3. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70, and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at steady state power levels not in excess of 2441 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

E. The analysis for rupture of a main steam pipe may be revised in accordance with the licensee's submittal dated March 6, 1990 (Serial No. 90-094).

F. Deleted by Amendment 66 and again by Amendment 71

G. Steam Generator Repair Program

(1) The Surry Power Steam Generator Repair Program for Unit No. 1 is approved.

(2) During the steam generator repair program the following conditions shall be met:

(a) All fuel shall be removed from the reactor pressure vessel and stored in the spent fuel pool.

(b) Temporary containment and ventilation systems shall be installed and operated for all cutting and grinding operations involving components with removable radioactive contamination greater than 2200 DPM per 100 cm^2 except

3. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70, and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at steady state power levels not in excess of 2441 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

E. The analysis for rupture of a main steam pipe may be revised in accordance with the licensee's submittal dated March 6, 1990 (Serial No. 90-094).

F. Deleted by Amendment 59 and again by Amendment 65

G. Steam Generator Repair Program

(1) The Surry Power Steam Generator Repair Program for Unit No. 2 is approved.

(2) During the steam generator repair program the following conditions shall be met:

(a) All fuel shall be removed from the reactor pressure vessel and stored in the spent fuel pool.

(b) Temporary containment and ventilation systems shall be installed and operated for all cutting and grinding operations involving components with removable radioactive contamination greater than 2200 DPM per 100 cm^2 except

ATTACHMENT 3

Significant Hazards Determination

Significant Hazards Determination

The proposed change to revise the rupture of a main steam pipe analysis by eliminating the low-low pressurizer pressure safety injection initiating function does not result in a significant hazards consideration per 10 CFR 50.92.

1. The proposed change does not significantly increase the probability or consequences of an accident previously evaluated. The elimination of the low-low pressurizer pressure safety injection initiating function is only for purposes of analysis, specifically in regard to the rupture of a main steam pipe. No physical changes or modifications are being made to the plant or its equipment. Thus, there is no change in the probability of occurrence of any accident, including rupture of a steam pipe.

   The current analysis for rupture of a steam pipe assumes that low-low pressurizer pressure safety injection is available. For certain small steam line break scenarios, safety injection is initiated based on pressurizer pressure. If the assumed pressurizer pressure safety injection is not available, the consequences of a small steam line break would require reanalysis.

   An analysis was performed to evaluate small steam line break consequences, without the benefit of low pressurizer pressure safety injection. Our analysis concludes that the applicable accident analysis acceptance criteria are met for the entire spectrum of steam line break sizes, without relying on low-low pressurizer pressure safety injection. The large (hypothetical) steam line break remains the limiting steam line rupture case. Accordingly, we conclude that the consequences of a main steam pipe rupture are not increased and that low pressurizer pressure safety injection is not necessary for a rupture of a main steam pipe. Because the low pressurizer pressure safety injection is not necessary to mitigate a rupture of a main steam pipe, there is no increase in the consequences of a malfunction of the associated equipment.

2. The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated. As noted above, no physical changes or modifications are being made to the plant or its equipment. The only change is to eliminate the low pressurizer pressure safety injection initiating function for the rupture of a main steam pipe analysis.

3. The proposed amendment does not involve a significant reduction in a margin of safety. As previously stated, our analysis shows that the applicable accident analysis criteria are met for the entire spectrum of steam line break sizes. In addition, the large (hypothetical) stream line break remains the limiting steam line rupture case.