy Generated (MWH).....	479250.0	1005618.0	82603478.0
19. Unit Service Factor.....	91.7%	91.2%	64.7%
20. Unit Availability Factor.....	91.7%	91.2%	66.8%
21. Unit Capacity Factor (Using MDC Net).....	91.3%	90.9%	60.2%
22. Unit Capacity Factor (Using DER Net).....	90.5%	90.1%	59.2%
23. Unit Forced Outage Rate.....	8.3%	8.8%	18.3%

24. Shutdowns Schedule Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down at End of Report Period Estimated Date of Start-up: _____

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

OPERATING DATA REPORT

Docket No.: 50-281
 Date: 03-05-93
 Completed By: D. Mason
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 2
- 2. Reporting Period:..... February 1993
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):.... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting Period	672.0	1416.0	173856.0
12. Number of Hours Reactor Was Critical	672.0	1416.0	115102.9
13. Reactor Reserve Shutdown Hours	0.0	0.0	328.1
14. Hours Generator On-Line.....	672.0	1416.0	113347.0
15. Unit Reserve Shutdown Hours.....	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH).....	1481355.0	3293768.2	26462484.0
17. Gross Electrical Energy Generated (MWH)....	497405.0	1103470.0	86299374.0
18. Net Electrical Energy Generated (MWH).....	471180.0	1047693.0	81838106.0
19. Unit Service Factor	100.0%	100.0%	65.2%
20. Unit Availability Factor.....	100.0%	100.0%	65.2%
21. Unit Capacity Factor (Using MDC Net).....	89.8%	94.7%	60.4%
22. Unit Capacity Factor (Using DER Net).....	89.0%	93.9%	59.7%
23. Unit Forced Outage Rate.....	0.0%	0.0%	14.2%

24. Shutdowns Schedule Over Next 6 Months (Type, Date, and Duration of Each):

Refueling, March 6, 1993 -- 60 days.

25. If Shut Down at End of Report Period Estimated Date of Start-up: _____

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: February 1993

Docket No.: 50-280
 Unit Name: Surry Unit 1
 Date: 03-05-93
 Completed by: Anthony Xenakis
 Telephone: (804) 365-2145

(1) Date	(1) Type	(2) Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	(4) LER No.	(4) System Code	(5) Component Code	Cause & Corrective Action to Prevent Recurrence
930209	F	55.5	A	3	1-93-002	JC	RLY	An automatic reactor trip occurred while performing PT-8.1 when a shunt trip coil relay failed on the "A" Reactor Trip Breaker.

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets
 for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source.

UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: February 1993

Docket No.: 50-281
 Unit Name: Surry Unit 2
 Date: 03-05-93
 Completed by: Anthony Xenakis
 Telephone: (804) 365-2145

	(1)	(2)	(3)	(4)	(5)			
Date	Type	Duration Hours	Reason	Method of Shutting Down Rx	LER No.	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence

No entries for this reporting period.

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets
 for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-280
Unit Name: Surry Unit 1
Date: 03-05-93
Completed by: M. A. Negrón
Telephone: (804) 365-2795

Month: February 1993

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	788	17	784
2	787	18	781
3	786	19	783
4	784	20	783
5	780	21	784
6	782	22	784
7	786	23	785
8	786	24	784
9	351	25	783
10	0	26	783
11	80	27	783
12	727	28	785
13	785	29	
14	784	30	
15	783	31	
16	782		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-281
Unit Name: Surry Unit 2
Date: 03-05-93
Completed by: M. A. Negron
Telephone: (804) 365-2795

Month: February 1993

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	773	17	689
2	770	18	684
3	762	19	679
4	756	20	675
5	747	21	670
6	746	22	667
7	742	23	661
8	732	24	657
9	725	25	653
10	718	26	649
11	708	27	646
12	708	28	639
13	698	29	
14	701	30	
15	693	31	
16	688		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: February 1993

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

02-01-93	0000	This reporting period began with the Unit operating at 100% power, 825 MWe.
02-09-93	1043	Automatic reactor trip occurred while performing PT-8.1 when a shunt trip coil relay failed on the "A" Reactor Trip Breaker.
02-11-93	0755	Reactor critical.
	1811	Unit on line.
02-12-93	0357	Stopped ramp at 97% power/800 MWe, due to #4 governor valve indicating open with EHC isolated to the valve.
	0606	Started ramp down to close and repair #4 governor valve; 97% power, 800 MWe.
	0630	Stopped ramp; 92% power, 755 MWe.
	1430	Started ramp up; 91% power, 750 MWe.
	1700	Stopped ramp; 100% power, 825 MWe.
02-28-93	2400	This reporting period ended with the Unit operating at 100% power, 820 MWe.

UNIT TWO

02-01-93	0000	This reporting period began with the Unit operating at 100% power, 815 MWe.
02-02-93	1200	Started Unit coastdown to maintain primary coolant average temperature (Tave) due to fuel depletion; 100% power, 815 MWe.
02-28-93	2400	This reporting period ended with the Unit in a coastdown due to fuel depletion at 81.5% power, 675 MWe.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: February 1993

- | | | |
|---------------|--|----------|
| EWR 90-246 | Engineering Work Request
(Safety Evaluation No. 90-168) | 02-04-93 |
| | <p>Engineering Work Request EWR 90-246 installed foundations and services to support the erection of a modular office building located within the protected area.</p> <p>The installation is a non-nuclear plant facility that has no adverse impact on any safety-related or nonsafety-related system, component, or structure. Therefore, an unreviewed safety question was not created.</p> | |
| DCP 89-18-3 | Design Change Package
(Safety Evaluation No. 90-116) | 02-09-93 |
| | <p>Design Change Package DCP 89-18-3 installed an air conditioning unit inside mechanical equipment room (MER) 4 to improve the environmental parameters of the room.</p> <p>The nonsafety-related modification does not negatively affect safety-related components or systems and will extend the life of equipment in MER 4. Therefore, an unreviewed safety question was not created.</p> | |
| DCP 92-27-3 | Design Change Package | 02-09-93 |
| | <p>Design Change Package DCP 92-27-3 replaced the Unit 1 and 2 charging pump service water temperature control valves.</p> <p>The design of the new valves is similar to the original valves, but is less susceptible to clogging by service water particulate. The modification did not adversely impact the operation or alter the design requirements or design features of any system. Therefore, an unreviewed safety question was not created.</p> | |
| EDSCR S92-081 | Equipment Data System Change Request
(Safety Evaluation No. 93-020) | 02-09-93 |
| | <p>Equipment Data System Change Request S92-081 changes the classification of the Unit 1 and 2 containment gas and particulate radiation monitors (1-RM-RMS-159, 1-RM-RMS-160, 2-RM-RMS-259, and 2-RM-RMS-260) from "Safety-Related" (SR) to "Nonsafety-Related with Special Quality/Regulatory Requirements" (NSQ).</p> <p>An evaluation of this change determined that these radiation monitors do not perform a safe shutdown function and are not required for accident mitigation. Furthermore, there is no failure mechanism associated with these radiation monitors that could result in a design basis accident. Therefore, an unreviewed safety question is not created.</p> | |

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: February 1993

TM S1-93-03 **Temporary Modification** 02-16-93
(Safety Evaluation No. 93-026)

Temporary Modification TM S1-93-03 installs temporary tubing to route the discharge of the Unit 1 recirculating spray heat exchanger (RSHX) service water (SW) radiation monitoring pumps 1-SW-P-5A, 5B, 5C, and 5D to a spare penetration on the Unit 2 SW 24 inch pipe in the Unit 2 safeguards. This modification supports the installation of flow instrumentation in the Unit 2 RSHX SW 24 inch piping.

The temporary tubing is pressure rated to withstand the RSHX SW radiation monitoring system pump discharge. The RSHX SW radiation monitoring system will be tested following installation and removal of the modification and will remain operable. Therefore, an unreviewed safety question is not created.

TM S1-93-04 **Temporary Modification** 02-21-93
(Safety Evaluation No. 93-030)

Temporary Modification TM S1-93-04 installed a temporary 120 volt AC power strip from the radiation monitor cabinet 1-1 power supply to radiation monitor cabinet 1-2.

This modification returned cabinet 1-2 to service following the failure of its Sola transformer. This alignment does not constitute any additional emergency bus loading since both cabinets are normally powered from the 1H1-1 motor control center. Therefore, an unreviewed safety question was not created.

FS 92-121 **UFSAR Change** 02-25-93
(Safety Evaluation 93-032)

Updated Final Safety Analysis Report (UFSAR) Section 10.3.9.2, "[Bearing Cooling Water System] Description" is being revised to reflect a change in the chemical treatment of the bearing cooling (BC) system.

The corrosion inhibitor for the bearing cooling system was changed from a sodium molybdate solution to sodium nitrate in 1992. This change was approved by the Virginia State Water Control Board. The new chemical treatment does not adversely affect the BC or any other plant system. Therefore, an unreviewed safety question was not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: February 1993

TM S1-93-05

Temporary Modification
(Safety Evaluation No. 93-034)

02-26-93

Temporary Modification TM S1-93-05 removed the inspection plates from the Unit 1 A reactor coolant pump stator cooler. This modification was made to lower the stator temperature which had risen due to a degraded stator cooling heat exchanger.

This modification was recommended by the pump designer (Westinghouse) and does not impact reactor coolant pump operability. Therefore, an unreviewed safety question was not created.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: February 1993

1-PT-18.8A
2-PT-18.8A

Periodic Test Procedures
(Safety Evaluation No. 93-018)

02-04-93

Periodic Test Procedures 1-PT-18.8A and 2-PT-18.8A, "Charging Pump Component Cooling Performance," were changed ("one-time only") to permit manual operator action in lieu of the automatic start function for component cooling (CC) pumps 1/2-CC-P-2A and 1/2-CC-P-2B during data collection for pump curve verification.

The standby pump will be placed in the "Off" mode (thereby defeating the automatic start capability) for approximately ten minutes while the running pump's discharge valve is throttled to obtain various flow rates. The running and standby pumps will both be considered fully operable since an operator will be present to initiate the required manual actions to ensure sufficient CC flow is maintained to the charging pump seals. Therefore, an unreviewed safety question is not created.

1-PT-18.8
2-PT-18.8

Periodic Test Procedures
(Safety Evaluation No. 93-019)

02-04-93

Periodic Test Procedures 1-PT-18.8 and 2-PT-18.8, "Charging Pump Service Water Performance," were changed ("one-time only") to permit manual operator action in lieu of the automatic start function for service water (SW) pumps 1/2-SW-P-10A and 1/2-SW-P-10B during data collection for pump curve verification.

The standby pump will be placed in the "Off" mode (thereby defeating the automatic start capability) for approximately ten minutes while the running pump's discharge valve is throttled to obtain various flow rates. The running and standby pumps will both be considered fully operable since an operator will be present to initiate the required manual actions to ensure sufficient SW flow is maintained to the charging pump lube oil coolers. Therefore, an unreviewed safety question is not created.

0-ECM-0105-01
0-ECM-0105-02

Electrical Corrective Maintenance Procedure
(Safety Evaluation No. 93-022)

02-11-93

Electrical Corrective Maintenance Procedures 0-ECM-0105-01, "Appendix R ELT Inspection and Rework" and 0-ECM-0105-02, "Fire Code ELT Inspection and Rework" were developed to provide instructions for conducting performance discharge tests for emergency lighting.

These procedures may be implemented while the Units are at power and will be performed for only one common (to both Units) fire area at a time or for one fire area on each Unit at a time. Battery powered lanterns are provided in the Appendix R cabinet for use by Operations personnel during the test and for the 24 hour post test period in which the emergency lighting batteries are being recharged. This activity will not affect the probability of occurrence or the consequences of any previously analyzed accidents. Therefore, an unreviewed safety question is not created.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: February 1993

1-OP-RC-005 **Operating Procedure** 02-11-93
2-OP-RC-005 (Safety Evaluation No. 93-023)

Operating Procedures 1-OP-RC-005 and 2-OP-RC-005, "Draining the RCS From Flange Level to Mid-Nozzle (Reduced Inventory)" were revised to change the residual heat removal (RHR) low flow alarm setpoint and the RHR heat exchanger outlet high temperature alarm setpoint to compensate for reduced inventory operating conditions.

These changes establish an additional annunciator setpoint for use during reduced inventory operation since RHR flow during reduced inventory operation can be less than the normal low flow annunciator setpoint, resulting in a continuous alarm condition. The changes do not affect control or protective circuits and will ensure annunciation of a loss of RHR flow. Therefore, an unreviewed safety question is not created.

1-OP-5.1.5 **Operating Procedure** 02-12-93
(Safety Evaluation No. 93-025)

Operating Procedure 1-OP-5.1.5, "Venting the PRT" was temporarily changed to provide instructions for venting the pressurizer relief tank (PRT) to the containment atmosphere.

The procedure imposes special venting requirements to significantly reduce the hazards associated with venting hydrogen from the PRT. The release of the PRT contents to the containment atmosphere does not pose any design or operational concerns and has no impact on the UFSAR accident analyses. Therefore, an unreviewed safety question is not created.

2-ECM-2403-02 **Electrical Corrective Maintenance Procedure** 02-16-93
(Safety Evaluation No. 93-027)

Electrical Corrective Maintenance Procedure 2-ECM-2403-02, "RSS Transformer B Outage with Backfeed of Transfer Bus E" was revised to provide instructions for implementing temporary modifications to defeat automatic tripping and lockout of selected 4160 volt breakers associated with transfer bus E. These modifications will ensure a reliable electric plant lineup during a reserve station service transformer B outage while on Unit 2 backfeed to transfer bus E.

This procedure will be performed with Unit 2 at cold shutdown and will provide for the normal Unit 2 backfeed line-up. Safety systems and emergency diesel generators will remain available to Unit 1. Therefore, an unreviewed safety question is not created.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: February 1993

2-ECM-2403-01 **Electrical Corrective Maintenance Procedure** 02-16-93
(Safety Evaluation No. 93-028)

Electrical Corrective Maintenance Procedure 2-ECM-2403-01, "RSS Transformer A Outage with Backfeed of Transfer Bus D" was revised to provide instructions for implementing temporary modifications to defeat automatic tripping and lockout of selected 4160 volt breakers associated with transfer bus D. These modifications will ensure a reliable electric plant lineup during a reserve station service transformer A outage while on Unit 2 backfeed to transfer bus D.

This procedure will be performed with Unit 2 at cold shutdown and will provide for the normal Unit 2 backfeed line-up. Safety systems and emergency diesel generators will remain available to Unit 1. Therefore, an unreviewed safety question is not created.

1/2-OPT-ZZ-001 **Operations Periodic Test Procedures** 02-18-93
1/2-OPT-ZZ-002 **Electrical Periodic Test Procedures**
1/2-EPT-1801-01 (Safety Evaluation No. 93-029)
1/2-EPT-1801-02

Operations Periodic Test Procedures 1/2-OPT-ZZ-001, "ESF Actuation with Undervoltage and Degraded Voltage - 1H [2H] Bus", 1/2-OPT-ZZ-002, "ESF Actuation with Undervoltage and Degraded Voltage - 1J [2J] Bus" and Electrical Periodic Test Procedures 1/2-EPT-1801-01, "Bus 1H [2H] Protective Relay Testing" and 1/2-EPT-1801-02, "Bus 1J [2J] Protective Relay Testing" were revised/developed to provide instructions for lifting electrical leads and installing jumpers to support engineered safety feature (ESF) and undervoltage testing.

The Unit will be at cold shutdown during the performance of these tests. 1/2-OPT-ZZ-001 and 1/2-OPT-ZZ-002 will be performed prior to refueling operations and will require two methods of core cooling. 1/2-EPT-1801-01 and 1/2-EPT-1801-02 will be performed with no fuel in the reactor vessel and no fuel movement in progress. The procedurally controlled temporary modifications and administrative controls will ensure compliance with the Technical Specifications. Therefore, an unreviewed safety question is not created.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: February 1993

2-OPT-SI-006

Operations Periodic Test Procedure
(Safety Evaluation No. 93-031)

02-23-93

Operations Periodic Test Procedure, 2-OPT-SI-006, "SI Accumulator Discharge Check Valves Full Open Test" was developed to verify that SI accumulator discharge check valves are free to open and exhibit full stroke operation.

The Unit will be at cold shutdown and the equipment and systems will be operated within design limits during the performance of this test. Measures will be taken to preclude the introduction of gas into the reactor coolant and residual heat removal systems. Reactivity addition will be precluded by ensuring the accumulator boron concentration is greater than the reactor coolant system (RCS) concentration and by ensuring accumulator temperature is greater than the RCS shutdown margin temperature. Therefore, an unreviewed safety question is not created.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: February 1993

No entries for this reporting period.

CHEMISTRY REPORT

MONTH/YEAR: February 1993

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioact., $\mu\text{Ci/ml}$	4.95E-1	2.33E-2	3.30E-1	3.57E-1	1.63E-1	2.59E-1
Suspended Solids, ppm	≤ 0.1	≤ 0.1	≤ 0.1	≤ 0.1	≤ 0.1	≤ 0.1
Gross Tritium, $\mu\text{Ci/ml}$	2.38E-1	1.51E-1	1.99E-1	8.22E-2	4.89E-2	2.65E-1
I^{131} , $\mu\text{Ci/ml}$	1.06E-3	8.69E-5	6.58E-4	4.92E-3	3.16E-4	2.43E-3
I^{131}/I^{133}	0.14	0.07	0.09	0.32	0.11	0.20
Hydrogen, cc/kg	47.5	28.2	37.2	48.2	31.4	39.0
Lithium, ppm	2.30	2.06	2.15	1.13	0.64	0.78
Boron - 10, ppm*	226.4	132.5	157.7	0.98	0.22	0.37
Oxygen, (DO), ppm	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005
Chloride, ppm	0.004	0.002	0.003	0.005	≤ 0.001	0.002
pH at 25 degree Celsius	6.76	6.42	6.70	9.03	8.32	8.68

* Boron - 10 = Total Boron x 0.196

Comments:

None.

**FUEL HANDLING
 UNITS 1 & 2**

MONTH/YEAR: February 1993

New or Spent Fuel Shipment Number	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
Unit 2 Batch 14 Shipment 6	02-11-93	14	6W2	LM0X47	4.0	Total Shipment 17.29 Ci
			0W9	LM0X2N	3.8	
			1W3	LM0X2S	3.8	
			3W2	LM0X45	3.8	
			6W5	LM0X4A	4.0	
			6W6	LM0X4B	4.0	
			3W1	LM0X44	3.8	
			6W3	LM0X48	4.0	
			6W7	LM0X4C	4.0	
			6W8	LM0X4D	4.0	
			2W9	LM0X42	3.8	
			6W4	LM0X49	4.0	
			3W0	LM0X43	3.8	
			6W1	LM0X46	4.0	

**DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS**

MONTH/YEAR: February 1993

None During This Reporting Period.