10CFR50.73

Virginia Electric and Power Company
Surry Power Station
5570 Hog Island Road
Surry, Virginia 23883

October 18, 1995

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Serial No.: 95-545
SPS:JDK
Docket No.: 50-280
License No.:DPR-37

Dear Sirs:

Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to Surry Power Station Unit 1.

REPORT NUMBER

50-280/95-008-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,

David A Christian

D. A. Christian
Station Manager

Enclosure

cc: Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

M. W. Branch
NRC Senior Resident Inspector
Surry Power Station

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NRC FORM 366 (5-92) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
SURRY POWER STATION, Unit 1	05000 - 280	1 OF 4

TITLE (4)
Pressurizer Safety Valve As Found Setpoint Out Of Tolerance

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER 05000 -
09	19	95	95	-- 008 --	00	10	17	95	FACILITY NAME	DOCKET NUMBER 05000 -

		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR:(Check one or more) (11)							
OPERATING MODE (9)	N		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(c)
POWER LEVEL (10)	0%		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER
			20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Including Area Code)
D. A. Christian, Station Manager	(804) 357-3184

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	AB	RV	C710	Y						

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On September 19, 1995 with Unit 1 shutdown for a refueling outage, a Station Deviation Report was submitted when a Pressurizer Safety Valve (PSV) As Found lift setting was 3.94% above the nominal value of 2485 psig. Technical Specification (TS) 3.1.A.3.b specifies the As Found tolerance should not exceed +/- 3% from 2485 psig. The PSV was adjusted and three acceptable lifts were achieved within the +/- 1% As Left tolerance required by TS.

The As Found setting was evaluated by Engineering. The evaluation determined the PSVs were capable of performing their safety function during operation and did not violate any accident analysis. Therefore, the health and safety of the public were not affected.

This event is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B)for a condition in excess of TS 3.1.A.3.b.

1.0 DESCRIPTION OF THE EVENT

On September 7, 1995 , Unit 1 was shut down to start a refueling outage. As part of the outage scope the three Pressurizer Safety Valves (PSVs) [EIIS-AB-RV] were removed and sent to Wyle Laboratories in Huntsville, Alabama for setpoint testing. The PSVs were tested in accordance with station testing procedures developed pursuant to the Surry Power Station ASME Section XI Testing Program for Pumps and Valves.

On September 19, 1995, a Deviation Report (DR) was submitted when PSV 1-RC-SV-1551C As Found lift setting was determined to be out of tolerance. The As Found lift setting was above the maximum value allowed by Technical Specification (TS) 3.1.A.3.b., i.e., 2485 psig +/- 3%. The valve exhibited a lift setting of 2583 psig which is 3.94% above the nominal value. The PSV was adjusted and three acceptable lifts were achieved within the TS required +/- 1% As Left setpoint.

The As Found setting was evaluated by Engineering. The evaluation determined that the PSVs were capable of performing their safety function during operation and they did not violate any accident analysis. Therefore, the health and safety of the public were not affected.

This event is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B) for a condition in excess of TS 3.1.A.3.b.

2.0 SIGNIFICANT SAFETY CONSEQUENCES AND IMPLICATIONS

The PSVs provide overpressure protection for the Reactor Coolant System. The PSV's lift pressure settings are bounded by the safety analysis for overpressure transients. The As Found setting was evaluated by Engineering. Engineering concluded that the measured pressurizer safety valve lift setpoints provided effective overpressure protection. Their review also concluded that the analysis for the overpressure transients remained bounding during operations.

The test data for pre-installation and the test data for the first operating cycle of the other two PSVs on Unit 1 were reviewed. The results of the review determined that no problems exisited with the As Found test data and no increasing or decreasing trends were identified with the As Left test data.

In Summary, these evaluations concluded that the PSVs would perform their design function and that the health and safety of the public were not affected.

3.0 CAUSE OF THE EVENT

A Category 2 Root Cause Evaluation was performed as a result of this event. Root Cause personnel reviewed pre-installation lift setting test data, the valve manufacturer's testing program, and Virginia Power's Pump and Valve testing program.

The PSVs were new when installed prior to the start of the most recent operating cycle. The valves were tested as part of the manufacturing process utilizing the manufacturer's procedures. Engineering performed a review of the pre-installation lift setting test data. During the review process, it was noted that the PSV 1-RC-SV-1551C's test data showed an increasing trend. As part of this review, it was determined that although the data showed an increasing trend, the lifts were within the As Left tolerance of +/- 1% required by Technical Specification (TS) 3.1.A.3 and the testing of the PSVs by the manufacturer had been performed in full compliance with ASME Code.

The Category 2 Root Cause Evaluation determined that even though the testing of the PSVs by the manufacturer was performed in full compliance with ASME Code, the increasing trend for 1-RC-SV-1551C indicated that the valve setting may not have stabilized. The cause of the event has been attributed to the test methodology used by the valve manufacturer to test the PSVs prior to installation.

4.0 IMMEDIATE CORRECTIVE ACTION(S)

The Unit condition at time of discovery did not require any immediate corrective actions. A Deviation Report was submitted to document the out-of-specification As Found setting of the valve.

5.0 ADDITIONAL CORRECTIVE ACTION(S)

An adjustment was made to lower the lift setting of 1-RC-SV-1551C. After the adjustment, three acceptable lifts were achieved within the As Left +/- 1% required by Technical Specification 3.1.A.3.b.

Pre-installation test data for the other two PSVs on Unit 1 was reviewed and did not indicate any increasing or decreasing trends. Additionally, the Unit 1 PSV first operating cycle test data was reviewed. The results of the review determined that no problems exisited with the As Found test data and no increasing or decreasing trends were identified with the As Left test data.

6.0 ACTIONS TO PREVENT RECURRENCE

The valve manufacturer has stated that evaluation of increasing/decreasing trends was a new guideline they have recently implemented as part of their ASME Code Testing Program.

Currently, Virginia Power sends the PSVs to Wyle Laboratories in Huntsville, Alabama. Wyle Laboratories uses Virginia Power's "Pressurizer Safety Valve Setpoint Test" procedure for the testing of our PSVs. This procedure is based on the ASME Section XI code. Additionally, Wyle Laboratories records all lift settings for a PSV in Virginia Power's procedure. As a good practice, they review this data, looking for an increasing or decreasing trend. This information is also reviewed by a Virginia Power representative.

7.0 SIMILAR EVENTS

Unit 2 LER 89-013-00, Pressurizer Safety Valves Setpoints Outside Technical Specification Allowable Limits Due to Establishing Setpoints Without Loop Seal

Unit 2 LER 91-003-00, Pressurizer Safety Valves Setpoints Outside Technical Specification Allowable Values

Unit 2 LER 2-95-002-00, Main Steam Safety And Pressurizer Safety Valves As Found Setpoint Out Of Tolerance

8.0 Additional Information

Unit 2 was at 100% power during this event. The test data for pre-installation and the test data for the first operating cycle of the three PSVs on Unit 2 were reviewed. The results of the review determined that no problems exisited with the As Found test data and no increasing or decreasing trends were identified with the As Left test data.

The TS provision allowing a +/- 3% As Found setting tolerance was incorporated into Surry TS by Amendment Number 203, dated August 3, 1995. Prior to this amendment the As Found and As Left setting tolerance was +/- 1%.