

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

December 15, 1989

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

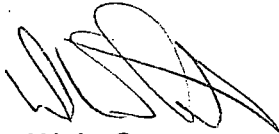
Serial No. 89-851
NO/RPC:vlh
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of November 1989.

Very truly yours,



W. L. Stewart
Senior Vice President - Nuclear

Enclosure

cc: U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, N. W.
Suite 2900
Atlanta, Georgia 30323

Mr. W. E. Holland
NRC Senior Resident Inspector
Surry Power Station

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VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION

MONTHLY OPERATING REPORT

REPORT # 89-11

APPROVED:


Station Manager

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OPERATING DATA REPORT

DOCKET NO.: 50-280
 DATE: 12/05/89
 COMPLETED BY: L.A. Warren
 TELEPHONE: (804)357-3184 x355

OPERATING STATUS

NOTES

1. Unit Name: Surry Unit 1
2. Reporting Period: NOV 01-30, 1989
3. Licensed Thermal Power (Mwt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 820
7. Maximum Dependable Capacity (Net MWe): 781
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reason For Restrictions, If Any: _____

	<u>THIS MONTH</u>	<u>YTD</u>	<u>CUMULATIVE</u>
11. Hours In Reporting Period	720.0	8016.0	148512.0
12. Number of Hours Reactor Was Critical	720.0	3553.0	92031.6
13. Reactor Reserve Shutdown Hours	0	0	3774.5
14. Hours Generator On-Line	720.0	3501.7	90107.1
15. Unit Reserve Shutdown Hours	0	0	3736.2
16. Gross Thermal Energy Generated (MWH)	1730947.0	8212036.0	209383303.0
17. Gross Electrical Energy Generated (MWH)	588890.0	2754080.0	67957753.0
18. Net Electrical Energy Generated (MWH)	560503.0	2610344.0	64450747.0
19. Unit Service Factor	100%	43.7%	60.7%
20. Unit Available Factor	100%	43.7%	63.2%
21. Unit Capacity Factor (Using MDC Net)	99.7%	41.7%	56.1%
22. Unit Capacity Factor (Using DER Net)	98.8%	41.3%	55.1%
23. Unit Forced Outage Rate	0	56.3%	21.7%
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down at End of Report Period Estimated Date of Startup: _____
26. Unit In Test Status (Prior to Commercial Operation): FORECAST ACHIEVED

INITIAL CRITICALITY _____
 INITIAL ELECTRICITY _____
 COMMERCIAL OPERATION _____

OPERATING DATA REPORT

DOCKET NO.: 50-281
 DATE: 12/05/89
 COMPLETED BY: L.A. Warren
 TELEPHONE: (804)357-3184 x355

OPERATING STATUS

NOTES

1. Unit Name: Surry Unit 2
2. Reporting Period: NOV 01-30, 1989
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 820
7. Maximum Dependable Capacity (Net MWe): 781
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reason For Restrictions, If Any: _____

	<u>THIS MONTH</u>	<u>YTD</u>	<u>CUMULATIVE</u>
11. Hours In Reporting Period	720.0	8016.0	145392.0
12. Number of Hours Reactor Was Critical	157.9	760.3	90454.6
13. Reactor Reserve Shutdown Hours	0	0	328.1
14. Hours Generator On-Line	152.2	611.9	88904.9
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	341996.3	7725110.3	207820743.8
17. Gross Electrical Energy Generated (MWH)	114585.0	341735.0	67477979.0
18. Net Electrical Energy Generated (MWH)	108802.0	320497.0	63967875.0
19. Unit Service Factor	21.1%	7.6%	61.1%
20. Unit Available Factor	21.1%	7.6%	61.1%
21. Unit Capacity Factor (Using MDC Net)	19.3%	5.1%	56.5%
22. Unit Capacity Factor (Using DER Net)	19.2%	5.1%	55.8%
23. Unit Forced Outage Rate	78.9%	63.4%	15.8%
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down at End of Report Period Estimated Date of Startup: _____
26. Unit In Test Status (Prior to Commercial Operation): FORECAST ACHIEVED

INITIAL CRITICALITY _____
 INITIAL ELECTRICITY _____
 COMMERCIAL OPERATION _____

UNIT SHUTDOWN AND POWER REDUCTION

REPORT MONTH: NOVEMBER 1989

DOCKET NO.: 50-280
 UNIT NAME: Surry Unit 1
 DATE: 12/05/89
 COMPLETED BY: L.A. Warren
 TELEPHONE: 804-357-3184 x355

NO.	DATE	TYPE(1)	DURATION (HOURS)	REASON(2)	METHOD OF SHUTTING DOWN REACTOR(3)	LICENSEE EVENT REPORT#	SYSTEM CODE(4)	COMPONENT CODE(5)	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
	11/16/89	S	0	B	4		TA	TRB	Ramped Unit to 73% power for 1-PT-29.1 Main Turbine Valve Testing and Clean Waterboxes. Load reduced to 600 MWE.
	11/28/89	S	0	B	4		SG	COND	Ramped down to 80% power 600 MWE to clean waterboxes.

(1)
F: Forced
S: Scheduled

(2)
REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & License Examination
 F - Administrative
 G - Operational Error (Explain)
 H - Other (Explain)

(3)
METHOD:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source

UNIT SHUTDOWN AND POWER REDUCTION

REPORT MONTH: NOVEMBER 1989

DOCKET NO.: 50-281
 UNIT NAME: Surry Unit 2
 DATE: 12/05/89
 COMPLETED BY: L.A. Warren
 TELEPHONE: 804-357-3184 x355

NO.	DATE	TYPE(1)	DURATION (HOURS)	REASON(2)	METHOD OF SHUTTING DOWN REACTOR(3)	LICENSEE EVENT REPORT#	SYSTEM CODE(4)	COMPONENT CODE(5)	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
C-10-3	11/01/89	F	567.8	A	1		AB	RV	Pressurizer safety 2-RC-SV-2551B leaking by its seat.

(1)	(2)	(3)	(4)
F: Forced	REASON:	METHOD:	
S: Scheduled	A - Equipment Failure (Explain)	1 - Manual	Exhibit G - Instructions for
	B - Maintenance or Test	2 - Manual Scram.	Preparation of Data Entry Sheets
	C - Refueling	3 - Automatic Scram.	for Licensee Event Report (LER)
	D - Regulatory Restriction	4 - Other (Explain)	File (NUREG 0161)
	E - Operator Training & License Examination		
	F - Administrative		(5)
	G - Operational Error (Explain)		
	H - Other (Explain)	4	Exhibit 1 - Same Source

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.: 50-280
UNIT NAME: Surry Unit 1
DATE: 12/05/89
COMPLETED BY: L.A. Warren
TELEPHONE: (804)357-3184 x355

MONTH: NOVEMBER 1989

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>783</u>	17	<u>786</u>
2	<u>783</u>	18	<u>787</u>
3	<u>781</u>	19	<u>785</u>
4	<u>782</u>	20	<u>785</u>
5	<u>783</u>	21	<u>788</u>
6	<u>781</u>	22	<u>789</u>
7	<u>783</u>	23	<u>786</u>
8	<u>783</u>	24	<u>786</u>
9	<u>783</u>	25	<u>789</u>
10	<u>783</u>	26	<u>792</u>
11	<u>783</u>	27	<u>790</u>
12	<u>783</u>	28	<u>769</u>
13	<u>785</u>	29	<u>662</u>
14	<u>785</u>	30	<u>791</u>
15	<u>784</u>	31	<u> </u>
16	<u>782</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.: 50-281
UNIT NAME: Surry Unit 2
DATE: 12/05/89
COMPLETED BY: L.A. Warren
TELEPHONE: (804)357-3184 x355

MONTH: NOVEMBER 1989

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>	17	<u>0</u>
2	<u>0</u>	18	<u>0</u>
3	<u>0</u>	19	<u>0</u>
4	<u>0</u>	20	<u>0</u>
5	<u>0</u>	21	<u>0</u>
6	<u>0</u>	22	<u>0</u>
7	<u>0</u>	23	<u>0</u>
8	<u>0</u>	24	<u>157</u>
9	<u>0</u>	25	<u>588</u>
10	<u>0</u>	26	<u>780</u>
11	<u>0</u>	27	<u>772</u>
12	<u>0</u>	28	<u>769</u>
13	<u>0</u>	29	<u>767</u>
14	<u>0</u>	30	<u>767</u>
15	<u>0</u>	31	<u></u>
16	<u>0</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: NOVEMBER 1989

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

11/01/89	0000	This reporting period begins with the Unit at 100% power.
11/16/89	0916	Started ramping down to perform PT-29.1. 100% power, 820 MWe.
	1109	Stopped ramp at 73% power, 600 MWe.
	1530	Started ramping up following PT-29.1, 590 MWe.
	1553	Stopped ramp for turnover, 640 MWe, 78% power.
	1625	Started ramp up.
	1816	Stopped ramp at 100% power, 815 MWe.
11/28/89	2021	Commenced ramp down to 660 MWe, 100% power, 825 MWe to clean all four waterboxes.
	2144	Stopped ramp, 79.8% power, 665 MWe.
11/29/89	1735	Commenced Unit ramp to 100% power; 80% power, 670 MWe.
	1842	Stopped ramp at 96% power, 800 MWe, 572°F Tavg. to stabilize and verify NIS and reactor power to match prior to approaching 100%.
	2023	Unit at 100%, 830 MWe, 575°F Tavg.
11/30/89	2400	This reporting period ends with the Unit at 100% power.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: NOVEMBER 1989

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT TWO

11/01/89 0000 This reporting period begins with the Unit at cold shutdown due to leakage of 2-RC-SV-2551B.

11/05/89 2105 RCS temp. >200°F.

11/06/89 0555 RCS temp. >350°F.

11/07/89 0132 Inserted shutdown banks due to SV-2551C inoperable due to setpoint exceeding 1% tolerance-commenced cooldown and declared unusual event.

1416 Unit 2 <350°F, 450 PSIG.

2230 RCS temperature indication <200°F. Unit at CSD, N.O.U.E. terminated.

11/20/89 1229 RCS AT 200°F and heating up.

2256 Unit at 350°F, 450 PSIG.

11/21/89 0315 RCS >1000 PSIG.

0345 RCS <1000 PSIG, hold temperature and pressure 470°F and 960 PSIG.

0428 RCS >1000 PSIG.

0946 RCS at 2200 PSIG.

2007 Inserted shutdown banks.

11/22/89 0515 Unit at 1800 PSIG due to leakage of pressurizer safety valves.

0655 Started pressure increase from 1800 - 2000 PSIG.

11/24/89 0743 Reactor start up commencing, pulling control rods.

0953 Reactor critical; increasing to 10^{-8} amps in the intermediate range.

1548 Unit on line.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: NOVEMBER 1989

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT TWO

11/25/89 0517 Commenced ramp-up reactor power 41%, 350 MWe.
0530 Stopped ramp due to 'B' main feed regulating valve dropped 'B' S/G level to 32%.
0538 'B' main feed regulating valve back in auto; commenced ramp up.
0616 Stopped ramp at 60%, 475 MWe to start 2-FW-P-1B.
0653 Started ramp to 70% reactor power, 60% and 480 MWe.
0726 Stopped ramp at 70%, 550 MWe. for calorimetric.
1212 Holding ramp at 84% reactor power, Techs adjusting IRPIs; 680 MWe.
1303 IRPI adjustments completed, commenced ramp up.
1415 Reactor at 100% power, 820 MWe.
11/26/89 2148 Reduced reactor power to 98% due to 'B' loop protection differential temperature reading high.
11/30/89 2400 This reporting period ended with the Unit at 98% power.

FACILITY CHANGES REQUIRING NRC APPROVAL

MONTH/YEAR: NOVEMBER 1989

NONE DURING THIS REPORTING PERIOD

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: NOVEMBER 1989

1-EWR-89-702 ENGINEERING WORK REQUEST

11/05/89

This work request was for the interim removal of drain valve 1-CH-427 and its associated piping and cap; also, to install a pipe plug in the 2" x 2" x 3/4" tee on line 2" CH-6-602.

Implementation of this activity will return the system and containment pressure boundaries to an acceptable state. The drain line removal will have no impact on continued operation of the charging system. An unreviewed safety question does not exist because the consequences of failure of the activity are bounded by the Design Basis Accident analysis in the UFSAR, section 14.5 and are in comparison, insignificant. This interim modification is necessary until an acceptable technical solution is implemented to reduce fatigue on the drain line assembly.

UFSAR
Sec.14.2.9

UPDATED FINAL SAFETY ANALYSIS REVIEW - UNITS 1&2
(SAFETY EVALUATION #N89-0022)

11/07/89

An Updated Final Safety Analysis Report (UFSAR) change was written to correct a calculational error in SM-457 that led to a non-conservative value for peak clad temperature (PCT) following a locked rotor event. This value was reported to USNRC in support of operation with Surry Improved Fuel.

This is a correction of the error that led to an increase in PCT from 1654°F to 1795°F. This evaluation demonstrates that an unreviewed safety question does not exist because the design acceptance criteria for PCT of 2700°F is still met. The NRC was formally informed of this correction by a letter (Serial No. 89-716) dated November 14, 1989.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: NOVEMBER 1989

SCAFFOLD REQUEST - UNIT 2 11/13/89
(SAFETY EVALUATION #N89-0027)

This scaffold request is for the erection of temporary scaffold to support painting of Unit 2 turbine building per Work Order #3800083521.

This temporary scaffold was required for safe working. Scaffolds erected per SUADM-ADM-07 have a high confidence factor against failure and it was reviewed for effects on accident analysis and equipment operability. In view of this, it was concluded that the assumptions and probabilities of accident analysis and equipment malfunctions are not significantly affected by this scaffold.

TM-S2-89-139	<u>TEMPORARY MODIFICATION</u> (SAFETY EVALUATION #N89-0030)	11/16/89
TM-S2-89-140	<u>TEMPORARY MODIFICATION</u> (SAFETY EVALUATION #N89-0033)	11/16/89
TM-S2-89-141	<u>TEMPORARY MODIFICATION</u> (SAFETY EVALUATION #N89-0032)	11/16/89

Each temporary modification allowed starting a reactor coolant pump (RCP) with normal cold reactor coolant system (RCS) starting amps expected.

Since RCPs are not used for safety related functions outside of their RCS pressure retaining capacity and Operators are instructed to stop a RCP within 10 seconds if no flow is indicated after pump start, an unreviewed safety question is not created by the temporary removal of the RCP speed sensing relay.

TM-S2-89-136	<u>TEMPORARY MODIFICATION</u> (SAFETY EVALUATION #N89-0031)	11/16/89
TM-S2-89-137	<u>TEMPORARY MODIFICATION</u> (SAFETY EVALUATION #N89-0035)	11/16/89
TM-S2-89-138	<u>TEMPORARY MODIFICATION</u> (SAFETY EVALUATION #N89-0034)	11/16/89

Each temporary modification allowed normal running of a RCP with bearing lift oil to lower shoes at low reactor coolant system pressures.

Since RCPs are not assumed in accident analyses and this change does not affect RCS pressure retaining portion of RCPs, an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: NOVEMBER 1989

TM-S2-89-147 TEMPORARY MODIFICATION (SAFETY EVALUATION #N89-044) 11/30/89

This modification was written to energize relay 33YBV595 (the relay should be energized when 2-RC-MOV-2595 is open, as it will be throughout the time this change is in effect) to enable restarting 'C' reactor coolant pump should it trip and be required/desired to be restarted.

Since 'C' loop cold leg loop stop valve will remain open and de-energized while change is in effect, the bypassing of the interlock to prevent starting 'C' RCP with cold leg loop stop valve closed has no effect.

PROCEDURE OR METHOD OF OPERATION CHANGES
REQUIRING NRC APPROVAL

MONTH/YEAR: NOVEMBER 1989

NONE DURING THIS REPORTING PERIOD

PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: NOVEMBER 1989

1-TOP-40.20 TEMPORARY OPERATING PROCEDURE 11/04/89
(SAFETY EVALUATION #N89-0021)

This procedure allows filling and placing in service the normal letdown following maintenance of 1-CH-427.

During the performance of this procedure the letdown line will remain isolated from the RCS. Also, primary grade water that is used will be diverted to the primary drains tank prior to placing letdown in service. Assumptions used in the accident analysis will not be altered during the performance of this procedure. Therefore, an unreviewed safety question does not exist.

1-TOP-2148 TEMPORARY OPERATING PROCEDURE 11/10/89
(SAFETY EVALUATION #N89-0024)

This procedure was written to allow adjusting 01-FW-E-1B level. This procedure included disabling the high level alarm.

Since this procedure only affects the condensate and main feedwater systems and is bounded by the previously analyzed loss of feedwater accident, an unreviewed safety question does not exist.

1/2-PT-18.8 PERIODIC TEST PROCEDURE DEVIATION 11/22/89
(SAFETY EVALUATION #N89-0037/38)

This deviation deleted the differential pressure method of testing check valves in the closed position for 1/2-SW-P-10A/B, and 2-CC-P-2A,B.

An unreviewed safety question is not created since the check valves will still be tested in the closed position to ensure system operability. This change is in accordance with the ASME XI inservice inspection plan and will not affect the ability of the charging pump service water and component cooling subsystems to mitigate the consequences of an accident.

TESTS AND EXPERIMENTS REQUIRING NRC APPROVAL

MONTH/YEAR: NOVEMBER 1989

NONE DURING THIS REPORTING PERIOD

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: NOVEMBER 1989

NONE DURING THIS REPORTING PERIOD

VIRGINIA POWER
SURRY POWER STATION
CHEMISTRY REPORT

MONTH/YEAR: NOVEMBER 1989

PRIMARY COOLANT ANALYSIS	UNIT NO. 1			UNIT NO. 2		
	MAX.	MIN.	AVG.	MAX.	MIN.	AVG.
Gross Radioact., $\mu\text{Ci/ml}$	8.05E-1	5.28E-1	6.65E-1	3.39E-1	9.40E-4	2.89E-2
Suspended Solids, ppm	0.0	0.0	0.0	0.0	0.0	0.0
Gross Tritium, $\mu\text{Ci/ml}$	2.97E-1	2.81E-1	2.87E-1	4.27E-2	7.34E-3	1.93E-2
Iodine-131, $\mu\text{Ci/ml}$	3.24E-2	2.29E-3	1.15E-2	5.01E-4	4.81E-5	2.13E-4
Iodine-131/Iodine-133	0.24	0.07	0.15	0.14	0.10	0.12
Hydrogen, cc/kg	32.9	15.3	22.3	33.5	3.7	18.7
Lithium, ppm	2.33	2.05	2.18	2.95	0.87	1.82
Boron - 10, ppm*	130.3	115.4	122.8	469.8	196.6	394.6
Oxygen, (DO), ppm	0.005	0.005	0.005	3.0	<0.005	0.400
Chloride, ppm	0.009	0.006	0.007	0.012	0.004	0.007
pH @ 25 degree Celsius	6.89	6.69	6.83	6.54	5.48	5.80

* Boron - 10 = Total Boron x 0.196

REMARKS:

UNIT ONE

The month began with the unit at 100% power with letdown secured due to a leak on the non-regenerative heat exchanger causing RCS dissolved hydrogen concentration to exceed the lower specification limit at 2055 on 10/01/89. On 11/05/89 at 0555, B Mixed Bed was placed back in service and the dissolved hydrogen concentration returned to within normal limits at 1230. A 165 gm lithium addition was also made on 11/05/89. A 320 gm lithium addition was made on 11/17/89. A 420 gm lithium addition was made on 11/29/89.

UNIT TWO

The month began with the unit at cold shutdown with BMB inservice. On 11/04/89 a total of 2.5 gallons of hydrazine were added to the RCS. A 1500 gm. lithium addition was also made on 11/04/89. On 11/23/89, 2822 gm of lithium were added to the RCS. The reactor went critical at 0955 on 11/24/89. A lithium addition of 275 gm to the RCS was made on 11/26/89. The cation bed was placed inservice on 11/28/89 for lithium removal.

UNIT 1&2

FUEL HANDLING

DATE: NOV 1989

NEW OR SPENT FUEL SHIPMENT #	DATE SHIPPED OR RECEIVED	NUMBER OF ASSEMBLIES PER SHIPMENT	ASSEMBLY NUMBER	ANSI NUMBER	INITIAL ENRICHMENT	NEW OR SPENT FUEL SHIPPING CASK ACTIVITY LEVEL
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NONE DURING THIS REPORTING PERIOD

DESCRIPTION OF PERIODIC TEST WHICH WERE NOT COMPLETED
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

MONTH/YEAR: NOVEMBER 1989

NONE DURING THIS REPORTING PERIOD

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: NOVEMBER 1989

AC-S1-89-1106 ADMINISTRATIVE CONTROL (SAFETY EVALUATION #N89-0040) 11/27/89

This administrative control ensures that the main control room (MCR) pressure envelope can be maintained. The service air supply valve to the #2 emergency switch gear room (ESGR) (1-SA-226) must be maintained under administrative control when the valve is open.

Administrative control over 1-SA-226 in accordance with SUADM-0-26 ensures the ability to maintain the MCR/ESGR pressure boundary in the event of a radioactive release. The valve will be closed immediately upon notification by the MCR. In addition, control over the valve will ensure the ability to maintain the pressure boundary thus maintaining operability of the MCR air bottle dump system. Therefore, an unreviewed safety question is not created.

SCAFFOLD REQUEST - UNIT 1
(SAFETY EVALUATION #N89-0041)

11/27/89

This request was written to erect scaffolds to support the replacement of component cooling heat exchangers in Unit 1 turbine basement per Design Change 87-29.

The temporary scaffold was required for safe working. Installation of scaffold constructed per SUADM-ADM-07 has a high confidence level against failure and was reviewed for effects on accident analyses and equipment operability/function. It is thus concluded that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

1-EWR-87-312 ENGINEERING WORK REQUEST

11-29/89

This request was to replace the turbine lube oil and to improve its quality and to reduce the maintenance time spent on the system's upkeep.

The Lube Oil System is an enhancement by design, as it is used to purify the turbine lube oil. Its operability is not required during safe shutdown nor has any effect on safety related equipment. Therefore, an unreviewed safety question is not created.

EWR-89-462 ENGINEERING WORK REQUEST - UNITS 1&2

11/29/89

This request is for the installation of additional air bottles in the compressed dry air system. The bottled air system serving the control room and the emergency switchgear room has marginal capacity to maintain the required positive pressure; the additional bottles will improve system performance.

The consequences or probability of an accident has not been increased due to the addition of air bottles to the control room compressed breathing air system.