

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Surry Power Station, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 8 1	PAGE (3) 1 OF 0 3
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TITLE (4) **"B" LHSI Pump Not Tested Within Required 8 Hour Interval Due To Inoperable Test Equipment**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	9	08	88	021	00	1	0	06			0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 7 2	20.405(a)(1)(i)	50.38(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.38(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)		
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)		
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME D. L. Benson, Station Manager	TELEPHONE NUMBER
	AREA CODE: 8 0 4 3 5 7 - 3 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 7, 1988 at 1750 hours, with Unit 2 at 72% power, the "A" Low Head Safety Injection (LHSI) pump was removed from service to facilitate maintenance activities. Prior to this, at 1611 hours, the "B" LHSI pump had been satisfactorily tested per the requirements of Technical Specification (T. S.) 3.3.B.3. At 2326 hours, a second test was commenced on the "B" pump per the eight hour test frequency required by T. S. The test was not completed satisfactorily until September 8, at 0050 hours, exceeding the required test frequency by 39 minutes. The delay of completing the test on the "B" pump was due to the inability of the operator performing the test to obtain the required vibration data with the test equipment being used. Another vibration test instrument of a different type was obtained and the test was completed satisfactorily. Formal training on use of the vibration test equipment used during the event will be administered to the non-licensed operators.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

1.0 Description of the Event

On September 7, 1988 at 1750 hours, with Unit 2 at 72% power, the "A" Low Head Safety Injection (LHSI) pump {EIIS-PB, P} was removed from service in order to repair a leak on a flow element fitting. Prior to this, at 1611 hours, the "B" LHSI pump had been satisfactorily tested, per the requirements of Technical Specification (T. S.) 3.3.B.3, in preparation for the removal of the "A" LHSI pump from service. At 2326 hours, a second test was commenced on the "B" LHSI pump, in compliance with the eight hour test frequency required by T. S. However, during this test, operators had difficulty obtaining the vibration data required to complete the test using the vibration test equipment at hand and the test was delayed until another vibration meter could be found. The test was restarted after the other vibration meter was obtained and was completed satisfactorily at 0050 hours on September 8. This exceeded the required eight hour test frequency by 39 minutes.

2.0 Safety Consequences and Implications

The LHSI pumps function to provide coolant to the reactor core in the event of a large break loss of coolant accident. One operable LHSI pump is assumed in the safety analysis. Since the "B" LHSI pump remained operable throughout the duration of the event, as verified by satisfactory testing, the plant remained within the bounds of the safety analysis. Therefore, the health and safety of the public were not affected.

3.0 Cause

The delay in completing the operability test on the "B" LHSI pump was due to the inability of the operator performing the test to obtain the required vibration data with the test equipment being used. The data could not be obtained because the test equipment had been damaged and was not operating properly. Also, the operator performing the test did not recognize that the equipment was not operable until the test was already in progress.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

4.0 Immediate Corrective Action(s)

When the operator determined that he was unable to obtain the vibration data with the test equipment at hand, another vibration test instrument, of a different type, was obtained and the test was completed satisfactorily. Additionally, at 0011 hours on September 8, 1988, the time at which the required eight hour test frequency was exceeded, a six hour clock to hot shutdown was commenced per T. S. 3.0.1, and was terminated when the test was completed satisfactorily.

5.0 Additional Corrective Action(s)

The remaining tests on the "B" LHSI pump were performed within the required time intervals and the "A" LHSI pump was returned to service at 1713 hours on September 8, 1988.

6.0 Action(s) Taken to Prevent Recurrence

Formal training on use of the vibration test equipment used during the event will be administered to the non-licensed operators. In addition, operators will be reminded to thoroughly inspect test equipment prior to use.

7.0 Similar Events

None.

8.0 Manufacturer/Model Number

N/A

VIRGINIA ELECTRIC AND POWER COMPANY
Surry Power Station
P. O. Box 315
Surry, Virginia 23883

October 6, 1988

U.S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Serial No.: 88-050
Docket No.: 50-281
Licensee No.: DPR-37

Gentlemen:

Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 2.

REPORT NUMBER

88-021-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

David L. Benson

David L. Benson
Station Manager

Enclosure

cc: Dr. J. Nelson Grace
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

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