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AUTH.NAME AUTHOR AFFILIATION
KANSLER,M.R. Virginia Power (Virginia Electric & Power Co.)
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 89-012-00:on 890919,individual rod position indicators out of spec for greater than 60 min in 24 h period.
W/8 ltr.

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TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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VIRGINIA ELECTRIC AND POWER COMPANY
Surry Power Station
P.O. Box 315
Surry, Virginia 23883

October 18, 1989

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Serial No.: 89-049
Docket Nos.: 50-281
License Nos.: DPR-37

Gentlemen:

Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report for Unit 2.

REPORT NUMBER

89-012-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

for

M. R. Kansler
Station Manager

Enclosure

cc: Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

IE22
1/1

NRC FORM 366 (6-89)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 4/30/92

ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Surry Power Station, Unit 2	0 5 0 0 0 2 8 1	1 OF 0 4

TITLE (4) Individual Rod Position Indicators Out of Specification For Greater Than 60 Minutes in a 24 Hour Period

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 9	1 9	8 9	8 9	— 0 1 2	— 0 0	1 0	1 8	8 9		0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 2	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
M. R. Kansler, Station Manager	AREA CODE 8 0 4 — 3 5 7 - 3 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	
X	A A	Z I	M 0 3 5	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 19, 1989 at 0815 hours during a unit startup with the reactor critical and less than 2% power, it was discovered that two Individual Rod Position Indicators (IRPI) in control bank "A", group two, had differed in indicated position from the bank's step demand counter by more than 12 steps for 63 and 64 minutes within the last 24 hours. This occurrence is contrary to Technical Specification (T.S.) 3.12.E, which allows this condition to exist for a maximum of 60 minutes in any 24 hour period. This event was due to an inadequate method used to log accumulated times for IRPI differences in excess of 12 steps. The procedure used to log these accumulated times will be modified and the plant computer will be reprogrammed to monitor accumulated times and alarm to alert the operator prior to exceeding the 60 minute limit.

1.0 Description of the Event

On September 19, 1989 at 0815 hours during a unit startup with the reactor critical and less than 2% power, it was discovered that two Individual Rod Position Indicators (IRPI) (EIIS-ZI) in control bank "A" group two, had differed in indicated position from the bank's step demand counter by more than 12 steps for 63 and 64 minutes within the last 24 hours. This occurrence is contrary to Technical Specification (T.S.) 3.12.E, which allows this condition to exist for a maximum of 60 minutes in any 24 hour period.

The subject IRPIs were adjusted to within 12 steps of their step demand counters approximately 15 minutes prior to reactor criticality, however, the event was not realized until thirty minutes after the adjustment.

2.0 Safety Consequences and Implications

The IRPIs provide control rod position indication in the main control room. The control rod demand counters, which indicate the control rod group position as demanded by the rod control system, (EIIS-AA) remained operable during the time the IRPIs were not within specification.

The inoperable IRPIs did not affect the operation of the control rod assemblies. All rods were capable of being tripped into the core if required. Therefore, the health and safety of the public were not affected.

3.0 Cause

IRPIs associated with Westinghouse PWRs are known to drift as a result of reactor coolant temperature and rod position changes at low reactor power levels. Due to this generic problem, a Technical Specification change was recently approved to allow a greater than 12 step, but less than 24 step difference between IRPIs and bank step demand counters for a maximum of one hour in any 24 hour period.

The event was due to an inadequate method used to log accumulated out of specification time for the IRPIs. The greater than one hour accumulated out of specification time occurred over three separate time periods during rod withdrawal. The three events were recorded on two separate procedures corresponding to two different shifts. The operator performing the last procedure on the later shift had not been informed of the accumulated time recorded on the earlier shift's procedure and was not aware of the additional time that the IRPIs difference had exceeded 12 steps. The event could have been averted by discontinuing rod withdrawal had the on shift operator been aware of this condition.

4.0 Immediate Corrective Action(s)

None required since the IRPIs had been adjusted to within the required T.S. tolerance prior to the discovery of the event.

5.0 Additional Corrective Action(s)

None required.

6.0 Action(s) Taken to Prevent Recurrence

The procedure used to log the accumulated time the IRPIs differ from their respective bank's step demand counter by more than 12 steps will be modified to ensure that these accumulated times are transferred during shift relief.

Additionally, the plant computer will be reprogrammed to monitor these accumulated times and will alarm to alert the operator prior to exceeding the 60 minute limit.

7.0 Similar Events

Although an event in which exceeding the allowed accumulated time for IRPI deviation has not occurred before, past incidents of IRPI deviation have been reported which prompted the T.S. change described above.

8.0 Manufacturer/Model Number(s)

Magnetics Inc./Project #K9391, K9392.