

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

W. R. CARTWRIGHT
VICE PRESIDENT
NUCLEAR

March 27, 1989

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Serial No. 88-796
NO/ETS:
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
SUMMARY REPORTS AND PROPOSED TECHNICAL SPECIFICATION CHANGES
CIRCULATING WATER, SERVICE WATER AND RELATED SYSTEMS

In our letter (Serial No. 88-689) of October 19, 1988, and during a meeting on October 26, 1988, several items were identified which required appropriate corrective action prior to restart of the Surry units. This letter provides summary reports which discuss the Canal Inventory issues, Component Cooling heat exchanger operability, and the Emergency Service Water Pump modifications.

As part of our corrective actions, pursuant to 10 CFR 50.90, the Virginia Electric and Power Company requests an amendment in the form of a change to the Technical Specifications to Operating Licenses DPR-32 and DPR-37 for Surry Power Station Units 1&2, respectively. This change, provided in Attachment 1, will:

- 1) raise the minimum Circulating and Service Water Intake Canal level from eighteen feet to twenty-three feet,
- 2) increase the requirement from two to three Emergency Service Water pumps to be operable with provision for limited duration maintenance outages, and
- 3) provide operability and surveillance requirements for the new safety-related canal level actuation system.

The summary reports address issues raised in the Safety System Functional Inspection. The proposed Technical Specification changes will ensure consistency between the operating requirements and the reconstituted plant design basis in the area of Intake Canal inventory management and Component Cooling heat exchanger operability.

These proposed changes have been reviewed by the Station Nuclear Safety and Operating Committee and the Safety Evaluation and Control staff. It has been

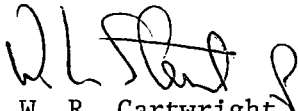
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determined that the changes constitute an unreviewed safety question as defined in 10CFR50.59, but do not involve a significant hazards consideration as defined in 10 CFR 50.92. The determination that these changes constitute an unreviewed safety question is largely due to the reconstituted design basis which represents a substantial change from the design specified in the UFSAR. Our basis for determining that no significant hazards consideration exists is contained in Attachment 1 and is supplemented by the report provided in Attachment 2.

This letter also provides the final summary report for the Emergency Service Water pumps. This report is provided in Attachment 3.

Very truly yours,


W. R. Cartwright

Attachments

1. Proposed Technical Specifications Change
2. Evaluation of Surry Intake Canal Inventory and Component Cooling Heat Exchanger Operability
3. Evaluation of Emergency Service Water Pump Modifications

cc: U. S. Nuclear Regulatory Commission
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