

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

April 15, 1991

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

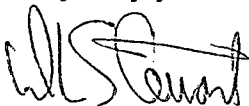
Serial No. 91-206
NO/RPC:vlh
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of March 1991.

Very truly yours,



W. L. Stewart
Senior Vice President - Nuclear

Enclosure

cc: U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, N. W.
Suite 2900
Atlanta, Georgia 30323

Mr. W. E. Holland
NRC Senior Resident Inspector
Surry Power Station

9104180292 910331
FDR ADOCK 05000280
R FDR

IE 24
11

**VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION
MONTHLY OPERATING REPORT
REPORT NO. 91-03**

Approved:

David A. Christman
Station Manager

4-8-91
Date

TABLE OF CONTENTS

Section	Page
Operating Data Report - Unit No. 1	3
Operating Data Report - Unit No. 2	4
Unit Shutdowns and Power Reductions - Unit No. 1.....	5
Unit Shutdowns and Power Reductions - Unit No. 2.....	6
Average Daily Unit Power Level - Unit No. 1.....	7
Average Daily Unit Power Level - Unit No. 2.....	8
Summary of Operating Experience - Unit No. 1	9
Summary of Operating Experience - Unit No. 2	9
Facility Changes That Did Not Require NRC Approval.....	10
Procedure or Method of Operation Changes That Did Not Require NRC Approval	15
Tests and Experiments That Did Not Require NRC Approval	16
Chemistry Report	17
Fuel Handling - Unit No. 1	18
Fuel Handling - Unit No. 2	18
Description of Periodic Test(s) Which Were Not Completed Within the Time Limits Specified in Technical Specifications	20

OPERATING DATA REPORT

Docket No.: 50-280
 Date: 04-03-91
 Completed By: M. A. Negron
 Telephone: 804-365-2795

- 1. Unit Name:..... Surry Unit 1
- 2. Reporting Period:..... March 1991
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):.... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting.....	744.0	2160.0	160176.0
12. Number of Hours Reactor Was Critical.....	744.0	2160.0	101634.2
13. Reactor Reserve Shutdown Hours.....	0.0	0.0	3774.5
14. Hours Generator On-Line.....	744.0	2160.0	99640.2
15. Unit Reserve Shutdown Hours.....	0.0	0.0	3736.2
16. Gross Thermal Energy Generated (MWH).....	1791731.0	5232540.0	231452450.3
17. Gross Electrical Energy Generated (MWH)....	604070.0	1769450.0	75346273.0
18. Net Electrical Energy Generated (MWH).....	573968.0	1685391.0	71468520.0
19. Unit Service Factor.....	100.0%	100.0%	62.2%
20. Unit Availability Factor.....	100.0%	100.0%	64.5%
21. Unit Capacity Factor (Using MDC Net).....	98.8%	99.9%	57.6%
22. Unit Capacity Factor (Using DER Net).....	97.9%	99.0%	56.6%
23. Unit Forced Outage Rate.....	0.0%	0.0%	20.3%

24. Shutdowns Schedule Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down at End of Report Period Estimated Date of Start-up: _____

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

OPERATING DATA REPORT

Docket No.: 50-281
 Date: 04-03-91
 Completed By: M. A. Negron
 Telephone: 804-365-2795

- 1. Unit Name:..... Surry Unit 2
- 2. Reporting Period:..... March 1991
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe): 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting.....	744.0	2160.0	157056.0
12. Number of Hours Reactor Was Critical.....	720.0	2136.0	101308.3
13. Reactor Reserve Shutdown Hours.....	0.0	0.0	328.1
14. Hours Generator On-Line.....	698.8	2114.8	99685.4
15. Unit Reserve Shutdown Hours.....	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH).....	1387703.0	4493943.2	232629411.5
17. Gross Electrical Energy Generated (MWH)....	467470.0	1510315.0	75740429.0
18. Net Electrical Energy Generated (MWH).....	442007.0	1432860.0	71811585.0
19. Unit Service Factor.....	93.9%	97.9%	63.5%
20. Unit Availability Factor.....	93.9%	97.9%	63.5%
21. Unit Capacity Factor (Using MDC Net).....	76.1%	84.9%	58.7%
22. Unit Capacity Factor (Using DER Net).....	75.4%	84.2%	58.0%
23. Unit Forced Outage Rate.....	0.0%	0.0%	15.0%

24. Shutdowns Schedule Over Next 6 Months (Type, Date, and Duration of Each):
 Refueling - 3-30-91 - 67 days

25. If Shut Down at End of Report Period Estimated Date of Start-up: _____ 06-05-91

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: March 1991

Docket No.: 50-280
 Unit Name: Surry Unit 1
 Date: 04-03-91
 Completed by: M. A. Negron
 Telephone: 804-365-2795

(1) Date	(1) Type	(2) Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	LER No.	(4) System Code	(5) Component Code	Cause & Corrective Action to Prevent Recurrence
03-07-91	S	0	B	4	N/A	TA	V	Ramped unit to 70% power to perform 1-PT-29.1 (Turbine Valve Freedom Test).

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source.

UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: March 1991

Docket No.: 50-281
 Unit Name: Surry Unit 2
 Date: 04-03-91
 Completed by: M. A. Negron
 Telephone: 804-365-2795

(1) Date	(1) Type	(2) Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	LER No.	(4) System Code	(5) Component Code	Cause & Corrective Action to Prevent Recurrence
03-30-91	S	45.2	C	1	N/A	N/A	N/A	Shut Unit down in accordance with applicable operating procedures for a scheduled refueling outage. Duration expected to be 67 days.

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-280
Unit Name: Surry Unit 1
Date: 04-03-91
Completed by: M. A. Negron
Telephone: 804-365-2795

Month: March 1991

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	782	17	783
2	784	18	720
3	785	19	679
4	785	20	681
5	786	21	725
6	786	22	779
7	720	23	784
8	788	24	785
9	790	25	784
10	790	26	777
11	788	27	783
12	785	28	783
13	783	29	783
14	783	30	786
15	783	31	785
16	783		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-281
Unit Name: Surry Unit 2
Date: 04-03-91
Completed by: M. A. Negron
Telephone: 804-365-2795

Month: March 1991

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	700	17	629
2	700	18	623
3	698	19	617
4	691	20	610
5	686	21	605
6	679	22	600
7	673	23	593
8	670	24	588
9	666	25	580
10	662	26	575
11	657	27	569
12	653	28	564
13	645	29	550
14	643	30	19
15	639	31	0.0
16	634		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: March 1991

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

03-01-91	0000	This reporting period started with the Unit operating at 100% power and 825 MWe.
03-07-91	0958	Started ramp down to perform 1-PT-29.1; 100% power, 825 MWe.
	1146	Stopped ramp; 70% power, 608 MWe.
	1812	Started ramp up after completion of 1-PT-29.1; 70% power, 608 MWe.
	2100	Stopped ramp; 100% power, 825 MWe.
03-18-91	0846	Started ramp down to remove the first point feedwater heaters from service in order to repair a steam leak; 100% power, 825 MWe.
	0921	Stopped ramp; 90% power, 720 MWe.
03-21-91	1355	Started ramp up after return to service of the first point feedwater heaters; 89% power, 740 MWe.
	1504	Stopped ramp; 100% power, 820 MWe.
03-31-91	2400	This report period ended with the Unit operating at 100% power and 825 MWe.

UNIT TWO

03-01-91	0000	This report period started with the Unit operating at 90% power and 740 MWe.
03-03-91	0717	Commence end of cycle unit coastdown.
03-29-91	2200	Started unit shutdown for the refueling outage; 73% power, 595 MWe.
03-30-91	0249	Unit offline.
	0344	Reactor manually tripped.
03-31-91	2400	This report period ended with the Unit at Cold Shutdown and preparing for Refueling Shutdown.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: March 1991

EWB 89-703 **Engineering Work Request** 03-13-91
(Safety Evaluation No. 90-002)

This Engineering Work Request replaces the existing Fischer Porter transmitter, 1-CC-PT-100, with an equivalent Rosemount model.

Replacement of the Fischer Porter Transmitter is necessary because replacement parts are no longer easily obtained. The transmitter is not safety related and performs no safe shutdown function. The replacement transmitter is seismically qualified, meets nuclear and other design requirements and does not perform a safety related function. Therefore, an unreviewed safety question is not created.

AC S2-91-0304 **Administrative Control** 03-05-91
(Safety Evaluation No. 91-041)

Upon its removal, Administrative Control will be established over the "A" Recirculation Spray (RS)/Service Water (SW) Valve Pit missile shield to allow an inspection of the valve pit sump level probe.

This inspection is being performed as part of an investigation as to the cause of the valve pit Hi Level alarm actuation. The missile shield is a passive component required to protect RS/SW piping during adverse weather conditions. Only blank flanged RS/SW piping exists in the "A" valve pit and personnel and equipment will be available to replace the shield if adverse weather conditions approach the plant. Therefore, an unreviewed safety question is not created.

TSR 91-013 **Temporary Shielding Request** 03-05-91
(Safety Evaluation No. 91-044)

This Temporary Shielding Request installs lead shielding around lines (4"-CH-112-152, 6"-CH-18-152, 8"-CH-17-152) in the unit one "A" and "B" charging pump cubicles.

The temporary shielding will lower area radiation levels and reduce the dose received by personnel while painting the charging pumps cubicles. The subject lines will remain operable while the shielding is installed and Engineering calculations have determined that additional seismic loading is acceptable. Therefore, an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: March 1991

[continued]

JCO 91-3-001 **Justification for Continued Operation** 03-07-91
(Safety Evaluation No. 91-048)

This Justification for Continued Operation opens power supply breakers and de-energizes, in the open position, Containment Spray (CS) pump suction valves from the Chemical Addition Tank, 1-CS-MOV-103A, B, C, D and 2-CS-MOV-203A, B, C, D.

This change will ensure these valves remain open following initiation of a Hi Hi Consequence-Limiting Safeguard (CLS) and do not unnecessarily close due to the erroneous indication of low pressure on the discharge of the CS pumps. In addition, the function of these valves on a low pressure CS pump condition will be eliminated since it is no longer required. Maintaining the valves open during a Hi Hi CLS and low pressure condition on the CS pump will have no impact on CS system functions and will prevent spurious closures. Therefore, an unreviewed safety question is not created.

TM S1-91-006 **Temporary Modification** 03-08-91
(Safety Evaluation No. 91-049)

This Temporary Modification (TM) bypasses the relays in the remote start circuitry of the ESW pumps to provide direct operator control of starter engagement.

This TM will improve start reliability by bypassing the crank-rest circuitry installed for remote start purposes and will be effective until a permanent change is implemented. Since the diesels are now local-only start, the remote start circuitry is no longer required and can be bypassed. The design basis function of the ESW pumps is not affected and the probability of start failure is reduced since the operators directly control starter engagement. Therefore, an unreviewed safety question is not created.

AC S1-91-0314 **Administrative Control** 03-14-91
(Safety Evaluation No. 91-055)

Administrative Control (AC) has been established to permit the installation of a jacking device in the Unit 1 "C" feedwater regulation valve to permit I&C technicians to troubleshoot flow fluctuations.

This AC locates an operator at the valve to manually remove the jacking device and isolate feedwater if directed by the control room to ensure the intended safety function is maintained. Therefore, an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: March 1991

[continued]

AC S1-91-0314A **Administrative Control** 03-15-91
(Safety Evaluation No. 91-057)

Administrative Control (AC) has been established to permit manual starting of the Independent Spent Fuel Storage Installation (ISFSI) diesel during a loss of normal electrical power.

The ISFSI diesel supplies power to ISFSI pressure and temperature monitoring instrumentation during a loss of normal power. This AC is necessary due to a faulty start coil on the fuel oil solenoid. When a replacement solenoid becomes available it will be installed and the AC will be terminated. A temporary loss of ISFSI monitoring, until a manual start can be initiated, would not increase the probability or consequence of a cask failure. Therefore, an unreviewed safety question is not created.

CAL SM-787 **Calculation** 03-22-91
(Safety Evaluation No. 91-062A)

Calculation SM-787 examines the pressure response with time of both the Main Steam System and the Reactor Coolant System under a Loss of Electrical Load, assuming that one of the 15 main steam safety valves fails to open.

The excess capacity of the Main Steam Safety Valves at 110% system design pressure (1210 psia), compared to the required capacity at the peak Main Steam System pressure (1179 psia), is greater than the capacity of any one valve. Since additional safety valves are available to provide relief capability prior to reaching the maximum analyzed pressure, the Main Steam System will be able to perform its design function with one valve inoperable on each of the three steam lines. Therefore, an unreviewed safety question does not exist.

TSR 91-023 **Temporary Shielding Request** 03-26-91
(Safety Evaluation No. 91-066)

This Temporary Shielding Request installs lead shielding around lines, 2"-RL-103-152, 2"-DA-134-152, and valve 2-RL-12 in the Unit 2 Containment basement.

The temporary shielding will lower area radiation levels and reduce the dose received for personnel while work in this area is performed. The reactor cavity drain lines will remain operable while the shielding is installed and Engineering calculations have determined that additional seismic loading is acceptable. Therefore, an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: March 1991

[continued]

AC S1-91-0321 **Administrative Control** 03-28-91
(Safety Evaluation No. 91-070)

Administrative Control (AC) will be established over outside instrument air supply isolation valves, 2-IA-446 and 2-IA-447, to Unit 1 Containment from Unit 2 instrument air to ensure containment integrity can be maintained with the valves open.

Personnel will be continuously stationed in the vicinity of the valves and will close them upon notification from the Control Room. This AC will ensure the ability to maintain containment partial pressure within the limits established by Technical Specifications. Therefore, an unreviewed safety question is not created.

AC S1-91-0320 **Administrative Control** 03-28-91
(Safety Evaluation No. 91-071)

Administrative Control will be established upon the removal of the flood dike at the Emergency Switchgear Room (ESGR) door.

The dike will be temporarily removed to facilitate moving heavy equipment into and out of the ESGR. A flood watch will be posted at the door to reinstall the dike, and a roving flood watch will be in the Turbine building while the dike is removed. The removal of the dike will not affect safety related equipment and the administrative controls provide adequate compensatory actions to combat potential flooding. Therefore, an unreviewed safety question is not created.

AC S1-91-0320A **Administrative Control** 03-28-91
(Safety Evaluation No. 91-072)

Administrative Control will be established upon the removal of the flood dike at the entrance to Mechanical Equipment Room (MER) 3.

The dike will be temporarily removed to facilitate moving heavy equipment into or out of MER 3. A flood watch will be posted at the door to reinstall the dike, and another flood watch will be posted in the MER while the dike is removed. The removal of the dike will not affect safety related equipment and the administrative controls provide adequate compensatory actions to combat potential flooding. Therefore an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: March 1991

[continued]

TSR 91-028
TSR 91-038
TSR 91-039**Temporary Shielding Request**
(Safety Evaluation No. 91-074)

03-28-91

These Temporary Shielding Requests install lead shielding on the "A", "B", and "C" Loop Resistance Temperature Detector (RTD) Manifold Lines in the Reactor Coolant Pump cubicles.

This temporary shielding will lower area radiation levels and reduce the dose received for personnel while work is being performed. The RTD lines will be inoperable while lead shielding is installed and the unit will be in cold shutdown. A deadweight analysis has been performed for the additional loading on the RTD manifolds and found to be acceptable. There is a small possibility of the lines rupturing in a seismic event, however, this break is enveloped by the LOCA accidents previously evaluated. Therefore, an unreviewed safety question is not created.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: March 1991

2-OPT-ZZ-001

Periodic Test Procedure
(Safety Evaluation No. 91-064)

3-21-91

A new periodic test, 2-OPT-ZZ-001, was developed to comply with the Technical Specifications refueling test requirements for Engineered Safety Feature (ESF) actuation with delayed undervoltage on the "2H" emergency bus.

The unit will be at cold shutdown conditions when ESF functions are not required. Two cooling loops, consisting of the Residual Heat Removal pump not being tested and a Reactor Coolant Pump loop, will be available to provide core decay heat removal. No permanent changes to systems are required. Electrical jumpers and lifted leads required for the testing will be controlled with Temporary Modifications (TM) requiring double verification for installation and removal. Therefore, an unreviewed safety question does not exist.

2-OPT-ZZ-002

Periodic Test Procedure
(Safety Evaluation No. 91-063)

3-25-91

A new periodic test, 2-OPT-ZZ-002, was developed to comply with the Technical Specifications refueling test requirements for Engineered Safety Feature (ESF) actuation with delayed undervoltage on the "2J" emergency bus.

The unit will be at cold shutdown conditions when ESF functions are not required. Two cooling loops, consisting of the Residual Heat Removal pump not being tested and a Reactor Coolant Pump loop, will be available to provide core decay heat removal. No permanent changes to systems are required. Electrical jumpers and lifted leads required for the testing will be controlled with Temporary Modifications (TM) requiring double verification for installation and removal. Therefore, an unreviewed safety question does not exist.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: March 1991

None During This Reporting Period.

CHEMISTRY REPORT

MONTH/YEAR: March 1991

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioact., $\mu\text{Ci/ml}$	7.15E-1	3.42E-1	5.06E-1	2.59E-1	9.85E-3	1.26E-1
Suspended Solids, ppm	0.0	0.0	0.0	0.0	0.0	0.0
Gross Tritium, $\mu\text{Ci/ml}$	3.84E-1	3.71E-1	3.79E-1	1.67E-1	1.21E-1	1.43E-1
I^{131} , $\mu\text{Ci/ml}$	3.37E-3	1.41E-3	1.87E-3	9.31E-4	3.37E-4	5.52E-4
$\text{I}^{131}/\text{I}^{133}$	0.17	0.07	0.10	0.15	0.06	0.10
Hydrogen, cc/kg	48.5	32.4	37.7	39.7	3.5	26.8
Lithium, ppm	2.44	2.05	2.18	0.96	0.28	0.73
Boron - 10, ppm*	157.2	141.9	149.5	321	0.20	39.8
Oxygen, (DO), ppm	≤ 0.005	≤ 0.005	≤ 0.005	1.0	≤ 0.005	0.067
Chloride, ppm	0.007	≤ 0.001	0.003	0.007	≤ 0.001	0.002
pH at 25 degree Celsius	6.67	6.52	6.60	9.29	5.36	7.95

* Boron - 10 = Total Boron x 0.196

Comments:

Unit 1 - Lithium out-of-specification high for 5 hours on 3/22/91.

**FUEL HANDLING
 UNITS 1 & 2**

MONTH/YEAR: March 1991

Units One and Two

Cask	Stored	Number for Assemblies per Shipment	Assembly Number	ANSI Number	Nominal Initial Enrichment	New or Spent Fuel Shipping Cask Activity
NAC-I28.1	03-26-91	N/A	A01	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A02	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A03	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A06	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A07	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A10	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A11	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A15	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A16	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A18	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A23	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A27	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A28	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A30	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A31	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A34	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A36	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A37	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A39	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A41	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A42	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A43	N/A	1.87	N/A

[continued]

FUEL HANDLING

MONTH/YEAR: March 1991

Units One and Two

Cask	Stored	Number for Assemblies per Shipment	Assembly Number	ANSI Number	Nominal Initial Enrichment	New or Spent Fuel Shipping Cask Activity
NAC-I28.1	03-26-91	N/A	A44	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A45	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A47	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A48	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A51	N/A	1.87	N/A
NAC-I28.1	03-26-91	N/A	A53	N/A	1.87	N/A

**DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS**

MONTH/YEAR: March 1991

<u>Number</u>	<u>Title</u>	<u>LER No.</u>
PT-53.3	High Energy Line Inspection	S1-91-001-00
2-PT-18.6I	Pressurizer Block Valve Stroke Test	S2-91-001-00