

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

January 15, 1992

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D. C. 20555

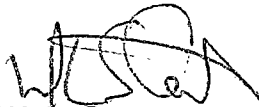
Serial No. 92-031  
NO/RPC:vlh  
Docket Nos. 50-280  
50-281  
License Nos. DPR-32  
DPR-37

Gentlemen:

**VIRGINIA ELECTRIC AND POWER COMPANY**  
**SURRY POWER STATION UNITS 1 AND 2**  
**MONTHLY OPERATING REPORT**

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of December 1991.

Very truly yours,



W. L. Stewart  
Senior Vice President - Nuclear

Enclosure

cc: U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, N. W.  
Suite 2900  
Atlanta, Georgia 30323

Mr. M. W. Branch  
NRC Senior Resident Inspector  
Surry Power Station

9201220150-911231  
PDR ADCK 05000280  
R PDR

FE24  
11

**VIRGINIA ELECTRIC AND POWER COMPANY  
SURRY POWER STATION  
MONTHLY OPERATING REPORT  
REPORT NO. 91-12**

Approved:

David A. Christian  
Station Manager

1-10-92  
Date

## TABLE OF CONTENTS

<b>Section</b>	<b>Page</b>
Operating Data Report - Unit No. 1 .....	3
Operating Data Report - Unit No. 2 .....	4
Unit Shutdowns and Power Reductions - Unit No. 1.....	5
Unit Shutdowns and Power Reductions - Unit No. 2.....	6
Average Daily Unit Power Level - Unit No. 1.....	7
Average Daily Unit Power Level - Unit No. 2.....	8
Summary of Operating Experience - Unit No. 1 .....	9
Summary of Operating Experience - Unit No. 2 .....	9
Facility Changes That Did Not Require NRC Approval.....	10
Procedure or Method of Operation Changes That Did Not Require NRC Approval .....	13
Tests and Experiments That Did Not Require NRC Approval .....	15
Chemistry Report .....	16
Fuel Handling - Unit No. 1 .....	17
Fuel Handling - Unit No. 2 .....	17
Description of Periodic Test(s) Which Were Not Completed Within the Time Limits Specified in Technical Specifications .....	19

### OPERATING DATA REPORT

Docket No.: 50-280  
 Date: 1-8-92  
 Completed By: M. A. Negron  
 Telephone: 804-365-2795

- 1. Unit Name:..... Surry Unit 1
- 2. Reporting Period:..... December 1991
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):..... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

---



---



---

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_

10. Reasons For Restrictions, If Any: \_\_\_\_\_

---



---



---

	This Month	YTD	Cumulative
11. Hours In Reporting Period .....	744.0	8760.0	166776.0
12. Number of Hours Reactor Was Critical .....	744.0	8760.0	108234.2
13. Reactor Reserve Shutdown Hours .....	0.0	0.0	3774.5
14. Hours Generator On-Line.....	744.0	8760.0	106240.2
15. Unit Reserve Shutdown Hours.....	0.0	0.0	3736.2
16. Gross Thermal Energy Generated (MWH).....	1812061.0	20896766.4	247116676.7
17. Gross Electrical Energy Generated (MWH)....	610335.0	6938365.0	80515188.0
18. Net Electrical Energy Generated (MWH).....	580655.0	6590937.0	76374066.0
19. Unit Service Factor.....	100.0%	100.0%	63.7%
20. Unit Availability Factor.....	100.0%	100.0%	65.9%
21. Unit Capacity Factor (Using MDC Net).....	99.9%	96.3%	59.1%
22. Unit Capacity Factor (Using DER Net).....	99.0%	95.5%	58.1%
23. Unit Forced Outage Rate.....	0.0%	0.0%	19.3%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Refueling, February 28, 1991, 64 days.

---



---



---

25. If Shut Down at End of Report Period Estimated Date of Start-up: \_\_\_\_\_

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

## OPERATING DATA REPORT

Docket No.: 50-281  
 Date: 1-8-92  
 Completed By: M. A. Negron  
 Telephone: 804-365-2795

- 1. Unit Name:..... Surry Unit 2
- 2. Reporting Period:..... December 1991
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):..... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

---



---



---

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_

10. Reasons For Restrictions, If Any: \_\_\_\_\_

---



---



---

	This Month	YTD	Cumulative
11. Hours In Reporting Period .....	744.0	8760.0	163656.0
12. Number of Hours Reactor Was Critical .....	574.3	6035.8	105208.1
13. Reactor Reserve Shutdown Hours .....	0.0	0.0	328.1
14. Hours Generator On-Line.....	553.1	5890.3	103460.9
15. Unit Reserve Shutdown Hours.....	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH).....	1324560.5	12743213.8	240878682.1
17. Gross Electrical Energy Generated (MWH)....	442635.0	4203735.0	78433849.0
18. Net Electrical Energy Generated (MWH).....	421229.0	3985213.0	74363938.0
19. Unit Service Factor.....	74.3%	67.2%	63.2%
20. Unit Availability Factor.....	74.3%	67.2%	63.2%
21. Unit Capacity Factor (Using MDC Net).....	72.5%	58.3%	58.3%
22. Unit Capacity Factor (Using DER Net).....	71.8%	57.7%	57.7%
23. Unit Forced Outage Rate.....	25.7%	17.0%	15.4%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

---



---



---

25. If Shut Down at End of Report Period Estimated Date of Start-up: \_\_\_\_\_

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

**UNIT SHUTDOWN AND POWER REDUCTION**  
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: December 1991

Docket No.: 50-280  
 Unit Name: Surry Unit 1  
 Date: 1-8-92  
 Completed by: M. A. Negron  
 Telephone: 804-365-2795

	(1)	(2)	(3)	(4)	(5)			
Date	Type	Duration Hours	Reason	Method of Shutting Down Rx	LER No.	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence

None during this reporting period.

(1)  
 F: Forced  
 S: Scheduled

(2)  
 REASON:  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & Licensing Examination  
 F - Administrative  
 G - Operational Error (Explain)

(3)  
 METHOD:  
 1 - Manual  
 2 - Manual Scram.  
 3 - Automatic Scram.  
 4 - Other (Explain)

(4)  
 Exhibit G - Instructions for Preparation of Data Entry Sheets  
 for Licensee Event Report (LER) File (NUREG 0161)

(5)  
 Exhibit 1 - Same Source.

**UNIT SHUTDOWN AND POWER REDUCTION**  
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: December 1991

Docket No.: 50-281  
 Unit Name: Surry Unit 2  
 Date: 1-8-92  
 Completed by: M. A. Negron  
 Telephone: 804-365-2795

	(1)		(2)	(3)		(4)	(5)	
Date	Type	Duration Hours	Reason	Method of Shutting Down Rx	LER No.	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
12-11-91	F	163.5	A	1	N/A	AB	FI	Leak at 2-RC-FE-2492, Repaired.
12-17-91	F	27.5	A	3	N/A	JB	FCV	Failure of 'B' Feed Reg. Vlv., Repaired.

(1)  
 F: Forced  
 S: Scheduled

(2)  
 REASON:  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & Licensing Examination  
 F - Administrative  
 G - Operational Error (Explain)

(3)  
 METHOD:  
 1 - Manual  
 2 - Manual Scram.  
 3 - Automatic Scram.  
 4 - Other (Explain)

(4)  
 Exhibit G - Instructions for Preparation of Data Entry Sheets  
 for Licensee Event Report (LER) File (NUREG 0161)

(5)  
 Exhibit 1 - Same Source.

### AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-280  
Unit Name: Surry Unit 1  
Date: 1-8-92  
Completed by: M. A. Negron  
Telephone: 804-365-2795

Month: December 1991

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	782.9	17	778.0
2	781.9	18	782.2
3	776.6	19	782.7
4	771.1	20	783.9
5	781.2	21	783.9
6	782.1	22	782.3
7	780.7	23	783.7
8	778.1	24	782.6
9	781.0	25	781.7
10	772.0	26	783.0
11	774.2	27	782.0
12	779.1	28	782.8
13	780.2	29	782.3
14	780.6	30	782.2
15	779.9	31	782.8
16	777.0		

#### INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.



### AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-281  
Unit Name: Surry Unit 2  
Date: 1-8-92  
Completed by: M. A. Negron  
Telephone: 804-365-2795

Month: December 1991

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	768.1	17	0.0
2	775.5	18	0.0
3	774.4	19	486.5
4	772.0	20	776.8
5	766.8	21	776.9
6	768.5	22	777.0
7	778.2	23	776.5
8	778.8	24	775.7
9	779.8	25	775.8
10	753.8	26	772.8
11	23.4	27	777.2
12	0.0	28	778.7
13	0.0	29	779.3
14	0.0	30	779.6
15	0.0	31	779.5
16	0.0		

#### INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

## SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: December 1991

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

### UNIT ONE

12-01-91 0000 This report period started with the Unit operating at 100% power, 820 MWe.  
12-31-91 2400 This report period ended with the Unit operating at 100% power, 820 MWe.

### UNIT TWO

12-01-91 0000 This report period started with the Unit operating at 100% power, 795 MWe.  
12-10-91 2102 Started Unit Ramp down to Cold Shutdown to repair a leak on 2-RC-FE-2492.  
12-11-91 0317 Unit off line.  
0349 Reactor manually tripped.  
12-17-91 1304 Reactor critical.  
2245 Unit on line.  
2254 Unit off line from an automatic reactor trip. This was due to a failure of "B" main feedwater regulating valve to maintain a demand position, resulting in high "B" Steam Generator level.  
12-18-91 1626 Reactor critical.  
12-19-91 0222 Unit on line. Commence ramping to 100%.  
1753 Unit at 100% power, 815 MWe.  
12-31-91 2400 This report period ended with the Unit operating at 100% power, 820 MWe.

## FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: December 1991

TM S2-91-50

**Temporary Modification**  
(Safety Evaluation No. 91-255)

12-02-91

This Temporary Modification (TM) lifted an electrical lead to remove the undervoltage lockout for the Unit 2 Charging Pump "A" to prevent the potential loss of high head safety injection (assuming the active failure to be the "B" charging pump powered from the emergency bus "J" or emergency diesel generator number 3) in the event of a simultaneous design basis accident and emergency bus "H" undervoltage condition.

Each unit has three charging pumps with two required to be operable. One pump is normally operating and the other pump is in standby designed to start automatically upon receipt of an engineered safety feature (ESF) signal. One of the three pumps can be supplied from either emergency electrical bus ("H" or "J" bus). The charging pump electrical supply breakers are equipped with interlock logic to ensure that only one charging pump can be supplied from an emergency bus at any given time. One such interlock for the "A" pump can stop the pump or prevent the pump from auto-starting in one operational configuration. This TM ensures that Unit 2 Charging Pump "A" will continue to operate or automatically start following receipt of an ESF signal coincident with a loss of offsite power. This TM does not affect Emergency Diesel Generator loading and assures the proper equipment response during accident conditions. Therefore, an unreviewed safety question is not created.

TM S2-91-51

**Temporary Modification**  
(Safety Evaluation No. 91-263)

12-13-91

This Temporary Modification (TM) mechanically holds open damper 2-VS-F-60A on the Unit 2 control rod drive mechanism (CRDM) shroud cooling fan to ensure adequate cooling is provided to the reactor head area.

This TM does not affect other systems and will ensure proper operation of the CRDM shroud cooling fan until the subject damper can be repaired. Therefore, an unreviewed safety question is not created.

## FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: December 1991

[continued]

FS 91-40

**UFSAR Change**  
(Safety Evaluation 91-273)

12-26-91

The Updated Final Safety Analysis Report (UFSAR) Figure 9.12-1 "Fuel Transfer System" was revised to correct errors associated with drawing references of plant elevations and dimensions of the fuel handling system inside Containment.

This change was administrative only. No physical changes to the station were made. Therefore an unreviewed safety question does not exist.

## FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: December 1991

[continued]

TM S2-91-52      **Temporary Modification**      12-26-91  
(Safety Evaluation No. 91-274)

This Temporary Modification (TM) installed electrical jumpers to permit the replacement of Unit 2 "A" train Reactor Protection System relay, 02-RP-REL-PRBXA.

This TM was used to maintain downstream RPS "A" train relays energized during this activity. Double verification of the jumper installation/removal and post maintenance testing were performed. The RPS "B" train logic was not affected. Therefore, an unreviewed safety question is not created.

AC S1-91-1231      **Administrative Control**      12-31-91  
(Safety Evaluation No. 91-277)

Administrative Control of the Unit 2 Emergency Switchgear room (ESGR)/Turbine Building door is established to facilitate operations and maintenance activities. The controls include stationing a person at the door to the ESGR from the Turbine building. The person's duties are to immediately isolate and disconnect temporary hoses routed through the door and close the door in the event of a main control room (MCR) air bottle bank discharge. The person also performs the required fire watch responsibilities while the door is open.

The controls for the subject door ensure the capability of the MCR/ESGR pressure envelope to achieve the proper positive pressure for minimizing the potential for exposure of MCR personnel to radioactivity releases in the event of design basis accidents. Therefore, an unreviewed safety question is not created.

DCP 87-031-2      **Design Change Package**      12-27-91  
(Safety Evaluation No. 91-017)

This Design Change Package (DCP) installs a new Unit 2 Chemical and Volume Control System (CVCS) Letdown line trip valve, 2-CH-TV-2204A (inside Containment).

The new trip valve, 2-CH-TV-2204A, performs the isolation function previously served by CVCS Letdown Flow Orifice isolation valves 2-CH-HCV-2200A, B, and C and will improve leakage testing concerns, reduce personnel exposure and improve penetration design features.

The new trip valve and related limit switches were designed consistent with the original design basis criteria for containment isolation valves and the requirements of Regulatory Guide 1.97. CVCS operation is unchanged by these modifications. Therefore, an unreviewed safety question is not created.

**PROCEDURES OR METHOD OF OPERATION CHANGES  
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: December 1991

0-MCM-1918-01      **Mechanical Corrective Maintenance Procedure**      12-09-91  
(Safety Evaluation No. 91-256)

Mechanical Corrective Maintenance Procedure 0-MCM-1918-01, "On-Line Leak Repairs" was implemented to correct a gasket leak from the Unit 2 Reactor Coolant system (RCS) flow element, 2-RC-FE-2492.

This repair involved the use of furmanite and installation of a clamp on RCS pipe spool, 3"-RC-477-1502. Due to the nature of this application, furmanite was not injected into the RCS. Thus, RCS chemistry was not affected. An evaluation of the additional weight of the clamp was determined to not adversely impact the seismic qualification of the affected RCS piping. Therefore, an unreviewed safety question is not created.

1-PT-18.1      **Operations Periodic Test Procedure**      12-12-91  
(Safety Evaluation No. 91-259)

Unit 1 Operations Periodic Test Procedure 1-PT-18.1, "Low Head Safety Injection Pump Test" was revised ("one-time only" change) to permit the use of temporary transmitters to enable the measurement of Low Head Safety Injection Pump pressure during the few seconds following pump start.

The use of the temporary transmitter does not affect the operation of the subject pump. As a precaution, however, the procedure change includes direction to isolate the transmitter if the Safety Injection system is required to perform its safety function. Therefore, an unreviewed safety question is not created.

MMP-C-FS-260      **Corrective Mechanical Maintenance Procedure**      12-12-91  
(Safety Evaluation No. 91-262)

Corrective Mechanical Maintenance Procedure, MMP-C-FS-260, "Freeze Sealing - Liquid Nitrogen Method - Single Freeze" was performed on the Unit 2 Reactor Coolant System (RCS) "C" loop Resistance Temperature Detector (RTD) manifold return piping to isolate and allow flow element, 2-RC-FE-2492, to be repaired.

The Unit was at Cold Shutdown during the installation and special RCS pressure and temperature requirements were imposed. Limiting conditions were established to limit potential RCS leakage, as a result of freeze seal failure, and to ensure the leakage would remain within the make-up capacity of the Safety Injection system. Contingency actions were also in place to minimize the consequences of a freeze seal failure. In addition, Nondestructive Examination of the affected piping was performed prior to and following the seal installation. Therefore, an unreviewed safety question does not exist.

**PROCEDURES OR METHOD OF OPERATION CHANGES  
THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** December 1991

[continued]

MMP-C-FS-260      **Corrective Mechanical Maintenance Procedure**      12-13-91  
(Safety Evaluation No. 91-264)

Corrective Mechanical Maintenance Procedure, MMP-C-FS-260, "Freeze Sealing - Liquid Nitrogen Method - Single Freeze" was performed on the Unit 2 Reactor Coolant System (RCS) "B" loop Resistance Temperature Detector (RTD) manifold return piping to isolate and allow flow element, 2-RC-FE-2491, to be repaired.

The Unit was at Cold Shutdown during the installation and special RCS pressure and temperature requirements were imposed. Limiting conditions were established to limit potential RCS leakage, as a result of freeze seal failure, and to ensure the leakage would remain within the make-up capacity of the Safety Injection system. Contingency actions were also in place to minimize the consequences of a freeze seal failure. In addition, Nondestructive Examination of the affected piping was performed prior to and following the seal installation. Therefore, an unreviewed safety question does not exist.

2-OP-5.2.1      **Operating Procedure**      12-16-91  
(Safety Evaluation No. 91-267)

Unit 2 Operating Procedure 2-OP-5.2.1, "Starting Any Reactor Coolant Pump" was revised ("one-time only" change) to permit lifting the lead to the malfunctioning Reactor Coolant system loop stop valve 2-RC-MOV-2593 stem switch. This action circumvents the associated interlock with the "B" reactor coolant pump (RCP) starting circuit, enabling the pump to be started.

This change does not affect the protective functions of the RCP circuitry. The original purpose of the loop stop valve stem switch/RCP starting circuit interlock was to allow two loop operation, which is currently not allowed. Therefore, an unreviewed safety question is not created.

**TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** December 1991

None during this reporting period.



**CHEMISTRY REPORT**

MONTH/YEAR: December 1991

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioact., $\mu\text{Ci/ml}$	1.08E0	5.19E-1	7.27E-1	2.98E-1	5.80E-3	1.31E-1
Suspended Solids, ppm	0.0	0.0	0.0	0.0	0.0	0.0
Gross Tritium, $\mu\text{Ci/ml}$	2.62E-1	1.76E-1	2.27E-1	4.55E-1	2.27E-1	3.18E-1
$\text{I}^{131}$ , $\mu\text{Ci/ml}$	1.57E-2	1.42E-3	3.45E-3	6.88E-4	6.91E-5	2.98E-4
$\text{I}^{131}/\text{I}^{133}$	0.19	0.07	0.11	0.16	0.06	0.09
Hydrogen, cc/kg	46.2	15.6	28.6	38.7	26.3	31.4
Lithium, ppm	1.44	0.99	1.22	2.99	1.26	2.17
Boron - 10, ppm*	27.2	11.2	19.2	455.3	218.9	308.5
Oxygen, (DO), ppm	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$	$\leq 0.005$
Chloride, ppm	0.004	$\leq 0.001$	0.002	0.005	0.003	0.004
pH at 25 degree Celsius	7.67	7.41	7.53	6.56	5.68	6.30

\* Boron - 10 = Total Boron x 0.196

Comments: **Unit One:** RCS dissolved hydrogen concentration decreased to 15.6 cc/kg on 12-25-91 due to letdown isolation. The recommended limits for RCS dissolved hydrogen are 25-50 cc/kg. When letdown was returned to service on 12-27-91, the dissolved hydrogen concentration increased to 29.5 cc/kg. Dissolved hydrogen was low for a total of 36 hours and 20 minutes.

**FUEL HANDLING  
 UNITS 1 & 2**

MONTH/YEAR: December 1991

Units One and Two

New or Spent Fuel Shipment No.	Date Stored	Number for Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity Level	
1	12-3-91	12	0J1	LM0 UY7	3.8	N/A	
	12-3-91	12	0J5	LM0 UYB	3.8	N/A	
	12-3-91	12	1J0	LM0 UYG	3.8	N/A	
	12-3-91	12	0J9	LM0 UYF	3.8	N/A	
	12-3-91	12	1J1	LM0 UYH	3.8	N/A	
	12-3-91	12	2J6	LM0 UYY	3.8	N/A	
	12-3-91	12	1J8	LM0 UYQ	3.8	N/A	
	12-3-91	12	0J8	LM0 UYE	3.8	N/A	
	12-3-91	12	1J9	LM0 UYR	3.8	N/A	
	12-3-91	12	0J2	LM0 UY8	3.8	N/A	
	12-3-91	12	3J8	LM0 UZA	4.0	N/A	
	12-3-91	12	4J3	LM0 UZF	4.0	N/A	
	2	12-5-91	12	1J5	LM0 UYM	3.8	N/A
		12-5-91	12	1J6	LM0 UYN	3.8	N/A
12-5-91		12	2J3	LM0 UYV	3.8	N/A	
12-5-91		12	5J5	LM0 UZT	4.0	N/A	
12-5-91		12	3J7	LM0 UZ9	4.0	N/A	
12-5-91		12	4J2	LM0 UZE	4.0	N/A	
12-5-91		12	4J1	LM0 UZD	4.0	N/A	
12-5-91		12	2J5	LM0 UYX	3.8	N/A	
12-5-91		12	2J4	LM0 UYW	3.8	N/A	
12-5-91		12	0J7	LM0 UYD	3.8	N/A	
12-5-91		12	0J4	LM0 UYA	3.8	N/A	
12-5-91	12	2J7	LM0 UYZ	3.8	N/A		

**FUEL HANDLING**

MONTH/YEAR: December 1991

[continued]

**Units One and Two**

New or Spent Fuel Shipment No.	Date Stored	Number for Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity Level	
3	12-10-91	12	2J0	LM0 UYS	3.8	N/A	
	12-10-91	12	2J1	LM0 UYT	3.8	N/A	
	12-10-91	12	1J3	LM0 UYK	3.8	N/A	
	12-10-91	12	1J4	LM0 UYL	3.8	N/A	
	12-10-91	12	1J7	LM0 UYP	3.8	N/A	
	12-10-91	12	2J2	LM0 UYU	3.8	N/A	
	12-10-91	12	0J6	LM0 UYC	3.8	N/A	
	12-10-91	12	6J2	LM0 VOU	4.0	N/A	
	12-10-91	12	3J6	LM0 UZ8	4.0	N/A	
	12-10-91	12	3J9	LM0 UZB	4.0	N/A	
	12-10-91	12	1J2	LM0 UYJ	3.8	N/A	
	12-10-91	12	5J9	LM0 VOR	4.0	N/A	
	4	12-17-91	12	0J3	LM0 UY9	3.8	N/A
		12-17-91	12	5J7	LM0 VOP	4.0	N/A
12-17-91		12	4J5	LM0 UZH	4.0	N/A	
12-17-91		12	5J2	LM0 UZQ	4.0	N/A	
12-17-91		12	4J0	LM0 UZC	4.0	N/A	
12-17-91		12	4J8	LM0 UZL	4.0	N/A	
12-17-91		12	6J3	LM0 VOV	4.0	N/A	
12-17-91		12	5J3	LM0 UZR	4.0	N/A	
12-17-91		12	5J8	LM0 VOQ	4.0	N/A	
12-17-91		12	4J7	LM0 UZK	4.0	N/A	
12-17-91		12	5J4	LM0 UZS	4.0	N/A	
12-17-91	12	5J6	LM0 VON	4.0	N/A		

**DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED  
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS**

**MONTH/YEAR:** December 1991

None During This Reporting Period.