

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

November 9, 1994

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

Serial No. 94-650
NO/RPC:vlh
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of October 1994.

Very truly yours,



M. L. Bowling, Manager
Nuclear Licensing and Programs


Enclosure

cc: U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, N. W.
Suite 2900
Atlanta, Georgia 30323

Mr. M. W. Branch
NRC Senior Resident Inspector
Surry Power Station

150015

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R PDR



**VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION
MONTHLY OPERATING REPORT
REPORT NO. 94-10**

Approved:



Station Manager

11-7-94

Date

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OPERATING DATA REPORT

Docket No.: 50-280
 Date: 11-02-94
 Completed By: D. Mason
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 1
- 2. Reporting Period:..... October, 1994
- 3. Licensed Thermal Power (MWt):..... 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):..... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting Period.....	745.0	7296.0	191616.0
12. Number of Hours Reactor Was Critical.....	745.0	5794.7	129601.9
13. Reactor Reserve Shutdown Hours.....	0	0	3774.5
14. Hours Generator On-Line.....	745.0	5757.5	127436.5
15. Unit Reserve Shutdown Hours.....	0	0	3736.2
16. Gross Thermal Energy Generated (MWH).....	1792471.6	13419214.7	296586959.8
17. Gross Electrical Energy Generated (MWH)....	597190.0	4453050.0	96992598.0
18. Net Electrical Energy Generated (MWH).....	575524.0	4283373.0	92110474.0
19. Unit Service Factor.....	100.0%	78.9%	66.5%
20. Unit Availability Factor.....	100.0%	78.9%	68.5%
21. Unit Capacity Factor (Using MDC Net).....	98.9%	75.2%	62.0%
22. Unit Capacity Factor (Using DER Net).....	98.0%	74.5%	61.0%
23. Unit Forced Outage Rate.....	0.0%	0.5%	16.8%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Maintenance (Steam Generator Chemical Cleaning), December 1, 1994, 23 Days

25. If Shut Down at End of Report Period, Estimated Date of Start-up: _____

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATING DATA REPORT

Docket No.: 50-281
 Date: 11-02-94
 Completed By: D. Mason
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 2
- 2. Reporting Period: October, 1994
- 3. Licensed Thermal Power (MWt): 2441
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):.... 820
- 7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting Period	745.0	7296.0	188496.0
12. Number of Hours Reactor Was Critical	745.0	6797.2	126873.5
13. Reactor Reserve Shutdown Hours	0	0	328.1
14. Hours Generator On-Line.....	745.0	6787.5	125003.1
15. Unit Reserve Shutdown Hours.....	0	0	0
16. Gross Thermal Energy Generated (MWH).....	1817197.2	16020726.5	291696468.9
17. Gross Electrical Energy Generated (MWH)....	604515.0	5305770.0	95251394.0
18. Net Electrical Energy Generated (MWH).....	583296.0	5111591.0	90443661.0
19. Unit Service Factor.....	100.0%	93.0%	66.3%
20. Unit Availability Factor.....	100.0%	93.0%	66.3%
21. Unit Capacity Factor (Using MDC Net).....	100.2%	89.7%	61.5%
22. Unit Capacity Factor (Using DER Net).....	99.4%	88.9%	60.9%
23. Unit Forced Outage Rate.....	0%	0%	13.4%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
 Refueling (10 Year ISI), February 4, 1995, 55 Days

25. If Shut Down at End of Report Period, Estimated Date of Start-up: _____

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: October, 1994

Docket No.: 50-280
 Unit Name: Surry Unit 1
 Date: 11-03-94
 Completed by: Craig Olsen
 Telephone: (804) 365-2155

(1)	(2)	(3)	(4)	(5)				
Date	Type	Duration Hours	Reason	Method of Shutting Down Rx	LER No.	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence

None During the Reporting Period

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets
 for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source.

**UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)**

REPORT MONTH: October, 1994

Docket No.: 50-281
 Unit Name: Surry Unit 2
 Date: 11-03-94
 Completed by: Craig Olsen
 Telephone: (804) 365-2155

(1)	(2)	(3)	(4)	(5)				
Date	Type	Duration Hours	Reason	Method of Shutting Down Rx	LER No.	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence

None During the Reporting Period

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets
 for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-280
Unit Name: Surry Unit 1
Date: 11-02-94
Completed by: Pat Kessler
Telephone: 365-2790

MONTH: October, 1994

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	781	17	763
2	781	18	762
3	784	19	761
4	785	20	761
5	785	21	760
6	786	22	760
7	787	23	760
8	785	24	760
9	783	25	760
10	784	26	763
11	787	27	763
12	787	28	765
13	786	29	762
14	787	30	793
15	775	31	759
16	766		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-281
Unit Name: Surry Unit 2
Date: 11-02-94
Completed by: Pat Kessler
Telephone: 365-2790

MONTH: October, 1994

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	768	17	786
2	779	18	787
3	780	19	787
4	766	20	782
5	781	21	785
6	780	22	785
7	762	23	786
8	779	24	785
9	784	25	785
10	785	26	786
11	786	27	787
12	786	28	788
13	785	29	789
14	784	30	822
15	786	31	788
16	786		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: October, 1994

The following chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE:

10/01/94	0000	The reporting period began with the unit operating at 100% power, 810 MWe.
10/15/94	1028 1034	Started power reduction to stabilize Steam Generator "C" level oscillations. Stopped power reduction at 97.5%, 800 MWe.
10/31/94	2400	The reporting period ended with the unit operating at 97.5% power, 800 MWe, due to Steam Generator "C" level oscillations.

UNIT TWO:

10/01/94	0000	The reporting period began with the unit operating at 100% power, 805 MWe.
10/04/94	0910 0945 1133 1235 1338 1450	Started power reduction to perform Operations Surveillance Test, 2-OSP-TM-001, "Turbine Inlet Valve Freedom Test." Stopped power reduction at 94%, 770 MWe. Started power reduction to continue performance of 2-OSP-TM-001. Stopped power reduction at 88%, 725 MWe. Started power increase. Stopped power increase at 100%, 815 MWe.
10/31/94	2400	The reporting period ended with the unit operating at 100% power, 815 MWe.

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: October, 1994

TM S1-94-35 Temporary Modification 10-03-94
(Safety Evaluation No. 94-174)

Temporary Modification S1-94-35 installed electrical jumpers to bypass test switch PB-3-TA on the Unit 1 Reactor Protection System (RPS) Train "A" circuitry to allow the switch to be replaced without rendering the circuits inoperable.

RPS Train "B" was not affected by this activity and both trains remained capable of operating as designed. Double verification of jumper installation/removal and post-maintenance testing were performed. Therefore, an unreviewed safety question did not exist.

DCP 88-10 Design Change Package 10-10-94
(Safety Evaluation No. 91-085)

Design Change Package 88-10 added additional inputs to the NUREG-0696 multiplexing system and the Emergency Response Facility computer to provide emergency ventilation damper position and semi-vital bus voltage indications of the Technical Support Center and the Local Emergency Operations Facility.

This modification provides a reliable means of monitoring required Regulatory Guide 1.97 variables in the TSC and the LEOF during and following a design basis event. The operation of safety-related equipment or the availability of safety-related power sources is not affected. Therefore, an unreviewed safety question does not exist.

SE 94-021 Safety Evaluation 10-10-94

Safety Evaluation 94-021 was performed to evaluate a vendor nonconformance report regarding a tipover impact limiter for spent fuel storage casks. Specifically, the crush strength was determined to be slightly below the vendor specified limit for two sections of aluminum honeycomb for a CASTOR X/33 cask tipover impact limiter.

The evaluation concluded that the acceleration of a cask tipover event would actually be less using the lower crush strength sections and would thereby decrease the loads and stresses on the cask and fuel basket. Therefore, the tipover impact limiter remains capable of performing its intended function and an unreviewed safety question does not exist.

SE 94-031 Safety Evaluation 10-17-94

Safety Evaluation 94-031 was performed to evaluate a vendor nonconformance report regarding a spent fuel storage cask. Specifically, the magnesium content of the ductile cast iron for the CASTOR X/33 cask No. 001 was determined to be below the vendor specified limits.

The results of metallurgical and mechanical testing indicated that the cask annealing process was correct, that the magnesium difference had no effect on the metallographic formation of ductile cast iron, and that the mechanical properties of the material were not altered. Therefore, an unreviewed safety question does not exist.

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: October, 1994

SE 94-181 **Safety Evaluation** 10-25-94

Safety Evaluation 94-181 was performed to evaluate the temporary installation and operation of a skid mounted wet oxidation system at the Surry Radwaste Facility (SRF). The wet oxidation equipment will be used to process liquid waste containing ethylene diamine tetra-acetic (EDTA) acid resulting from the steam generator chemical cleaning process.

The evaluation concluded that the temporary installation and operation of a skid mounted wet oxidation system was acceptable. Adequate spill control measures will be established and the integrity of the SRF systems will be maintained. Therefore, an unreviewed safety question does not exist.

SE 94-183 **Safety Evaluation** 10-26-94

Safety Evaluation 94-183 was performed to evaluate the operation of the Unit 1 steam generators (SG) with tube support plate broached quatrefoil holes that are partially blocked by sludge deposits. The blockage causes hydrodynamic instability resulting in SG water level oscillations.

The condition exhibited by the Unit 1 SGs was determined to be bounded by the previous evaluations and analyses of a similar condition that was exhibited by the Unit 2 SGs before being chemically cleaned in June, 1994. The evaluation concluded that continued Unit 1 operation is acceptable until the SGs can be chemically cleaned provided that steady state reactor power remains at or above 85%. Therefore, an unreviewed safety question does not exist.

EWR 90-142 **Engineering Work Request** 10-26-94
(Safety Evaluation No. 91-085)

Engineering Work Request 90-142 replaced chilled water pumps 1-VS-P-2A, 1-VS-P-2B, and 1-VS-P-2C since they were approaching the end of their service life. Obtaining replacement parts had also become difficult.

The seismically qualified replacement pumps meet the system design flow requirements and will improve system availability. Therefore, an unreviewed safety question does not exist.

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: October, 1994

TM S2-94-15

Temporary Modification
(Safety Evaluation No. 94-185)

10-28-94

Temporary Modification S2-94-15 installed a temporary electrical jumper to bypass Cell 52 of the Main Station Battery 2A to allow the weak cell to be replaced.

The electrical jumper material and quality were equivalent to the existing battery cables and connections and was capable of carrying the load of the 2A DC bus. An evaluation of the impact of this activity on battery design performance was performed. The evaluation concluded that the battery would remain operable with the electrical jumper installed without any reduction in the Technical Specifications margin of safety. This activity was discussed with and agreed upon by the NRC during a conference call on October 28, 1994. Therefore, an unreviewed safety question did not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: October, 1994

1(2)-MOP-IA-001 **Maintenance Operating Procedures** 10-11-94
(Safety Evaluation No. 94-175)

Maintenance Operating Procedures 1(2)-MOP-IA-001, "Containment Instrument Air Compressors," were revised to provide instructions for lifting electrical leads to the low water level switch when a containment instrument air (IA) compressor is removed from service. By lifting the subject electrical leads, the associated control room annunciator remains functional for the containment IA compressor that remains in service.

The subject electrical leads will be landed before the respective containment IA compressor is returned to service. Implementation of this procedure will not change the normal operating configuration of the Containment IA System. Therefore, an unreviewed safety question does not exist.

1(2)-OP-CH-015 **Operating Procedures** 10-13-94
(Safety Evaluation No. 94-177)

Operating Procedures 1(2)-OP-CH-015, "Installation and Operation of the PDTT to CH System Jumper," were developed to provide instructions for the installation of a temporary mechanical jumper to connect the Gaseous Drains (DG) System to the Chemical and Volume Control System (CVCS).

The mechanical jumper will be installed when a unit is shut down in order to conserve primary system inventory and thereby reduce the volume of liquid waste. The jumper and associated connections will be leak tested before being placed in service. Instructions are also provided to prevent inadvertent Reactor Coolant System dilution and to ensure refueling containment integrity is maintained. Therefore, an unreviewed safety question does not exist.

2-TMOP-3029 **Temporary Maintenance Operating Procedure** 10-13-94
(Safety Evaluation No. 94-176)

Temporary Maintenance Operating Procedure 2-TMOP-3029, "Resin Removal of 2-CH-I-1A: Vacuum Method," was developed to provide instructions for cleaning any residual resin or foreign debris from the retention element of the Unit 2 "A" mixed bed demineralizer, 2-CH-I-1A.

The subject procedure will be implemented while the demineralizer is isolated from the in-service letdown train and within the ventilation boundaries of the Auxiliary Building filtered exhaust. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: October, 1994

1-TOP-4045 **Temporary Operating Procedure** 10-17-94
(Safety Evaluation No. 94-178)

Temporary Operating Procedure 1-TOP-4045, "Reactor Coolant Pump 1C Number 1 Seal Leakoff Bucket Check," was developed to provide instructions for verifying the accuracy of the low range flow instrumentation for Unit 1 reactor coolant pump (RCP) "C" seal leak-off flow.

Implementation of this procedure, which involves the collection and measurement of RCP seal leak-off flow, will not affect the reactor coolant system pressure boundary and will not reduce the margin of safety as described by the Technical Specifications. Therefore, an unreviewed safety question does not exist.

1-TOP-4046 **Temporary Operating Procedure** 10-19-94
(Safety Evaluation No. 94-179)

Temporary Operating Procedure 1-TOP-4046, "Reactor Coolant Pump 1C Number 2 Seal Leakoff Bucket Check," was developed to provide instructions for measuring the amount of water that is bypassing the Unit 1 reactor coolant pump (RCP) "C" No. 1 seal leak-off flow instrumentation.

Implementation of this procedure, which involves the collection and measurement of RCP No. 2 seal leak-off flow, will not affect the reactor coolant system pressure boundary and will not reduce the margin of safety as described by the Technical Specifications. Therefore, an unreviewed safety question does not exist.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: October, 1994

None During the Reporting Period

CHEMISTRY REPORT

MONTH/YEAR: October, 1994

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioactivity, $\mu\text{Ci/ml}$	3.23E-1	2.44E-1	2.79E-1	1.93E-1	8.01E-2	1.26E-1
Suspended Solids, ppm	≤ 0.1	≤ 0.1	≤ 0.1	≤ 0.1	≤ 0.1	≤ 0.1
Gross Tritium, $\mu\text{Ci/ml}$	2.95E-1	2.54E-1	2.77E-1	2.50E-1	2.10E-1	2.35E-1
I^{131} , $\mu\text{Ci/ml}$	4.79E-4	3.43E-4	4.23E-4	1.83E-4	2.02E-4	1.48E-4
I^{131}/I^{133}	0.08	0.06	0.07	0.10	0.06	0.08
Hydrogen, cc/kg	35.1	25.5	32.2	43.1	26.1	37.6
Lithium, ppm	2.28	2.09	2.17	2.27	1.83	2.07
Boron - 10, ppm*	155.2	140.9	148.4	65.3	44.3	53.5
Oxygen, (DO), ppm	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005
Chloride, ppm	0.011	0.002	0.004	0.010	0.001	0.003
pH at 25 degree Celsius	6.92	6.76	6.85	7.41	7.29	7.36

* Boron - 10 = Total Boron x 0.196

Comments:

None

**FUEL HANDLING
 UNITS 1 & 2**

MONTH/YEAR: October, 1994

New or Spent Fuel Shipment Number	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
Spent Fuel Storage Cask CASTOR V/21 500.11-023	10/12/94	21	N21	NOANSI	2.5560	N/A
			T05	LM02JW	3.1100	
			T06	LM02JV	3.1100	
			5R4	LM0C2W	3.6032	
			N27	NOANSI	2.5560	
			N28	NOANSI	2.5560	
			T20	LM02K0	3.1100	
			T21	LM02JE	3.1100	
			N29	NOANSI	2.5560	
			T22	LM02JF	3.1100	
			N30	NOANSI	2.5560	
			N22	NOANSI	2.5560	
			N31	NOANSI	2.5560	
			N32	NOANSI	2.5560	
			N23	NOANSI	2.5560	
			3R0	LM0C1V	5.5948	
			N24	NOANSI	2.5560	
			T04	LM02JK	3.1100	
			4R9	LM0C2F	3.5907	
			N25	NOANSI	2.5560	
			N26	NOANSI	2.5560	

**FUEL HANDLING
 UNITS 1 & 2**

MONTH/YEAR: October, 1994

New or Spent Fuel Shipment Number	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
New Fuel Unit 2 Batch 15 Receipt 1	10/18/94	8	2X7	LM0ZX8	3.8238	9.96 Ci
			2X0	LM0ZX1	3.8135	
			2X2	LM0ZX3	3.8145	
			2X1	LM0ZX2	3.8136	
			2X9	LM0ZXA	4.0113	
			2X8	LM0ZX9	3.8147	
			5X0	LM0ZXX	4.0231	
			4X9	LM0ZXW	4.0082	
Receipt 2	10/20/94	12	5X5	LM0ZY2	4.0186	14.73 Ci
			2X6	LM0ZX7	3.8149	
			1X2	LM0ZWT	3.8060	
			2X5	LM0ZX6	3.8257	
			4X7	LM0ZXU	4.0097	
			0X1	LM0ZWG	3.8162	
			5X3	LM0ZY0	4.0026	
			5X4	LM0ZY1	4.0281	
			1X9	LM0ZX0	3.8176	
			1X4	LM0ZVV	3.8153	
			2X3	LM0ZX4	3.8097	
			1X3	LM0ZVU	3.8184	

**FUEL HANDLING
 UNITS 1 & 2**

MONTH/YEAR: October, 1994

New or Spent Fuel Shipment Number	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
New Fuel Unit 2 Batch 15 Receipt 3	10/25/94	12	2X4	LM0ZX5	3.8113	14.92 Ci
			0x9	LM0ZWQ	3.8186	
			4x0	LM0ZXM	4.0112	
			3X9	LM0ZXL	4.0118	
			4X6	LM0ZXT	4.0207	
			4X8	LM0ZXV	3.9990	
			3X2	LM0ZXD	4.0108	
			0X7	LM0ZWN	3.8095	
			1X8	LM0ZWZ	3.8118	
			0X5	LM0ZWL	3.8132	
			3X3	LM0ZXE	4.0098	
			5X1	LM0ZXY	4.0309	
Receipt 4	10/27/94	12	5X6	LM0ZY3	4.0252	14.85 Ci
			3X6	LM0ZXH	4.0044	
			4X4	LM0ZXR	4.0079	
			1X1	LM0ZWS	3.8075	
			0X4	LM0ZWK	3.8117	
			5X2	LM0ZXZ	4.0012	
			1X7	LM0ZWY	3.8105	
			1X5	LM0ZWW	3.8166	
			1X6	LM0ZWX	3.8024	
3X0	LM0ZXB	4.0068				

**FUEL HANDLING
UNITS 1 & 2**

MONTH/YEAR: October, 1994

New or Spent Fuel Shipment Number	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
New Fuel Unit 2 Batch 15 Receipt 4 (Continued)			0X2	LM0ZWH	3.8136	
			3X5	LM0ZWG	4.0109	

**DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS**

MONTH/YEAR: October, 1994

<u>Number</u>	<u>Title</u>	<u>LER Number</u>
0-EPT-0109	Weekly Emergency Diesel Generator Battery Pilot Cell and Bus Voltage Checks	S1-94-010-00