

VIRGINIA ELECTRIC AND POWER COMPANY

RICHMOND, VIRGINIA 23261

March 9, 1992

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

Serial No. 92-033A
NO/ETS
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
PROPOSED TECHNICAL SPECIFICATION FOR THE
MAIN CONTROL ROOM AND EMERGENCY SWITCHGEAR ROOM
AIR CONDITIONING SYSTEM - SUPPLEMENTAL SUBMITTAL

In support of the Main Control Room (MCR) and Emergency Switchgear Room (ESGR) air conditioning system upgrades, several Technical Specifications amendments have been necessary for both Units 1 and 2. Initially, both the Units 1 and 2 Technical Specifications were concurrently changed to provide interim operating restrictions (Amendment Nos. 129/129, dated May 30, 1989). Subsequently, each unit's Technical Specifications were individually modified to account for unit specific Air Handling Unit (AHU) replacements, which occurred during sequential refueling outages. Unit specific Technical Specification changes were granted which removed the interim operating restrictions on the AHUs (Amendments 145, dated October 11, 1990 and 151, dated April 11, 1991, for Units 1 and 2 respectively). On January 22, 1992, Virginia Electric and Power Company requested an amendment, in the form of a change to the Technical Specifications, for Surry Power Station Units 1 and 2 to complete additional air conditioning system upgrades. This change would permit continued operation for seven days with one of two AHUs in each ventilation space out of service to install chilled water connections for planned modifications to add chiller capacity.

As a shared system with cross-referenced Technical Specifications these multiple unit-specific changes to Section 3.23 of the Technical Specifications have resulted in several administrative inconsistencies between the individual unit's Technical Specifications (i.e., incorrect cross-reference requirements, format, and pagination problems). To clarify this condition, we are resubmitting our January 22, 1992 request for both units to include a complete cross-referenced Section 3.23. This change provides consistency in the cross-referencing of opposite unit's requirements by integrating the previously NRC approved unit-specific Technical Specification changes (Amendments 145 and 151) into both operating licenses. The modified Technical Specification change request is included in Attachment 1. The changes for the administrative cross-referencing between units are identified with a single bar and the original change request is identified with double bars. Please replace Attachment 2 of the January 22, 1992 letter with Attachment 1 of this letter.

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The change request of January 22, 1992 was reviewed and approved by the Station Nuclear Safety and Operating Committee and the Management Safety Review Committee. It was determined that those proposed changes do not involve an unreviewed safety question as defined in 10 CFR 50.59 or a significant hazards consideration as defined in 10 CFR 50.92. The resubmittal of the entire Section 3.23 to cross-reference previously NRC approved, unit-specific NRC license amendments into both operating licenses is administrative in nature and does not impact the safety evaluation or the previously submitted Significant Hazards Consideration Determination. The previously approved, unit-specific NRC SERs form the basis for the cross-reference resubmittal.

Very truly yours,



W. L. Stewart
Senior Vice President - Nuclear

Attachments

cc: U. S. Nuclear Regulatory Commission
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