UNITED STATES NUCLEAR REGULATORY COMMISSION **REGION II** 101 MARIETTA STREET, N.W., SUITE 2900 ATLANTA, GEORGIA 30323-0199 Report Nos.: 50-280/94-16 and 50-281/94-16 Licensee: Virginia Electric and Power Company Glen Allen, VA 23060 License Nos.: DPR-32 and DPR-37 Docket Nos.: 50-280 and 50-281 Facility Name: Surry Power Station Units 1 and 2 Inspection Conducted June 20-24, 1994 Inspector: Kleinsbrge P.E. Reactor Inspector Approved by: Jerome J. Blake, Chief Date Signed Materials and Process Section Enginéering Branch Division of Reactor Safety

SUMMARY

Scope:

This routine, announced inspection was conducted in the areas of Chemically Enhanced Pressure Pulse Cleaning (PP/CC) treatment of the three Unit 2 Steam Generators (S/G)s, feedwater system leak, and flow accelerated corrosion.

Results:

Interviews, observations and records review indicated that the Unit 2 S/G PP/CC project was well planned and executed, and appeared to be effective. With the exception of the weakness noted concerning the accuracy of weld documentation, the licensee's actions related to the feedwater line failure were appropriate to the circumstances. The licensee has established an effective program to maintain high energy carbon steel piping systems within acceptable wall thickness limits.

In the areas inspected, no violations or deviations were identified.

REPORT DETAILS

Persons Contacted 1.

Licensee Employees

W. Benthall, Supervisor, Licensing

*B. Graber, Licensing *D. Hanson, Maintenance

*L. Moris, Radiation Protection

*J. Price, Assistant Station Manager

*E. Turko, Engineering

Other licensee employees contacted during this inspection included engineers, operators, technicians, and administrative personnel.

NRC Resident Inspectors

*M. Branch, Senior Resident Inspector S. Tingen, Resident Inspector

*Attended exit interview

Acronyms and initialisms used throughout this report are listed in the last paragraph.

Steam Generator (S/G) Cleaning (92903)

Background

2.

In May 1993, Unit 2 noted small oscillations in "C" steam generator water level. By August 1993, the "C" S/G water level oscillations had enlarged to 15% to 20%. Investigation indicated that the "C" S/G level water level oscillations were independent of feedwater flow rate. It was determined that the water level oscillations were the result of density wave (hydrodynamic) instability, the root cause of which was blockage of flow area in the Tube Support Plates (TSPs), in particular the upper TSPs. In order to stabilize plant operation, the licensee incrementally decreased reactor power. By early November 1993, reactor power had dropped to 96%.

During a forced outage in November 1993 (trip caused by a loss of power, to all three feedwater regulation valves), Westinghouse Company (W) performed a pressure pulse cleaning treatment on "C" S/G. The cleaning removed a certain amount of the blockage, but significant blockage remained. Subsequent to the outage the unit was stable at full power. S/G water level oscillations of 10% soon returned and reactor power was ramped back in 1% to 2% steps.

It was evident that the pressure pulse cleaning treatment, performed by \underline{W} , had not adequately corrected the S/G water level oscillations. In late February 1994, the licensee started vender interviews, planning for a more aggressive cleaning process. By late March 1994, the licensee decided on Chemically Enhanced Pressure Pulse Cleaning (PP/CC) treatment

for the three Unit 2 S/Gs. PP/CC is a process in which chemicals are selectively introduced into the S/Gs at varying temperatures, pH levels, and concentrations to remove iron and copper. Due to system load demands, the licensee elected to perform PP/CC in June 1994. The licensee solicited bids for PP/CC in March-April 1994. The Unit 2 PP/CC outage commenced June 3, 1994.

 \underline{W} was selected to perform the PP/CC as the prime-contractor (the project manager and the provider of corrosion monitoring). The subcontractors were Vectra (provider of personnel and the majority of the equipment) and DIS (provider of some equipment, chemicals and the chemical laboratory facility). The project was conducted under the umbrella of the \underline{W} Quality Assurance Program.

The <u>W</u> PP/CC process uses the EPRI/SGOG (described in EPRI NP-6354-M, Qualification of PWR Steam Generator Chemical Cleaning for Indian Point-2, May 1989) chemical cleaning solvents at low temperature combined with pressure pulse cleaning to remove sludge buildup at the TSP elevations. The main chemical ingredient used in PP/CC operation was the chelating agent ethylene diamine tetra-acetic acid (EDTA). Other major ingredients included corrosion inhibitors, ammonia, oxidizing agents, and hydrazine. The PP/CC process included real-time insitu corrosion monitoring of the S/Gs. The effectiveness of the PP/CC process was monitored by inprocess chemical analysis of the cleaning solutions and selected remote visual examinations of the S/G internals.

Inspection

To evaluate the PP/CC process in the areas of process control, personnel qualification, and protection of safety related equipment, the inspector conducted interviews with licensee and contractor personnel; examined selected records and video tape; and observed work activities.

Identification	Revision	Title
2-SSS 2.13.2- VIR-01 6/1/94	0	Master Governing Procedure for Chemi- cally Enhanced Pressure Pulse Cleaning of Surry-2 Steam Generators
0-SSS 2.13.2- VIR-02 6/1/94	0	Steam Generator Equipment and Pressure Pulse Operation for Chemical Cleaning
2-SSS 2.13.2- VIR-03 6/1/94	0	Sampling and Analysis Procedure for Surry-2 Steam Generator Chemically Enhanced Pressure Pulse Cleaning
2-SSS 2.13.2- VIR-04 6/1/94	0	Process Application and Termination for Surry-2 Steam Generator Chemically Enhanced Pressure Pulse Cleaning

Documents Reviewed

Documents Reviewed

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Identification	Revision	Title
2-SSS 2.13.2- VIR-05 6/1/94	0	Set-Up and Operation of Corrosion Mon- itoring Equipment for Chemically En- hanced Pressure Pulse Cleaning
SECL-94-091 6/1/94	-	Safety Evaluation for: Surry Unit 2 Steam Generator Chemically Enhanced Pressure Pulse Cleaning
NSD-JLH-4028 SG-94-01-026 1/18/94	<u>-</u>	Analysis of Performance and Hydrodynamic Stability for Water Level Oscillation in Surry Unit 2 Steam Gen- erators
<u>W</u> STD-OP-1994- 6823 5/27/94	0	Corrosion Monitoring Application Code and System Qualification Procedure

In addition to the documents indicated above the inspector examined documentation attesting to and certifying personnel qualifications and equipment calibration.

The inspector viewed a video tape of remote visual examinations of the internals of the "C" S/G, depicting before and after views of the top of the seventh TSP. Virtually all the deposits had been removed from the surfaces of the TSPs, the quatrefoil flow passages, and the tube surfaces visible in the limited field of view.

A significant quantity of material was removed from the generators, which is indicated below.

· · · ·	Copper (Lbs.)	Iron (Lbs.)	Sludge (Lbs.)
S/G A	350	2600	1258
S/G B	470	3050	984
S/G C	500	3900	610
Total	1,320	9,550	2,852

Materials Removed From Unit 2 S/G

The inspector conducted a walk-through inspection of the chemical handling facilities used for the PP/CC. The inspector noted the dikes erected to contain spills and to prevent the commingling of chemicals with the potential of synergistic reactions. The inspector observed deactivation activities.

Interviews, observations and records review indicated that the Unit 2 PP/CC project was well planned and executed, and appeared to be effective.

Within the areas examined, no violations or deviations were identified.

3. Feedwater System Pipe Leak (92903)

Background

The licensee noted water on the floor adjacent to the 2A Feedwater heater. On May 25, 1994, after isolation and cool down of the 2A Feedwater heater, the insulation was removed exposing a through-wall hole in heater drain line 10"-WLD-101-301, at a 10" x 14" expander. The hole was located at the 12:00 position in the heat affected zone of the weld connecting the horizontal expander to a down directed 10" elbow (see Figure 1). The licensee cut out the carbon steel expander and elbow and replaced the fittings with low alloy steel, 2¹/₄Cr 1Mo, fittings. At this writing, the licensee is conducting a metallurgical failure analysis.

In 1992, prior to the implementation of the CHECMATE® program, the licensee ultrasonically examined the elbow and expander on the Unit 1 "B" train line (in the same relative position as above). The minimum wall thickness value found, though acceptable then, would have been unacceptable prior to the end of the next fuel cycle. The maximum point of wear was approximately 16" to 20" down the extrados of the elbow at the 12:00 position (see Figure 1).





The licensee expanded the sample and examined the "A" train elbow and expander. They were found to be acceptable for continued service. The

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Unit 1 "B" train elbow was replaced with a low alloy steel, $2\frac{1}{4}$ Cr 1Mo, fitting. The Unit 1"B" train expander remains the original carbon steel.

The elbow and reducer in the Unit 2 "B" train was ultrasonically examined in June 1994, and found to be acceptable for continued service for several cycles. The licensee indicated that they intend to replace both "B" train fittings at the next Unit 2 refueling outage, when they replace the Unit 2 feedwater heaters.

Inspection

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To evaluate the licensee's actions relative to the balance of plant nonsafety related feedwater system leak, the inspector conducted interviews with licensee personnel; examined the failure sites; examined the cut out elbow and expander; and examined selected records.

The inspector examined welder qualification and certification documentation; Quality Control (QC) inspector qualification, certification, and visual acuity documentation; welding filler material certified material test reports (CMTRs); and welding procedure specifications (WPSs) and their associated procedure qualification records (PQRs); involved with the replacement of the failed Unit 2, carbon steel, 10" x 14" expander and the worn, 10" elbow with $2\frac{1}{4}$ Cr 1Mo low alloy steel fittings. The documents were examined for compliance with licensee procedures and Sections II and IX of the ASME B&PV Code.

Туре	Size	Heat/Lot No.
ER-805 B2	1/8"	F5409
ER-80S B2	3/32"	F2874
E-8018-B2	1/8"	412W1371
E-8018-B2	3/32"	26847

Welding Filler Material CMTRs Examined

As a result of the review of documentation for the welding filler materials, discrepancies were identified on "Weld Material Field Control" forms issued by the licensee for welding filler material control.

- On "Weld Material Field Control" form Serial No.038590, the Heat and Purchase Order Numbers for electrode type E-7018 (minimum tensile strength 70 ksi carbon steel coated electrode) filler material was annotated when type E-8018-B2 (80 ksi 1‡Cr-1Mo steel coated electrode) was issued and used.
- Electrode type E-8018 (minimum tensile strength 80 ksi carbon steel coated electrode) was annotated on several "Weld Material Field Control" forms, when Type E-8018-B2 (80 ksi 1½Cr-1Mo steel coated electrode) was intended.

Type ER-8018 (unknown type) was annotated on one "Weld Material Field Control" form, when Type ER-80S-B2 (80 ksi 1‡Cr-1Mo steel bare rod) was intended.

The licensee documented the incorrect Heat and Purchase Order Numbers on a Station Deviation Report (DR) dated June 23, 1994 (DR number was not available at this writing). the licensee indicated that they would add the discrepant material types to that DR or institute a new DR. The licensee conducted an immediate investigation into this matter. The licensee's preliminary results indicated that welding filler materials used were consistent with the materials specified in the applicable WPS. The errors were only in the documentation and did not affect plant equipment.

The above discrepancies demonstrate a weakness concerning the accuracy of welding documentation. This is of concern because the same welding filler material issue stations, personnel and procedures used for this non-safety related, balance-of-plant job, are employed for plant safety related work.

Welder's Qualification Certification Documentation Examined

GM 7784, KS 5016, DB 6994, KS 4632, and RT 0360

QC Inspector Certification and Documentation Examined

GH MT-II, GAM Visual Weld, SRW Visual Weld, and CWM Mechanical

WPSs Examined

WPS	PQR		
WPS-508, Revision F 9/91	505, Revision A, 2/2/82 506, Revision A, 10/30/82 532, 8/27/90 533, 9/9/91 534, 9/9/91		

The WPS examined above was properly supported by PQRs. The welds were made by properly qualified and certified welders, using appropriate welding materials in accordance with properly qualified welding procedure specifications, and inspected by properly qualified and certified inspectors/examiners.

With the exception of the weakness noted concerning the accuracy of weld documentation, the licensee's actions related to the feedwater line failure were appropriate to the circumstances.

Within the areas examined, no violations or deviations were identified.



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Flow Accelerated Corrosion (FAC) (49001)

The licensee has established a FAC inspection program which implements the CHECMATE® EPRI (Electric Power Research Institute) computer code, industry experience, and previous inspection data as predictive tools for determining and prioritizing inspection locations. The inspector conducted interviews with licensee personnel and reviewed records as indicated below.

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Observations/Findings

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During Unit 1 Refueling Outage RFO 13, the licensee planned to examine 121 locations in their Unit 1 FAC program. The licensee expanded the sample size to 137 locations and identified 16 FAC degraded components: 13 were replaced and three were repaired.

The inspector examined the engineering evaluations for the 13 replaced components. The observations were compared with the applicable procedures.

FAC Engineering Evaluations Examined

Component Identification	Component Type	System	Disposition
1-SD-PPS-598	6" Pipe Sch. XS	Steam Drain	Replaced
1-SD-PSF2-297	6" 90° Elbow Sch. 80	Steam Drain	Replaced
1-SD-PSF1-44	6" 45° Elbow Sch. XS	Steam Drain	Replaced
1-SD-PPS- 597	6" Pipe Sch. XS	Steam Drain .	Replaced
1-SD-PSF2-282	6" 90° Elbow Sch. STD	Steam Drain	Replaced
1-SD-PSF1-45	6" 45° Elbow Sch. 80	Steam Drain	Replaced
1-SD-PSF2-299	6" 90° Elbow Sch. XS	Steam Drain	Replaced
1-SD-PFS2-284	6" 90° Elbow Sch. 80	Steam Drain	Replaced
1-FW-PFS2-41	8" 90° Elbow Sch. 100	Feedwater	Replaced
1-FW-PPS-27	8" Pipe Sch. 100	Feedwater	Replaced
1-FW-PSF2-44	8" 90° Elbow Sch. 100	Feedwater	Replaced



Component Identification	Component Type	System	Disposition
1-CN-PFS2-122	18" 90° Elbow Sch. XS	Condensate	Replaced
1-CN-PPS-118	18" Pipe Sch. XS	Condensate	Replaced

The licensee is in the process of converting their CHECMATE® data to CHECWORKS® which operates in the Microsoft Windows® environment.

Notwithstanding the feedwater system leak discussed above, the licensee has established an effective program to maintain high energy carbon steel piping systems within acceptable wall thickness limits.

Within the areas examined, no violations or deviations were identified.

5. Exit Interview

The inspection scope and results were summarized on June 24, 1993, with those persons indicated in paragraph 1. The inspector described the areas inspected. Although reviewed during this inspection, proprietary information is not contained in this report. Dissenting comments were not received from the licensee.

6. Acronyms and Initialisms

B&PV-Boiler and Pressure VesselCMTR-Certified Material Test ReportCN-CondensateCr-ChromiumDPR-Demonstration Power ReactorDR-Station Deviation ReportEDTA-Ethylene Diamine Tetra-Acetic AcidEPRI-Electric Power Research InstituteFAC-Flow Accelerated CorrosionFW-Feedwaterksi-103 Lbs. per square inchLbsPoundsMo-NumberNRC-NumberP.EProfessional EngineerPQR-Ouality Assurance	ers
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CN - Condensate Cr - Chromium DPR - Demonstration Power Reactor DR - Station Deviation Report EDTA - Ethylene Diamine Tetra-Acetic Acid EPRI - Electric Power Research Institute FAC - Flow Accelerated Corrosion FW - Feedwater ksi - 10 ³ Lbs. per square inch Lbs Pounds Mo - Molybdenum No Number NRC - Nuclear Regulatory Commission P.E Professional Engineer PP/CC - Chemically Enhanced Power Pulse Clean PQR - Procedure Qualification Record QA - Quality Assurance	
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QC - Quality Control	
Rev. – Revision	•



RFO Refueling Outage Steam Generator S/G Sch. Schedule SD Steam Drain Steam Generator Owners Group SGOG _ STD Standard **-** . Tube Support Plate TSP VA Virginia Westinghouse Company W -₩̈́PS Welding Procedure Specification -XS Extra Strong _

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