

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

June 2, 1994

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

Serial No. 94-324
SPS/ETS
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen,

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
INSERVICE TESTING PROGRAM
ASME SECTION XI INSTRUMENTATION

By NRC letter dated June 26, 1992, (TAC Nos. M80297 and M80298) Surry Power Station was granted interim relief from the requirements of Section XI of the ASME Boiler and Pressure Vessel Code for certain components in the service water side of the Main Control Room and Emergency Switchgear Room Air Conditioning System. The basis for Relief Requests P-16 and V-46, in the Unit 1 Inservice Testing Program, was that flow and inlet pressure instrumentation needed to be purchased and installed before the Code requirements could be met. These instruments were to be purchased and installed by the second quarter of 1994 at which point compliance with the Code would be achieved.

Flow and inlet pressure instrumentation were installed during the recent Unit 1 refueling outage. However, extensive preservice testing of the flow instrumentation has revealed that the instruments are not reading the full system flow. The problem appears to be associated with instrument configuration and highly silted service water. The preservice testing of the newly installed suction pressure gauges has produced erratic results. Additional evaluation is necessary to determine if either the the erratic pressure readings can be eliminated or the configuration must be redesigned.

An engineering effort has been initiated to redesign the flow instrumentation and to evaluate and possibly redesign the suction pressure gauge configuration. The redesign of the flow instrument is planned to be completed and installed by the end of the next Surry Unit 1 outage. The evaluation and possible redesign and installation, if necessary, of the suction pressure configuration will also be completed by the end of the next Surry Unit 1 outage. In the interim, Surry Power Station requests that the alternate testing, identified in Relief Requests P-16 and V-46, be extended through the end of the next scheduled Surry Unit 1 refueling outage, currently scheduled for the third quarter of 1995. The alternate testing includes pump and condenser differential

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pressure for flow measurement and disassembly and inspection of the valves.

If you have any questions, please contact us.

Very truly yours,



James P. O'Hanlon
Senior Vice President-Nuclear

cc: U.S. Nuclear Regulatory Commission
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Mr. Morris Branch
NRC Senior Resident Inspector
Surry Power Station