

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Surry Power Station, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 8 1 1	PAGE (3) 1 OF 0 3
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TITLE (4)
Inoperable Individual Rod Position Indicators Due To Instrument Drift

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
09	09	88	88	020	00	10	10	88			05000

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 5 2	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)						
	20.405(a)(1)(i)	50.38(c)(1)	50.73(a)(2)(v)	73.71(c)						
	20.405(a)(1)(ii)	50.38(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(ii)	50.73(a)(2)(viii)(A)							
	20.405(a)(1)(iv)	50.73(a)(2)(iii)	50.73(a)(2)(viii)(B)							
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME D. L. Benson, Station Manager	AREA CODE 8 0 3	NUMBER 3 5 7 1 - 3 1 1 8 4	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	AIA	Z III	M 10 3 1 5	N						

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 9, 1988 at 2245 hours, with Unit 2 at 52% power, following the initiation of ramp from 68% power for a shutdown, the control room operator observed that the Individual Rod Position Indicators (IRPIs) for four control rods of control bank "D", differed from the rod group demand counter by greater than 12 steps. The IRPIs were declared inoperable in accordance with Technical Specification 3.12.E. The cause of the event is due to the drift in IRPI indication as a result of primary coolant temperature changes. This is known to be a generic Westinghouse PWR concern.

The rampdown was stopped and IRPIs were adjusted to within the required T. S. tolerance. Operators have been instructed to more closely monitor IRPIs during planned primary temperature changes and initiate appropriate corrective actions when trends indicate the IRPIs are drifting out of specification. A Technical Specification change is being prepared which will accommodate the effects of primary coolant temperature changes on the IRPIs.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			- 0 2 0	- 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

1.0 Description of the Event

On September 9, 1988 at 2245 hours, with Unit 2 at 52% power, following the initiation of ramp from 68% power for a shutdown, the control room operator observed that the Individual Rod Position Indicators (IRPIs) {EIIS-ZI} for control rods {EIIS ROD} B-8, H-2, F-6 and K-6 of control bank "D", differed from the rod group demand counter by greater than 12 steps.

The IRPIs were declared inoperable in accordance with Technical Specification 3.12.E. The IRPIs were adjusted and the rampdown was reinitiated.

2.0 Safety Consequences and Implications

The IRPIs provide control rod position indication in the main control room. The control rod demand counters, which indicate the control rod group position as demanded by the rod control system {EIIS-AA}, remained operable during the event. In addition, inoperable IRPIs will not affect the operation of the control rod assemblies. All rods were capable of being dropped into the core, if required. Therefore, the health and safety of the public were not affected.

3.0 Cause

The cause of the event is due to the drift in IRPI indication as a result of primary coolant temperature changes. This is known to be a generic Westinghouse PWR concern.

4.0 Immediate Corrective Action(s)

The rampdown was stopped and IRPIs were adjusted to within the required T. S. tolerance.

5.0 Additional Corrective Action(s)

None required.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

6.0 Action(s) Taken to Prevent Recurrence

A setpoint change has been made which lowered the P-250 process computer rod deviation alarm to alert the operator when the IRPIs deviate from the group demand position by 10 steps. Although this prompted operators to initiate corrective action prior to exceeding the T. S. limits, it did not prevent recurrence of the event. Therefore, operators have been instructed to more closely monitor IRPIs during planned primary temperature changes and initiate appropriate corrective actions when trends indicate the IRPIs are drifting out of specification.

A Technical Specification change is being prepared which will accommodate the effects of primary coolant temperature changes on the IRPIs. Until the T. S. change is approved, the IRPIs will be adjusted to agree with the step demand counters as required during primary temperature changes.

7.0 Similar Events

Unit 1 LERs: 88-026, 88-024, 88-010, 88-006, 88-002, 86-021, & 85-019.

Unit 2 LERs: 88-012, 88-007, 88-003, 86-019, & 85-009.

8.0 Manufacturer/Model Number

Magnetics Inc./Project #K9391, K9392.

VIRGINIA ELECTRIC AND POWER COMPANY
Surry Power Station
P. O. Box 315
Surry, Virginia 23883

October 10, 1988

U.S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Serial No.: 88-052
Docket No.: 50-281
Licensee No.: DPR-37

Gentlemen:

Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 2.

REPORT NUMBER

88-020-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

David L. Benson

David L. Benson
Station Manager

Enclosure

cc: Dr. J. Nelson Grace
Regional Administrator
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Atlanta, Georgia 30323

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