



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

March 16, 1994

Docket No. 50-280

Mr. W. L. Stewart  
Senior Vice President - Nuclear  
Virginia Electric and Power Company  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: SURRY POWER STATION, UNIT 1 - INTERIM RELIEF OF THE THIRD 10-YEAR  
INSERVICE INSPECTION PROGRAM (TAC NO. M87312)

The NRC staff has performed a preliminary review of relief requests RR-10 and RR-11 in the Surry, Unit No. 1 Inservice Inspection (ISI) Program for the third 10-year interval. You submitted this program by letter dated July 16, 1993. This correspondence addresses only two of the relief requests, RR-10 and RR-11, in the July 16, 1993, submittal. Relief request RR-10, Part 1, requests relief from the 1989 Edition of the ASME Code IWA-5250(a)(2) which states: "if leakage occurs at a bolted connection, the bolting shall be removed, VT-3 visually examined for corrosion, and evaluated in accordance with IWA-3100." The proposed alternative selects only one bolt initially, which is the same as the Code requirements in the 1992 Edition of the ASME Section XI, IWA-5250(a)(2), and if the selected bolt has evidence of degradation, then all other bolting in the connection will be removed and visually (VT-3) examined and evaluated to the Code requirements. Relief request RR-11 requests relief from Subparagraph IWA-5242(a) of the 1989 Edition of the ASME Code which requires, for systems bolated for the purpose of controlling reactivity, removal of insulation from pressure-retaining bolted connections for visual examination VT-2 conducted in conjunction with system leakage tests performed prior to plant startup.

We have not yet completed our final review nor issued our final Safety Evaluation (SE). However, based upon our review of relief request RR-11, we have determined that certain Code requirements are impractical and would be a burden on the licensee; therefore, interim relief for relief request RR-11 is granted in accordance with 10 CFR 50.55a(g)(6)(i). Relief request RR-11 is granted for the systems inside the containment with the following conditions: for the system pressure test associated with plant startup, the system shall be held at nominal operating pressure for at least 4 hours for insulated systems and 10 minutes for noninsulated systems before performing the VT-2 visual examination.

With respect to relief request RR-10, Part 1, which contains a proposed alternative examination, we have determined, for the interim, in accordance with 10 CFR 50.55a(a)(3)(i), that the alternative is acceptable. We are continuing our review of the proposed ISI program including relief request

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RR-10, Part 2. The approval of the relief requests is based upon the fact that this interim relief extends a relief formerly given and extending this relief does not increase the likelihood of an accident. Further, the relief requested is similar to other reliefs granted for similar conditions. We have concluded that during the interim period there is reasonable assurance that the proposed alternatives would provide an acceptable level of quality and safety. Since the interim approval does not represent the results of the final program review, the final SE could contain relief request denials. Therefore, you are authorized to implement your proposed program for relief requests RR-10, Part 1, and RR-11 as described in the above-cited submittal.

Until we complete our detailed review of the ISI program, you must comply with both the existing Technical Specifications (TS) and the proposed ISI program. Thus, the granting of this relief from the ASME Code does not relieve you from any of the requirements in the existing TS.

When our detailed review of your submittal is complete, we will issue a final SE of your program. Completion of this review may require additional information which will be requested by separate correspondence. Our SE may contain modifications resulting from the staff's review or may grant relief from any ASME Code requirements that are determined to be impractical for the facility for the duration of the third 10-year testing interval. We anticipate issuing the final SE by early May 1994.

The staff has determined that pursuant to 10 CFR 50.55a(g)(6)(i) approval of the interim relief for relief request RR-11 is authorized by law and will not endanger life or property or the common defense and security. We also find that the proposed alternative provides an acceptable level of quality and safety on an interim basis; therefore, pursuant to 10 CFR 50.55a(a)(3)(i) the proposed alternative contained in the request for relief request RR-10, Part 1 is authorized.

The staff has also concluded that granting the approvals is otherwise in the public interest considering the burden that could result if the requirements were imposed on the facility. The interim relief will terminate upon issuance of our final SE.

Sincerely,

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(Original Signed By)

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 Office of Nuclear Reactor Regulation

cc: See next page

\* See previous concurrence

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