

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

November 14, 1996

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Serial No. 96-581
NL/RPC
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

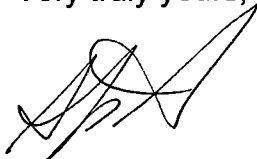
Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of October 1996.

If you have any questions or require additional information, please contact us.

Very truly yours,



S. P. Sarver, Acting Manager
Nuclear Licensing and Operations Support

Enclosure

cc: U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, N.W.
Suite 2900
Atlanta, Georgia 30323

Mr. R. A. Musser
NRC Senior Resident Inspector
Surry Power Station

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IE24/1

**VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION
MONTHLY OPERATING REPORT
REPORT NO. 96-10**

Approved:


Station Manager

11-12-96
Date

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OPERATING DATA REPORT

Docket No.: 50-280
 Date: 10-04-96
 Completed By: D. K. Mason
 Telephone: (804) 365-2459

1. Unit Name: Surry Unit 1
2. Reporting Period: October, 1996
3. Licensed Thermal Power (MWt): 2546
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): ... 840
7. Maximum Dependable Capacity (Net MWe): 801

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	<u>This Month</u>	<u>YTD</u>	<u>Cumulative</u>
11. Hours In Reporting Period.....	745.0	7320.0	209160.0
12. Number of Hours Reactor Was Critical	745.0	7320.0	145370.7
13. Reactor Reserve Shutdown Hours.....	0.0	0.0	3774.5
14. Hours Generator On-Line.....	745.0	7320.0	143067.0
15. Unit Reserve Shutdown Hours	0.0	0.0	3736.2
16. Gross Thermal Energy Generated (MWH)	1884854.8	18512995.8	334910788.3
17. Gross Electrical Energy Generated (MWH).....	630595.0	6149345.0	109726528.0
18. Net Electrical Energy Generated (MWH).....	608834.0	5935603.0	104391576.0
19. Unit Service Factor.....	100.0%	100.0%	68.4%
20. Unit Availability Factor.....	100.0%	100.0%	70.2%
21. Unit Capacity Factor (Using MDC Net).....	102.0%	101.2%	64.2%
22. Unit Capacity Factor (Using DER Net)	103.7%	102.9%	63.3%
23. Unit Forced Outage Rate	0.0%	0.0%	15.4%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
 Refueling, February 20, 1997, 37 Days

25. If Shut Down at End of Report Period, Estimated Date of Start-up: _____ N/A

26. Unit In Test Status (Prior to Commercial Operation):

	<u>FORECAST</u>	<u>ACHIEVED</u>
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATING DATA REPORT

Docket No.: 50-281
 Date: 10-04-96
 Completed By: D. K. Mason
 Telephone: (804) 365-2459

- 1. Unit Name: Surry Unit 2
- 2. Reporting Period: October, 1996
- 3. Licensed Thermal Power (MWt): 2546
- 4. Nameplate Rating (Gross MWe): 847.5
- 5. Design Electrical Rating (Net MWe): 788
- 6. Maximum Dependable Capacity (Gross MWe): ... 840
- 7. Maximum Dependable Capacity (Net MWe): 801

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting Period.....	745.0	7320.0	206040.0
12. Number of Hours Reactor Was Critical	745.0	6350.2	141853.1
13. Reactor Reserve Shutdown Hours.....	0.0	0.0	328.1
14. Hours Generator On-Line.....	745.0	6328.1	139884.2
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1896586.7	15899950.8	328435004.8
17. Gross Electrical Energy Generated (MWH).....	636535.0	5274405.0	107430049.0
18. Net Electrical Energy Generated (MWH).....	614330.0	5095805.0	102206220.0
19. Unit Service Factor.....	100.0%	86.4%	67.9%
20. Unit Availability Factor.....	100.0%	86.4%	67.9%
21. Unit Capacity Factor (Using MDC Net).....	102.9%	86.9%	63.5%
22. Unit Capacity Factor (Using DER Net)	104.6%	88.3%	63.0%
23. Unit Forced Outage Rate	0.0%	2.0%	12.6%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
 Maintenance, December 6, 1996, 10 Days

25. If Shut Down at End of Report Period, Estimated Date of Start-up: _____ N/A

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

**UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)**

REPORT MONTH: October, 1996

Docket No.: 50-280
 Unit Name: Surry Unit 1
 Date: 11-01-96
 Completed by: M. J. Fanguy
 Telephone: (804) 365-2155

(1) Date	(1) Type	(2) Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	LER No.	(4) System Code	(5) Component Code	Cause & Corrective Action to Prevent Recurrence
961010	S	N/A	B	N/A	N/A	TA	V	Reactor power was reduced to perform Operations Surveillance Procedure 1-OSP-TM-001, Turbine Inlet Valve Freedom Test.
961031	F	N/A	A	N/A	N/A	SJ	TBG	Reactor power was reduced to replace a leaking oil pump hose on the 1-FW-P-1A feedwater pump.

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source

**UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)**

REPORT MONTH: October, 1996

Docket No.: 50-281
 Unit Name: Surry Unit 2
 Date: 11-01-96
 Completed by: M. J. Fanguy
 Telephone: (804) 365-2155

(1)	(2)	(3)	(4)	(5)
Date	Type	Duration Hours	Reason	Method of Shutting Down Rx
			LER No.	System Code
			Component Code	Cause & Corrective Action to Prevent Recurrence

None During the Reporting Period

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets
 for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-280
 Unit Name: Surry Unit 1
 Date: 11-04-96
 Completed by: B. C. Bryant
 Telephone: (804) 365-2786

MONTH: October, 1996

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	818	17	822
2	819	18	821
3	819	19	822
4	821	20	823
5	822	21	821
6	822	22	823
7	822	23	823
8	823	24	822
9	822	25	823
10	782	26	819
11	823	27	854
12	823	28	822
13	824	29	823
14	823	30	823
15	823	31	721
16	822		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-281

Unit Name: Surry Unit 2

Date: 11-04-96

Completed by: Barry C. Bryant

Telephone: (804) 365-2786

MONTH: October, 1996

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	822	17	823
2	821	18	825
3	820	19	826
4	822	20	828
5	822	21	828
6	824	22	827
7	825	23	826
8	825	24	826
9	824	25	823
10	824	26	826
11	825	27	860
12	825	28	825
13	826	29	825
14	825	30	825
15	825	31	825
16	825		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: October, 1996

The following chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE:

10/01/96	0000	The reporting period began with the unit operating at 100% power, 850 MWe.
10/10/96	0934	Started power reduction to perform Operations Surveillance Procedure 1-OSP-TM-001, Turbine Inlet Valve Freedom Test.
	1450	Stopped power reduction at 71%, 620 MWe.
	1515	Started power increase following the completion of 1-OSP-TM-001.
	1800	Stopped power increase at 100%, 850 MWe.
10/29/96	1120	Revised calorimetric computer program was installed. 100% power, 853 MWe.
10/31/96	1503	Started power reduction following a report of oil leakage from the 1-FW-P-1A feedwater pump.
	1604	Stopped power reduction at 64%, 500 MWe.
	2056	Started power increase following the replacement of a leaking oil pump hose on the 1-FW-P-1A feedwater pump.
	2347	Stopped power increase at 96%, 785 MWe.
10/31/96	2400	The reporting period ended with the unit operating at 96% power, 785 MWe.

UNIT TWO:

10/01/96	0000	The reporting period began with the unit operating at 100% power, 857 MWe.
10/29/96	1443	Revised calorimetric computer program was installed.
10/31/96	2400	The reporting period ended with the unit operating at 100% power, 855 MWe.

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: October, 1996

DCP 94-068 **Design Change Package** 10-15-96
(Safety Evaluation 94-045)

Design Change Package 94-068 installed speed sensing panels and new relays to enhance the starting and running overcurrent protection for the Units 1 and 2 reactor coolant pumps (RCP).

The modification provides better motor protection for the RCPs, thereby reducing the possibility of failure. The change does not negatively affect the operation of the RCPs and does not reduce the margin of safety defined in the Technical Specifications. Therefore, an unreviewed safety question does not exist.

DCP 94-069 **Design Change Package** 10-15-96
(Safety Evaluation 96-017)

Design Change Package 94-069 upgraded a previously installed temporary ventilation duct jumper to permanent plant equipment. When in use, the jumper connects the inlet of the auxiliary building ventilation system filters and the inlet of the Category II iodine filter bank. This alignment is used only during containment purging operations to protect the safety-related filters from potential contaminants originating in containment.

When the subject jumper is in use, the affected unit will be shutdown and no fuel handling in containment will be permitted. The modification does not impair the functionality of safety-related equipment in meeting safety injection demands. Therefore, an unreviewed safety question does not exist.

SE 96-134 **Safety Evaluation** 10-17-96

Safety Evaluation 96-134 assessed the replacement of the existing waste processing stream demineralizer at the Surry Radwaste Facility with a new Thermex Waste Processing System.

The Thermex Waste Processing System utilizes inlet and outlet demineralizers and a reverse osmosis unit. The new system will more efficiently remove high conductivity and/or Antimony wastes from the effluent stream which will result in lower activity releases to the environment and a reduction in the amount of reject water that must be disposed of. Therefore, an unreviewed safety question does not exist.

DCP 95-025 **Design Change Package** 10-30-96
(Safety Evaluation 95-004)

Design Change Package 95-025 added an automatic trip function for the main control room (MCR) ventilation system supply fan 1-VS-AC-4. The new trip function will be automatically initiated following any design basis event requiring control room isolation to reduce the potential for any inleakage past MCR isolation dampers 1-VS-103A and 1-VS-103C. The DCP also installed an indicating light and switch in the MCR to permit 1-VS-AC-4 to be manually started or stopped.

The modification does not affect the operation of the subject isolation dampers and does not negatively impact control room or offsite dose limits. Therefore, an unreviewed safety question does not exist.

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: October, 1996

FS 96-43

Updated Final Safety Analysis Report Change
(Safety Evaluation 96-139)

10-31-96

Updated Final Safety Analysis Report Change 96-43 revised Chapter 17, "Quality Assurance," to reflect changes in the nuclear organization and to incorporate editorial changes.

These changes are administrative in nature and do not reduce the effectiveness of the QA Program or reduce the commitments in the NRC approved QA Program. There are no changes to any component, system or structure in the plant, or to any method of operations as a result of these changes. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: October, 1996

1-TOP-CH-4052 **Temporary Operating Procedure** 10-14-96
(Safety Evaluation No. 96-132)

Temporary Operating Procedure 1-TOP-CH-4052, "Backwashing Sample Line for 1-CH-I," was developed to provide instructions for backflushing the sample line from ion exchanger (IX) 1-CH-I-2 to remove any obstruction.

The backflushing operation requires the installation of a flexible hose and the use of primary grade water. The temporary modification and valve alignment changes are procedurally controlled. 1-CH-I-2 will be removed from service during the backflushing operation and will be flushed and sampled to ensure proper boron concentration before being returned to service. Therefore, an unreviewed safety question does not exist.

1[2]-PT-18.8 **Periodic Test Procedures** 10-21-96
1[2]-PT-18.8A (Safety Evaluation No. 96-135)

Periodic Test Procedures 1[2]-PT-18.8, "Charging Pump Service Water Performance," and 1[2]-PT-18.8A, "Charging Pump Component Cooling Performance," were revised to incorporate administrative controls that will ensure the operability of the charging pump service water and component cooling subsystems when returning a pump to service following maintenance.

While testing one of the subject pumps following maintenance, the operable train's pump will be placed in the "Off" mode. This mode of operation defeats the automatic start capability of the pump thereby preventing an inadvertent autostart should a transient period of low pressure be experienced. The administrative controls require that an operator be present to manually start the pump if an actual low pressure condition exists. Therefore, an unreviewed safety question is not created.

FLOWCALC **P-250 Computer Programs** 10-22-96
CALCALC
CALCCHK

FLOWPRNT **Operations Periodic Test Procedures**
2-OPT-RX-001 (Safety Evaluation No. 96-133)
2-OPT-RX-002
2-OPT-RX-003

2-OPT-RX-004
Plant computer (P-250) programs FLOWCALC, CALCALC, CALCCHK, and FLOWPRNT and Operations Periodic Test Procedures 2-OPT-RX-001, "Reactor Power Calorimetric Using CALCALC Computer Program," 2-OPT-RX-002, "Reactor Power Calorimetric Using Steam Flow and P-250 Computer Points (Manual)," 2-OPT-RX-003, "Reactor Power Calorimetric Using Feed Flow and P-250 Computer Points (Manual)," and 2-OPT-RX-004, "Reactor Power Calorimetric Using Feed Flow with P-250 Out of Service (Manual)," were revised to improve the accuracy and stability of calorimetric calculations.

The subject changes did not involve any hardware modifications and did not affect methods of operation or surveillance criteria. The accuracy of the calorimetric calculations remains within that assumed by the safety analysis. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: October, 1996

0-ECM-2201-01 **Electrical Corrective Maintenance Procedure** 10-24-96
(Safety Evaluation No. 96-136)

Electrical Corrective Maintenance Procedure 0-ECM-2201-01, "ELT Elimination Blackout Testing," was developed to provide instructions for conducting a discharge test of Appendix R emergency lights without the normal lighting in order to verify and, as necessary, adjust the emergency lighting.

This procedure may be implemented while the units are at power and will be performed for only one common fire area at a time or one fire area on each unit at a time. Battery powered lanterns are provided in the Appendix R cabinet for use by Operations personnel during the test and for the 24 hour post test period in which the emergency lighting batteries are being recharged. This activity will not affect the probability of occurrence or the consequences of any previously analyzed accidents. Therefore, an unreviewed safety question does not exist.

1-TOP-4076 **Temporary Operating Procedure** 10-28-96
(Safety Evaluation No. 96-137)

Temporary Operating Procedure 1-TOP-4076, "Systematic Sampling of the Component Cooling [CC] System," was developed to provide instructions for sampling the Unit 1 CC system to determine the source of in-leakage and radioactive contamination.

The CC sample is obtained at the equalizing valves for the reactor coolant pump thermal barrier discharge flow transmitters. When the equalizing valves are open, the CC thermal barrier discharge trip valves high flow trip function is defeated. To address this loss of function, the TOP establishes administrative controls to ensure that the trip valves are manually closed, if required. Therefore, an unreviewed safety question does not exist.

2-MOP-HT-001 **Maintenance Operating Procedure** 10-28-96
(Safety Evaluation No. 96-138)

Maintenance Operating Procedure 2-MOP-HT-001, "Removal from Service and Return to Service of Heat Tracing Panel 2B3," was developed to provide instructions for removing heat tracing panel (HTP) 2B3 from service and returning the panel to service. The procedure also describes the required actions to be taken when the auxiliary ventilation system flow indicators that are powered by HTP 2B3 are inoperable.

The performance of the subject procedure, including the alternate methods of monitoring auxiliary ventilation system flow, will not affect the operation or control of the auxiliary ventilation system and will not affect the margin of safety as defined by the Technical Specifications. Therefore, an unreviewed safety question does not exist.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: October, 1996

None During the Reporting Period

CHEMISTRY REPORT

MONTH/YEAR: October, 1996

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioactivity, $\mu\text{Ci/ml}$	7.60E-1	4.87E-1	6.20E-1	1.55E-1	9.53E-2	1.30E-1
Suspended Solids, ppm	-	-	-	-	-	-
Gross Tritium, $\mu\text{Ci/ml}$	4.80E-1	4.05E-1	4.35E-1	7.03E-1	5.82E-1	6.47E-1
^{131}I , $\mu\text{Ci/ml}$	2.23E-2	6.46E-3	8.51E-3	5.86E-5	3.27E-5	4.36E-5
$^{131}\text{I}/^{133}\text{I}$	0.53	0.37	0.45	0.10	0.06	0.08
Hydrogen, cc/kg	41.7	37.4	39.3	35.0	27.8	31.8
Lithium, ppm	2.27	2.03	2.13	2.32	2.08	2.22
Boron - 10, ppm*	84.7	66.2	75.3	229.1	216.8	222.9
Oxygen, (DO), ppm	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005
Chloride, ppm	0.003	0.001	0.002	0.005	0.004	0.005
pH at 25 degree Celsius	7.10	6.61	6.87	6.59	5.97	6.20

* Boron - 10 = Total Boron x 0.196

Comments:

None

**FUEL HANDLING
 UNITS 1 & 2**

MONTH/YEAR: October, 1996

New or Spent Fuel Shipment Number	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
New Fuel Unit 1 Batch 17 Shipment 1	10/22/96	12	27B	LM13U4	3.7970	14.63 Ci
			25B	LM13U2	3.8193	
			29B	LM13U6	3.8012	
			32B	LM13U9	3.8215	
			08B	LM13TK	3.8066	
			28B	LM13U5	3.7990	
			07B	LM13TJ	3.8095	
			43B	LM13UL	4.0132	
			30B	LM13U7	3.7998	
			26B	LM13U3	3.7988	
			33B	LM13UA	4.0114	
			42B	LM13UK	4.0117	
New Fuel Unit 1 Batch 17 Shipment 2	10/24/96	12	39B	LM13UG	3.9968	14.92 Ci
			34B	LM13UB	4.0109	
			10B	LM13TM	3.8100	
			44B	LM13UM	4.0069	
			48B	LM13UR	4.0073	
			45B	LM13UN	4.0003	
			11B	LM13TN	3.8010	
			12B	LM13TP	3.8143	
			55B	LM13UY	4.0169	
			09B	LM13TL	3.8101	
			46B	LM13UP	3.9994	
			49B	LM13US	3.9959	

**FUEL HANDLING
 UNITS 1 & 2**

MONTH/YEAR: October, 1996

New or Spent Fuel Shipment Number	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
New Fuel Unit 1 Batch 17 Shipment 3	10/29/96	12	54B	LM13UX	4.0205	14.83 Ci
			56B	LM13UZ	4.0133	
			22B	LM13TZ	3.8221	
			19B	LM13TW	3.8201	
			18B	LM13TV	3.8134	
			01B	LM13TC	3.8023	
			51B	LM13UU	4.0159	
			53B	LM13UW	3.9959	
			13B	LM13TQ	3.8083	
			14B	LM13TR	3.8059	
			50B	LM13UT	4.0022	
			58B	LM13V1	4.0123	
New Fuel Unit 1 Batch 17 Shipment 4	10/31/96	12	06B	LM13TH	3.8092	14.66 Ci
			60B	LM13V3	4.0198	
			47B	LM13UQ	3.9983	
			31B	LM13U8	3.8041	
			23B	LM13U0	3.8171	
			24B	LM13U1	3.8186	
			20B	LM13TX	3.8089	
			57B	LM13V0	4.0164	
			02B	LM13TD	3.8022	
			03B	LM13TE	3.8038	
			05B	LM13TG	3.8144	
			04B	LM13TF	3.8108	

**DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS**

MONTH/YEAR: October, 1996

None During the Reporting Period