



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
101 MARIETTA STREET, N.W.  
ATLANTA, GEORGIA 30323

ENCLOSURE 1

Facility Licensee: Virginia Electric and Power Company

Facility Name: Surry Power Station

Facility Docket Nos.: 50-280 and 50-281

Facility License Nos.: DRP-32 and DPR-37

Examinations were administered at Surry Power Station near Surry Virginia.

Chief Examiner: R.S. BALDWIN *R.S. Baldwin* 4/19/91  
Richard S. Baldwin Date Signed

Approved By: Lawrence L. Lawyer *Lawrence L. Lawyer* 4/19/91  
Lawrence L. Lawyer, Chief Date Signed  
Operator Licensing Section 1  
Division of Reactor Safety

SUMMARY

An Initial Retake examination was administered on March 5, 1991.

One Operating test (Simulator only) was administered to one SRO applicant. The SRO passed this examination.

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REPORT DETAILS

1. Examiner

\*R. Baldwin

\*Chief Examiner

2. Facility Representatives at exit meeting

H. McCallum, Operations Coordinator Training

J. McCarthy, Superintendent of Operations

A. Friedman, Superintendent of Nuclear Training

B. Bilyeu, Station Licensing

3. Resident Inspector at exit meeting

S. Tingen

4. At the conclusion of the site visit the examiner met with representatives of the plant staff to discuss the results of the examinations.

The following items concerning this examination were discussed:

- 1) Procedures for the simulator development were not initially provided. When material was requested the facility did not provide the FR-S.1 and FR-S.2 procedures. In order to develop valid and accurate simulator examinations the facility was reminded that all examination material requested must be received in a timely manner. Delays in receiving or omissions of material can lead to examination schedule changes.
- 2) During the performance of AOP-16.00, Excessive RCS Leakage, it was noted that at step 6 if the RCS leak rate is greater than 150 gpm then the RNO required the operator to "Manually initiate SI and GO TO 1-E-O, Reactor Trip or Safety Injection, step 1." This procedure does not address a manual reactor trip prior to manual safety injection. This same concern was previously addressed in the Emergency Operating Procedure inspection report No.'s 50-280/90-09 and 50-281/90-09, Appendix B item #35. Appendix B is covered under IFI 280, 281/90-09-07.

Manual safety injection prior to manually tripping the reactor places the plant in an undesirable condition if an ATWS simultaneously occurs. This is due to the fact that a manual SI causes a main feed water isolation.

This feed water isolation coupled with an ATWS may cause an undesired loss of heat sink, as well as a severe reactor plant pressure transient. This item will continue to be followed under IFI 280, 281/90-09-07.

ENCLOSURE 2

SIMULATOR FIDELITY REPORT

Facility Licensee: Virginia Electric and Power Company

Facility Docket Nos.: 50-280 and 50-281

Operating Tests Administered on: March 5, 1991

This form is to be used only to report observations. These observations do not constitute audit or inspection findings and are not, without further verification and review, indicative of noncompliance with 10 CFR 55.45(b). These observations do not affect NRC certification or approval of the simulation facility other than to provide information which may be used in future evaluations. No licensee action is required in response to these observations.

During the conduct of the simulator portion of the operating tests, the following item was observed:

ITEM

Spurious Radiation Alarms

DESCRIPTION

During the evaluation, numerous spurious radiation alarms annunciated that were not related to the scenarios being conducted.