

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

February 14, 1990

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D. C. 20555

Serial No. 90-079  
NO/RPC:vlh  
Docket Nos. 50-280  
50-281  
License Nos. DPR-32  
DPR-37

Gentlemen:

**VIRGINIA ELECTRIC AND POWER COMPANY**  
**SURRY POWER STATION UNITS 1 AND 2**  
**MONTHLY OPERATING REPORT**

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of January 1990.

Very truly yours,



W. L. Stewart  
Senior Vice President - Nuclear

Enclosure

cc: U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, N. W.  
Suite 2900  
Atlanta, Georgia 30323

Mr. W. E. Holland  
NRC Senior Resident Inspector  
Surry Power Station

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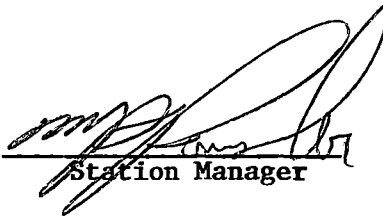
VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION

MONTHLY OPERATING REPORT

REPORT # 90-01

APPROVED:

  
Station Manager

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OPERATING DATA REPORT

DOCKET NO.: 50-280  
 DATE: 02/07/90  
 COMPLETED BY: L.A. Warren  
 TELEPHONE: (804)357-3184 x355

OPERATING STATUS

NOTES

1. Unit Name: Surry Unit 1
2. Reporting Period: JAN 01-31, 1990
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 820
7. Maximum Dependable Capacity (Net MWe): 781
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: \_\_\_\_\_

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reason For Restrictions, If Any: \_\_\_\_\_

	<u>THIS MONTH</u>	<u>YTD</u>	<u>CUMULATIVE</u>
11. Hours In Reporting Period	744.0	744.0	150000.0
12. Number of Hours Reactor Was Critical	744.0	744.0	93494.8
13. Reactor Reserve Shutdown Hours	0	0	3774.5
14. Hours Generator On-Line	744.0	744.0	91567.2
15. Unit Reserve Shutdown Hours	0	0	3736.2
16. Gross Thermal Energy Generated (MWH)	1789748.0	1789748.0	212906551.0
17. Gross Electrical Energy Generated (MWH)	606545.0	606545.0	69151948.0
18. Net Electrical Energy Generated (MWH)	578112.0	578112.0	65589042.0
19. Unit Service Factor	100%	100%	61%
20. Unit Availability Factor	100%	100%	63.5%
21. Unit Capacity Factor (Using MDC Net)	99.5%	99.5%	56.5%
22. Unit Capacity Factor (Using DER Net)	98.6%	98.6%	55.5%
23. Unit Forced Outage Rate	0%	0	21.5
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down at End of Report Period Estimated Date of Startup: \_\_\_\_\_
26. Unit In Test Status (Prior to Commercial Operation): FORECAST ACHIEVED

INITIAL CRITICALITY \_\_\_\_\_  
 INITIAL ELECTRICITY \_\_\_\_\_  
 COMMERCIAL OPERATION \_\_\_\_\_

OPERATING DATA REPORT

DOCKET NO.: 50-281  
 DATE: 02/07/90  
 COMPLETED BY: L.A. Warren  
 TELEPHONE: (804)357-3184 x355

OPERATING STATUS

NOTES

1. Unit Name: Surry Unit 2
2. Reporting Period: JAN 01-31, 1990
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 820
7. Maximum Dependable Capacity (Net MWe): 781
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: \_\_\_\_\_

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reason For Restrictions, If Any: \_\_\_\_\_

	<u>THIS MONTH</u>	<u>YTD</u>	<u>CUMULATIVE</u>
11. Hours In Reporting Period	744.0	744.0	146880.0
12. Number of Hours Reactor Was Critical	744.0	744.0	91942.6
13. Reactor Reserve Shutdown Hours	0	0	328.1
14. Hours Generator On-Line	744.0	744.0	90392.9
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1694307.6	1694307.6	211304642.4
17. Gross Electrical Energy Generated (MWH)	572250.0	572250.0	68652849.0
18. Net Electrical Energy Generated (MWH)	544553.0	544553.0	65085512.0
19. Unit Service Factor	100%	100%	61.5%
20. Unit Availability Factor	100%	100%	61.5%
21. Unit Capacity Factor (Using MDC Net)	93.7%	93.7%	56.9%
22. Unit Capacity Factor (Using DER Net)	92.9%	92.9%	56.2%
23. Unit Forced Outage Rate	0%	0%	15.6%
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down at End of Report Period Estimated Date of Startup: \_\_\_\_\_
26. Unit In Test Status (Prior to Commercial Operation): FORECAST ACHIEVED

INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWN AND POWER REDUCTION (>20%)

REPORT MONTH: JANUARY 1990

DOCKET NO.: 50-280

UNIT NAME: Surry Unit-1

DATE: 02/07/90

COMPLETED BY: L.A. Warren

TELEPHONE: 804-357-3184 x355

NO.	DATE	TYPE(1)	DURATION (HOURS)	REASON(2)	METHOD OF SHUTTING DOWN REACTOR(3)	LICENSEE EVENT REPORT#	SYSTEM CODE(4)	COMPONENT CODE(5)	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
	01/07/90	S	0	B	4 (manual power reduction)		SD	V	Ramped down to repair leak on 1-CN-207. Valve is associated with 1-FW-P-1A. The main feed pump, 1-FW-P-1A, was secured to facilitate valve repair.
	01/14/90	F	0	A	4 (manual power reduction)		SD	P	Ramped down upon securing 1-SD-P-1B due to fire in seal area of pump.
	01/21/90	S	0	B	4 (manual power reduction)		EL	FAN	Ramped down to replace belt on 'C' phase bus duct cooling fan.

(1)	(2)	(3)	(4)
F: Forced S: Scheduled	REASON: A - Equipment Failure (Explain) B - Maintenance or Test C - Refueling D - Regulatory Restriction E - Operator Training & License Examination F - Administrative G - Operational Error (Explain) H - Other (Explain)	METHOD: 1 - Manual 2 - Manual Scram. 3 - Automatic Scram. 4 - Other (Explain)	Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)
			(5) Exhibit 1 - Same Source

UNIT SHUTDOWN AND POWER REDUCTION (>20%)

REPORT MONTH: JANUARY 1990

DOCKET NO.: 50-281

UNIT NAME: Surry Unit 2

DATE: 02/07/90

COMPLETED BY: L.A. Warren

TELEPHONE: 804-357-3184 x355

NO.	DATE	TYPE(1)	DURATION (HOURS)	REASON(2)	METHOD OF SHUTTING DOWN REACTOR(3)	LICENSEE EVENT REPORT#	SYSTEM CODE(4)	COMPONENT CODE(5)	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
	01/06/90	S	0	B	4 (manual power reduction)		SJ	P	Ramped down to remove 2-FW-P-1B from service for repairs.

(1)	(2)	(3)	(4)
F: Forced S: Scheduled	REASON: A - Equipment Failure (Explain) B - Maintenance or Test C - Refueling D - Regulatory Restriction E - Operator Training & License Examination F - Administrative G - Operational Error (Explain) H - Other (Explain)	METHOD: 1 - Manual 2 - Manual Scram. 3 - Automatic Scram. 4 - Other (Explain)	Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)
			(5) Exhibit 1 - Same Source

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.: 50-280  
UNIT NAME: Surry Unit 1  
DATE: 02/07/90  
COMPLETED BY: L.A. Warren  
TELEPHONE: (804)357-3184 x355

MONTH: JANUARY 1990

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>788</u>	17	<u>782</u>
2	<u>791</u>	18	<u>789</u>
3	<u>788</u>	19	<u>789</u>
4	<u>785</u>	20	<u>787</u>
5	<u>791</u>	21	<u>734</u>
6	<u>792</u>	22	<u>787</u>
7	<u>560</u>	23	<u>788</u>
8	<u>791</u>	24	<u>789</u>
9	<u>791</u>	25	<u>790</u>
10	<u>789</u>	26	<u>791</u>
11	<u>788</u>	27	<u>790</u>
12	<u>786</u>	28	<u>791</u>
13	<u>787</u>	29	<u>789</u>
14	<u>763</u>	30	<u>789</u>
15	<u>761</u>	31	<u>787</u>
16	<u>767</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.



AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.: 50-281  
UNIT NAME: Surry Unit 2  
DATE: 02/07/90  
COMPLETED BY: L.A. Warren  
TELEPHONE: (804)357-3184 x355

MONTH: JANUARY 1990

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>783</u>	17	<u>783</u>
2	<u>779</u>	18	<u>742</u>
3	<u>783</u>	19	<u>708</u>
4	<u>774</u>	20	<u>726</u>
5	<u>774</u>	21	<u>779</u>
6	<u>497</u>	22	<u>677</u>
7	<u>771</u>	23	<u>670</u>
8	<u>783</u>	24	<u>669</u>
9	<u>782</u>	25	<u>667</u>
10	<u>783</u>	26	<u>671</u>
11	<u>783</u>	27	<u>673</u>
12	<u>779</u>	28	<u>672</u>
13	<u>780</u>	29	<u>679</u>
14	<u>761</u>	30	<u>698</u>
15	<u>783</u>	31	<u>697</u>
16	<u>783</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: JANUARY 1990

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

01/01/90 0000 This reporting period began with the Unit at 100% power, 830 MW.

01/04/90 1141 Started rampdown for 1-ST-273; 2-FW-P-2 full flow test; 100% power, 830 MW.

1249 Stopped ramp, 95.5% power, 790 MW.

1618 Started ramp up, 1-ST-273 completed; 95% power, 785 MW.

1712 Stopped ramp; 100% power, 830 MW.

01/07/90 0402 Started ramp down to remove 1-FW-P-1A in order to repair 1-CN-207; 100% power, 830 MW.

0639 Stopped ramp; 60% power.

0700 Started ramp down to decrease flow requirements for 1-FW-P-1B; 60% power.

0730 Stopped ramp; 56% power, 445 MW.

1936 Started ramp up, 1-CN-207 repaired and 1-FW-P-1A returned to service; 56% power, 440 MW.

01/12/90 2025 Started ramp down in order to prepare for testing of Ch-2 over power/over temperature delta temperature (OP/OT DT) (Ch-3 alarming intermittently); 100%, 825 MW.

2037 Stopped ramp; 99% power, 820 MW.

01/13/90 0030 Ramped up after completion of test for Ch-2 OP/OT DT; 100% power, 825 MW.

01/14/90 2011 Started ramp down; fire occurred in the lower seal area of 1-SD-P-1B and as a result the pump was secured; 100% power, 830 MW.

2018 Plant stable at 96% power.

2039 Started ramp down to increase main feed pump suction pressure; 96% power.

2108 Stopped ramp; 90% power 700 MW.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: JANUARY 1990

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

01/14/90 2125 Started ramp down to increase main feed pump suction pressure to 425 psig for reset of pressure switch; 90% power, 700 MW.

2140 Stopped ramp; 87% power, 670 MW.

2150 Started ramp down to maintain main feed pump suction pressure while closing the condensate polisher bypass valve (1-CP-AOV-122).

2225 Stopped ramp; 80% power, 620 MW.

01/15/90 0302 Started ramp up; 1-SD-P-1A placed in service; 80% power, 650 MW.

0421 Stopped ramp; 100% power, 825 MW.

01/16/90 1156 Started ramp down to prepare for 1-PT-29.1.

1252 Stopped ramp, 90% power, 745 MW.

1817 Started ramp up; 1-PT-29.1 completed; 90% power, 745 MW.

1901 Stopped ramp; 100% power, 825 MW.

01/21/90 0830 Started ramp down to allow belt replacement on 'C' isolated phase bus duct cooling fan; 100% power, 830 MW.

1018 Stopped ramp; 62% power, 500 MW.

1217 Started ramp up; belt on 'C' cooling fan replaced and fan returned to service; 63% power, 480 MW.

1454 Stopped ramp; 100% power, 830 MW.

01/31/90 1108 Started ramp down to prepare for turbine driven auxiliary feed pump, 1-FW-P-2, operation; 100% power 825 MW.

1120 Stopped ramp; 98% power, 800 MW.

1315 Started ramp up; 1-FW-P2 secured; 98% power, 800 MW.

1330 Stopped ramp; 100% power, 830 MW.

01/31/90 2400 This reporting period ended with the Unit at 100% power, 830 MW.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: JANUARY 1990

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT TWO

01/01/90 0000 This reporting period began with the Unit at 100% power, 820 MW.

01/04/90 0950 Started ramp down to ensure <7% difference exists between 'A' loop over temperature delta temperature (OTDT) setpoint (STPT) and actual delta temperature (DT); 100% power, 825 MW.

1005 Stopped ramp; 98% power, 815 MW.

01/05/90 1959 Started ramp up; adequate margin exists between OTDT STPT. and actual DT for 'A' loop; 98% power, 810 MW.

2012 Stopped ramp; 100% power, 820 MW.

01/06/90 0104 Started ramp down to remove 2-FW-P-1B from service for repairs; 100% power, 820 MW.

0344 Stopped ramp; 60% power, 510 MW.

2230 Started ramp up; 2-FW-P-1B returned to service, waterboxes cleaned; 60% power, 500 MW.

01/07/90 0130 Stopped ramp; 99.5% power, 825 MW. Started ramp down because of receipt of 'B' loop Hi DT (protection) computer alarm; 99.5% power, 825 MW.

0140 Stopped ramp; 98.3% power, 815 MW.

1409 Started ramp up; 2-PT-2.1B completed on OTDT; 98.3% power, 810 MW.

1422 Stopped ramp; 100% power, 820 MW.

01/08/90 2230 Started ramp down after placing Ch-1 OP/OT DT in trip. Channel placed in trip after being declared inoperable from slowly increasing TAVE indication; Ch-2 reading slightly high; 100% power, 820 MW. Stopped ramp; 99% power, 820 MW.

01/09/90 0533 Started slow power increase to 100%; Ch-1 OP/OT DT returned to service; 99% power, 820 MW.

01/09/90 0620 Stopped power increase; 100% power, 820 MW.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: JANUARY 1990

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT TWO

01/12/90 1657 Started ramp down in preparation for auxiliary feed flow test, 2-ST-273; 100% power, 820 MW.  
1723 Stopped ramp; 95% power, 790 MW.  
2009 Started ramp up after flow test completed; 95% power, 790 MW.  
2030 Stopped ramp; 100% power, 820 MW.  
01/13/90 2037 Started ramp down for implementation of EWR-90-021, DT setpoints; 100% power, 820 MW.  
2111 Stopped ramp; 95% power, 780 MW.  
01/14/90 1447 Started ramp up after EWR-90-021 completed; 95% power, 785 MW.  
1519 Stopped ramp; 100% power, 820 MW.  
01/18/90 1016 Started ramp down to prepare for 2-PT-29.1; 100% power, 820 MW.  
1102 Stopped ramp; 90% power, 755 MW. 2-PT-29.1 test of #2 LP intercept and reheat valves caused 2-MS-SV-206A to lift. As a result of the lift a crack developed in an associated 2" drain line.  
01/20/90 1757 Started ramp up to complete checks of crossunder safties; 91% power, 750 MW.  
1855 Stopped ramp; 100% power, 825 MW.  
01/21/90 2025 Started ramp down due to results of analysis on crossunder safety valves; all operable except 2-MS-SV-207A; 100% power, 820 MW.  
2040 Stopped ramp; 97% power, 800 MW.  
01/22/90 0132 Started ramp down; initial calculations on crossunder safety indicate relieving capability of operable safety is approximately 86% of 100% steam flow. 97% power, 800 MW.  
0228 Stopped ramp; 85% power, 710 MW.  
01/29/90 1722 Started ramp after determining satisfactory calibration on 2-MS-SV-206B; 85% power, 715 MW.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: JANUARY 1990

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT TWO

01/31/90 2400      This reporting period ended with the Unit at 89% power, 735 MW.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: JANUARY 1990

EWR-90-003      ENGINEERING WORK REQUEST UNITS 1&2      01/13/90  
(Safety Evaluation #90-0004)

This Engineering Work Request installed permanent audio radiological postings at the entrance to the auxiliary building basement.

This does not constitute an unreviewed safety question in that this is a health physics posting device which is non-safety related and does not affect any safety related systems. This change does not increase the probability or consequences of any accidents as analyzed in the UFSAR, nor does it change the basis for any technical specifications.

EWR-89-716      ENGINEERING WORK REQUEST UNITS 1&2      01/18/90  
(Safety Evaluation #N89-0042)

This Engineering Work Request added filters to the reactor coolant system temperature channels, resulting in an additional time delay in both delta-T and TAVG signals.

The additional time delay does not exceed the allowable limits of the existing safety analysis. Therefore, an unreviewed safety question does not exist.

S1-AC-90-0119 ADMINISTRATIVE CONTROL      01/19/90  
(Safety Evaluation #90-0008)

Administrative control will be established over 1-SW-124, the manual isolation to TCV-SW-108C, while the normal temperature control valve is under repair.

By maintaining control of the manual valve the lube oil temperature can be maintained within required ranges and allow the charging pump to operate per design. Therefore an unreviewed safety question is not created.

S1-AC-90-0122 ADMINISTRATIVE CONTROL      01/22/90  
(Safety Evaluation #90-0009)

This administrative control documents that the dike at the emergency switchgear room door will temporarily be removed, a flood watch will be placed at the door and a roving flood watch will walk down the turbine building while the dike is down. Administrative measures are provided for reinstallation of the dikes if flooding is observed.

The removal of this dike will not adversely affect safety related equipment or create an unanalyzed condition since the administrative measures compensate.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: JANUARY 1990

SCAFFOLD REQUEST UNIT 1 01/22/90  
(Safety Evaluation #90-0010)

Temporary scaffold was erected to support the maintenance on damper/actuator mark #01-VSP-AOD-111A for the containment purge exhaust.

The temporary scaffold is required for safe working conditions. Installation of scaffold constructed per SUADM-ADM-07 has a high confidence level against failure and was reviewed for effects on accident analyses and equipment operability/function. It is thus concluded that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLD REQUEST UNIT 1 01/22/90  
(Safety Evaluation #90-0011)

Temporary scaffold was erected to support the maintenance work on damper/actuator for fuel building filter exhaust damper.

The temporary scaffold is required for safe working conditions. Installation of scaffold constructed per SUADM-ADM-07 has a high confidence level against failure and was reviewed for effects on accident analyses and equipment operability/function. It is thus concluded that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLD REQUEST UNIT 1 01/22/90  
(Safety Evaluation #90-0012)

Temporary scaffold was erected to support the maintenance work on damper/actuator mark # 01-VSP-AOD-107A for the auxiliary building.

The temporary scaffold is required for safe working conditions. Installation of scaffold constructed per SUADM-ADM-07 has a high confidence level against failure and was reviewed for effects on accident analyses and equipment operability/function. It is thus concluded that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLD REQUEST UNIT 1 01/22/90  
(Safety Evaluation #90-0013)

Temporary scaffold was erected to support the maintenance work on damper/actuator mark #01-VSP-AOD-109A for general area exhaust fan discharge.



FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: JANUARY 1990

The temporary scaffold is required for safe working conditions. Installation of scaffold constructed per SUADM-ADM-07 has a high confidence level against failure and was reviewed for effects on accident analyses and equipment operability/function. It is thus concluded that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLD REQUEST UNIT 1  
(Safety Evaluation #90-0015)

01/25/90

Temporary scaffold was erected to support the maintenance work on an auxiliary steam line which has a leak and is causing airborne contamination.

The temporary scaffold is required for safe working conditions. Installation of scaffold constructed per SUADM-ADM-07 has a high confidence level against failure and was reviewed for effects on accident analyses and equipment operability/function. It is thus concluded that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

S1-AC-89-1227 ADMINISTRATIVE CONTROL UNIT 1  
(Safety Evaluation #90-0016)

01/25/90

Administrative control will be placed on 1-IA-446 and 1-IA-447 to ensure that containment integrity can be maintained.

The administrative control established on 1-IA-446 and 1-IA-447 in accordance with SUADM-O-26 ensures the ability to maintain containment integrity as defined in technical specifications. The valves will be closed immediately upon notification by the main control room. In addition, control over the valves will ensure the ability to maintain the containment partial pressure within the limits established by Technical Specification 3.8. Therefore, an unreviewed safety question is not created.

S1-AC-89-1228 ADMINISTRATIVE CONTROL UNIT 1  
(Safety Evaluation #90-0017)

01/25/90

Administrative control will be placed on 2-IA-446 and 2-IA-447 to ensure that containment integrity can be maintained.

The administrative control established on 2-IA-446 and 2-IA-447 in accordance with SUADM-O-26 ensures the ability to maintain containment integrity as defined in technical specifications. The valves will be closed immediately upon notification by the main control room. In addition, control over the valves will ensure the ability to maintain the containment partial pressure within the limits established by Technical Specification 3.8. Therefore, an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: JANUARY 1990

SCAFFOLD REQUEST UNIT 1 01/30/90  
(Safety Evaluation #90-0022) Temporary scaffold was erected to support the maintenance work on damper/actuator mark #01-VSP-AOD-103A for decon building filter exhaust damper.

The temporary scaffold is required for safe working conditions. Installation of scaffold constructed per SUADM-ADM-07 has a high confidence level against failure and was reviewed for effects on accident analyses and equipment operability/function. It is thus concluded that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

EWR-89-681 ENGINEERING WORK REQUEST UNITS 1 & 2 01/31/90  
Change request to UFSAR Sec. 9.10.4.19  
(Safety Evaluation #N89-0025)

This EWR evaluated permanently leaving the Emergency Diesel Generator (EDG) rooms air supply louvers open and making a change to the UFSAR. This change removes the temporary modifications which covered keeping these louvers open and makes permanent design changes disconnecting the actuation mechanism.

The loss of the EDG from overheating was considered and since the air louvers will be maintained opened, the EDG will continue to operate as designed. Therefore, an unreviewed safety question does not exist.

PROCEDURE OR METHOD OF OPERATION CHANGES  
THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: JANUARY 1990

NONE DURING THIS REPORTING PERIOD

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: JANUARY 1990

NONE DURING THIS REPORTING PERIOD

VIRGINIA POWER  
SURRY POWER STATION  
CHEMISTRY REPORT

MONTH/YEAR: JANUARY 1990

PRIMARY COOLANT ANALYSIS	UNIT NO. 1			UNIT NO. 2		
	MAX.	MIN.	AVG.	MAX.	MIN.	AVG.
Gross Radioact., $\mu\text{Ci/ml}$	8.53E-1	6.61E-2	7.34E-1	2.71E-1	167E-1	2.13E-1
Suspended Solids, ppm	0.0	0.0	0.0	0.0	0.0	0.0
Gross Tritium, $\mu\text{Ci/ml}$	1.89E-1	1.51E-1	1.66E-1	3.99E-1	2.88E-1	3.36E-1
Iodine-131, $\mu\text{Ci/ml}$	1.44E-2	3.02E-3	6.33E-3	1.03E-3	3.11E-4	5.48E-4
Iodine-131/Iodine-133	0.20	0.09	0.13	0.25	0.08	0.12
Hydrogen, cc/kg	31.5	25.9	28.4	34.5	27.5	31.2
Lithium, ppm	2.34	2.06	2.19	2.33	2.11	2.22
Boron - 10, ppm*	107	85	93	191	176	180
Oxygen, (DO), ppm	0.005	0.005	0.005	0.005	0.005	0.005
Chloride, ppm	0.010	0.006	0.007	0.010	0.006	0.007
pH @ 25 degree Celsius	7.05	6.77	6.90	6.67	6.40	6.53

\* Boron - 10 = Total Boron x 0.196

REMARKS:

UNIT ONE: Excess letdown was placed in service on 1/10 at 0928 hrs. to perform work on normal letdown; normal letdown was returned to service on 1/11 at 1059 hrs. The following lithium additions were made: 1/7, 250 gm; 1/8, 420 gm; 1/10, 410 gm; 1/15, 450 gm; 1/16, 400 gm; 1/19, 480 gm; 1/21, 489 gm; 1/27, 437 gm. Total addition was 3317 gms.

UNIT TWO: Cation bed was placed in service on 1/16 from 0540 hrs. to 0710 hrs. No lithium additions were made during January.

NEW OR SPENT FUEL SHIPMENT #	DATE SHIPPED OR RECEIVED	NUMBER OF ASSEMBLIES PER SHIPMENT	ASSEMBLY NUMBER	ANSI NUMBER	INITIAL ENRICHMENT	NEW OR SPENT FUEL SHIPPING CASK ACTIVITY LEVEL
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NONE DURING THIS REPORTING PERIOD

DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED WITHIN  
THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

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