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 FACIL:50-281 Surry Power Station, Unit 2, Virginia Electric & Powe 05000281
 AUTH.NAME AUTHOR AFFILIATION
 KANSLER,M.R. Virginia Power (Virginia Electric & Power Co.)
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 89-014-02:on 891117,leakage through containment purge
 MOV exceeds max allowable.

W/8 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 4
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NOTES:1cy NMSS/FCAF/PM.

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	ACRS WYLIE	1 1	AEOD/DOA	1 1
	AEOD/DSP/TPAB	1 1	AEOD/ROAB/DSP	2 2
	DEDRO	1 1	NRR/DET/ECMB 9H	1 1
	NRR/DET/EMEB9H3	1 1	NRR/DET/ESGB 8D	1 1
	NRR/DLPQ/LHFB11	1 1	NRR/DLPQ/LPEB10	1 1
	NRR/DOEA/OEAB11	1 1	NRR/DREP/PRPB11	2 2
	NRR/DST/SELB 8D	1 1	NRR/DST/SICB 7E	1 1
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	NUDOCS-ABSTRACT	1 1	<u>REG FILE</u> 02	1 1
	RES/DSIR/EIB	1 1	RGN2 FILE 01	1 1
EXTERNAL:	EG&G WILLIAMS, S	4 4	L ST LOBBY WARD	1 1
	LPDR	1 1	NRC PDR	1 1
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VIRGINIA ELECTRIC AND POWER COMPANY

Surry Power Station
P.O. Box 315
Surry, Virginia 23883

December 18, 1989

U. S. Nuclear Regulatory Commission
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Washington, D.C. 20555

Serial No.: 89-058
Docket No.: 50-281
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Gentlemen:

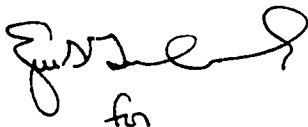
Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report for Unit 2.

REPORT NUMBER

89-014-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Corporate Nuclear Safety.

Very truly yours,



M. R. Kansler
Station Manager

Enclosure

cc: Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

8912260023 891218
FDR ADCOCK 05000281
S FDC



LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Surry Power Station, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 8 1	PAGE (3) 1 OF 0 3
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TITLE (4)
Leakage Through Containment Purge MOV Exceeds Maximum Allowable

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
11	17	89	89	014	00	12	18	89		0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)				
POWER LEVEL (10) 0.00	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)	
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.38(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)	
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.38(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify on Abstract below and in Text: NRC Form 365A)	
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)		
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(B)		
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)			

LICENSEE CONTACT FOR THIS LER (12)

NAME M. R. Kansler, Station Manager	TELEPHONE NUMBER
	AREA CODE: 8 0 4 3 5 7 - 3 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	JM	V	A 180						

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 17, 1989 with Unit 2 in cold shutdown, the results of the Containment Purge Leakage test indicated that the Unit 2 Containment Purge Supply Penetration (comprised of 2-VS-MOV-200A, 2-VS-MOV-200B and 2-VS-MOV-202) exceeded the maximum allowable leakage value of .6La (180 SCFH) as defined in 10CFR50 Appendix J. These valves are part of the containment isolation boundary. It was determined that the total leakage through these valves was 272 SCFH. This total leakage was assigned to the penetration in accordance with ANSI 56.8 requirements for the test method used. Because some of these valves had been cycled since they were last tested on October 24, 1989, and the unit had operated above 200 degrees Fahrenheit for three days, the potential existed that containment leakage exceeded Technical Specification requirements. The valves were repaired and tested satisfactorily prior to Unit 2 leaving cold shutdown.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THE INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Surry Power Station, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 8 1	LER NUMBER (3)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 9	0 1 4	0 0	0 2	OF 0 0

TEXT (If more space is required, use additional NRC Form 368A's) (17)

1.0 Description of the Event

On November 17, 1989, with Unit 2 in cold shutdown, the results of the Containment Purge Leakage test 2-PT-16.11 indicated that the Unit 2 Containment Purge (EIIS-BA) Supply Penetration (comprised of 2-VS-MOV-200A, 2-VS-MOV-200B and 2-VS-MOV-202) (EIIS-V) exceeded the maximum allowable leakage value of .6La (180 SCFH) as defined in 10CFR50 Appendix J. These valves are part of the containment isolation boundary. It was determined that the total leakage through these valves was 272 SCFH. Unit 2 had been greater than 200 degrees Fahrenheit for approximately three days and returned to cold shutdown for maintenance on the pressurizer safety valves. Since some of these valves had been cycled and not retested prior to exceeding 200 degrees Fahrenheit, the potential existed that containment leakage exceeded Technical Specification requirements. This report is being made in accordance with 10CFR50.73(a)(2)(i)(B) based upon the potential for exceeding the Technical Specification allowed containment leakage.

2.0 Safety Consequences and Implications

Containment Purge Supply valves provide purge air to containment and the Containment Vacuum Breaker Atmospheric Supply Valve is used to break containment vacuum during shutdown conditions. These valves are installed and are closed when the unit is above cold shutdown to provide containment integrity. Due to system configuration, an individual valve leakage rate could not be computed and a total leakage rate of 272 was assigned to the penetration in accordance with ANSI 56.8 requirements. Assuming the worst case leakage through each valve, the maximum leakage out of containment would have been 136 SCFH and within the allowable value. Therefore, the health and safety of the public were not affected.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 9	- 0 1 4	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

3.0 Cause

It is suspected that the majority of the leakage was through 2-VS-MOV-200A since it was discovered with a worn rubber seating material and only minor adjustments were required for the other valves.

4.0 Immediate Corrective Action(s)

None.

5.0 Additional Corrective Action(s)

The seat was replaced on 2-VS-MOV-200A, the seating surface on 2-VS-MOV-200B was cleaned and adjusted, and the packing was adjusted on 2-VS-MOV-202. The valves were retested satisfactorily in accordance with 2-PT-16.11.

6.0 Action(s) Taken to Prevent Recurrence

None.

7.0 Similar Events

LER 2-86-014: The inside and outside containment sump trip valves had as found leakage greater than 300 SCFH. It was determined that the failure of the globe type trip valves were due to erosion of the plug and seat debris in the effluent stream.

LER 2-88-002: The reactor coolant cold leg sample isolation valves leakage was greater than specification.

8.0 Manufacturer/Model Number(s)

Allis Chalmers Corp./50FWR.