VIRGINIA ELECTRIC AND POWER COMPANY Richmond, Virginia 23261

November 14, 1996

United States Nuclear Regulatory Commission Attention: Document Control Desk Washington, D.C. 20555

Serial No. 96-581 NL/RPC Docket Nos. 50-280 50-281 License Nos. DPR-32 DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY SURRY POWER STATION UNITS 1 AND 2 MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of October 1996.

If you have any questions or require additional information, please contact us.

Very truly yours,

S. P. Sarver, Acting Manager Nuclear Licensing and Operations Support

Enclosure

cc: U. S. Nuclear Regulatory Commission Region II 101 Marietta Street, N.W. Suite 2900 Atlanta, Georgia 30323

> Mr. R. A. Musser NRC Senior Resident Inspector Surry Power Station



JE241/,

VIRGINIA ELECTRIC AND POWER COMPANY SURRY POWER STATION MONTHLY OPERATING REPORT REPORT NO. 96-10

Approved:

Station Manager

<u>//-/2-96</u> Date

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OPERATING DATA REPORT

			Docket No.:	50-280
			Date:	10-04-96
			Completed By:	D. K. Mason
			Telephone:	(804) 365-2459
1.	Unit Name:	Surry Unit 1		
2.	Reporting Period:	October, 1996	5	
3.	Licensed Thermal Power (MWt):	2546		
4.	Nameplate Rating (Gross MWe):	847.5		
5.	Design Electrical Rating (Net MWe):	788		
6.	Maximum Dependable Capacity (Gross MWe):	840		

6. Maximum Dependable Capacity (Gross MWe): ...

Maximum Dependable Capacity (Net MWe): 7.

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

801

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

		This Month	YTD	Cumulative
11.	Hours In Reporting Period	745.0	7320.0	209160.0
12.	Number of Hours Reactor Was Critical	745.0	7320.0	145370.7
13.	Reactor Reserve Shutdown Hours	0.0	0.0	3774.5
14.	Hours Generator On-Line	745.0	7320.0	143067.0
15.	Unit Reserve Shutdown Hours	0.0	0.0	3736.2
16.	Gross Thermal Energy Generated (MWH)	1884854.8	18512995.8	334910788.3
17.	Gross Electrical Energy Generated (MWH)	630595.0	6149345.0	109726528.0
18.	Net Electrical Energy Generated (MWH)	608834.0	5935603.0	104391576.0
19.	Unit Service Factor	100.0%	100.0%	68.4%
20.	Unit Availability Factor	100.0%	100.0%	70.2%
21.	Unit Capacity Factor (Using MDC Net)	102.0%	101.2%	64.2%
22.	Unit Capacity Factor (Using DER Net)	103.7%	102.9%	63.3%
23.	Unit Forced Outage Rate	0.0%	0.0%	15.4%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling, February 20, 1997, 37 Days

25. If Shut Down at End of Report Period, Estimated Date of Start-up:

N/A _____

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
COMMERCIAL OPERATION		

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OPERATING DATA REPORT

Docket No.: 50-281 Date: 10-04-96 Completed By: D. K. Mason Telephone: (804) 365-2459

1. Unit Name: Surry Unit 2 2. Reporting Period: October, 1996 3. Licensed Thermal Power (MWt): 2546 4. Nameplate Rating (Gross MWe); 847.5 5. Design Electrical Rating (Net MWe): 788 840

6. Maximum Dependable Capacity (Gross MWe): ...

7. Maximum Dependable Capacity (Net MWe):

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

801

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

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	-	This Month	YTD	Cumulative
11.	Hours In Reporting Period	745.0	7320.0	206040.0
12.	Number of Hours Reactor Was Critical	745.0	6350.2	141853.1
13.	Reactor Reserve Shutdown Hours	0.0	0.0	328.1
14.	Hours Generator On-Line	745.0	6328.1	139884.2
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	1896586.7	15899950.8	328435004.8
17.	Gross Electrical Energy Generated (MWH)	636535.0	5274405.0	107430049.0
18.	Net Electrical Energy Generated (MWH)	614330.0	5095805.0	102206220.0
19.	Unit Service Factor	100.0%	86.4%	67.9%
20.	Unit Availability Factor	100.0%	86.4%	67.9%
21.	Unit Capacity Factor (Using MDC Net)	102.9%	86.9%	63.5%
22.	Unit Capacity Factor (Using DER Net)	104.6%	88.3%	63.0%
23.	Unit Forced Outage Rate	0.0%	2.0%	12.6%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Maintenance, December 6, 1996, 10 Days

25. If Shut Down at End of Report Period, Estimated Date of Start-up:

N/A

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION		

UNIT SHUTDOWN AND POWER REDUCTION (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: October, 1996

							Do	ocket No.:	50-280
							Ui	nit Name:	Surry Unit 1
								Date:	11-01-96
							Comp	oleted by:	M. J. Fanguy
							Te	elephone:	(804) 365-2155
	(1)		(2)	(3) Method		(4)	(5)		
		Duration		of	LER	System	Component	Cause &	Corrective Action to
Date	Туре	Hours	Reason	Shutting Down Rx	No.	Code	Code	Prevent	Recurrence
961010	S	N/A	В	N/A	N/A	ТА	V	Reactor to perfor Surveilla 1-OSP-T Valve Fr	power was reduced m Operations nce Procedure M-001, Turbine Inlet eedom Test.
961031	F	N/A	A	N/A	N/A	SJ	TBG	Reactor to replac hose on feedwate	power was reduced e a leaking oil pump the 1-FW-P-1A er pump.

(2)	
(2)	

- F: Forced S: Scheduled
- REASON:
- A Equipment Failure (Explain)
- B Maintenance or Test
- C Refueling
- D Regulatory Restriction
- E Operator Training & Licensing Examination

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- F Administrative
- G Operational Error (Explain)

(4)

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

- (3)
- METHOD: 1 - Manual
- 2 Manual Scram
- 3 Automatic Scram
- 4 Other (Explain)

(5) Exhibit 1 - Same Source

UNIT SHUTDOWN AND POWER REDUCTION (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: October, 1996

							Do	cket No.:	50-281
							U	Date:	5011y 0111 2 11-01-96
							Comp	leted by:	M. J. Fanguy
							Te	lephone:	(804) 365-2155
	(1)		(2)	(3) Method		(4)	(5)		
Date	Туре	Duration Hours	Reason	of Shutting Down Rx	LER No.	System Code	Component Code	Cause & Prevent	Corrective Action to Recurrence

None During the Reporting Period

(1)

5 3. A. S.

(2)

- F: Forced S: Scheduled
- **REASON:**
- A Equipment Failure (Explain)
- в -Maintenance or Test Refueling
- С-
- D -Regulatory Restriction
- Е -**Operator Training & Licensing Examination**
- F -Administrative
- Operational Error (Explain) G-

(4)

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(3)

- METHOD: Manual 1 -
- 2 -
- Manual Scram 3 - Automatic Scram
- 4 -Other (Explain)

(5) Exhibit 1 - Same Source

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AVERAGE DAILY UNIT POWER LEVEL

Docket No .:	50-280
Unit Name:	Surry Unit 1
Date:	11-04-96
Completed by:	B. C. Bryant
Telephone:	(804) 365-2786

MONTH: October, 1996

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Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	818	17	822
2	819	18	821
3	819	19	822
4	821	20	823
5	822	21	821
6	822	22	823
7	822	23	823
8	823	24	822
9	822	25	823
10	782	26	819
11	823	27	854
12	823	28	822
13	824	29	823
14	823	30	823
15	823	31	721
16	822		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

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AVERAGE DAILY UNIT POWER LEVEL

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50-281
Surry Unit 2
11-04-96
Barry C. Bryant
(804) 365-2786

MONTH: October, 1996

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Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	822	17	823
2	821	18	825
3	820	19	826
4	822	20	828
5	822	21	828
6	824	22	827
7	825	23	826
8	825	24	826
9	824	25	823
10	824	26	826
11	825	27	860
12	825	28	
13	826	29	825
14	825	30	825
15	825	31	825
16	825		

INSTRUCTIONS

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On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

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SUMMARY OF OPERATING EXPERIENCE

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MONTH/YEAR: October, 1996

The following chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE:		
10/01/96	0000	The reporting period began with the unit operating at 100% power, 850 MWe.
10/10/96	0934	Started power reduction to perform Operations Surveillance Procedure 1-OSP-TM-001, Turbine Inlet Valve Freedom Test.
	1450	Stopped power reduction at 71%, 620 MWe.
	1515	Started power increase following the completion of 1-OSP-TM-001.
	1800	Stopped power increase at 100%, 850 MWe.
10/29/96	1120	Revised calorimetric computer program was installed. 100% power, 853 MWe.
10/31/96	1503	Started power reduction following a report of oil leakage from the 1-FW-P-1A feedwater pump.
	1604	Stopped power reduction at 64%, 500 MWe.
	2056	Started power increase following the replacement of a leaking oil pump hose on the 1-FW-P-1A feedwater pump.
	2347	Stopped power increase at 96%, 785 MWe.
10/31/96	2400	The reporting period ended with the unit operating at 96% power, 785 MWe.

UNIT TWO:		
10/01/96	0000	The reporting period began with the unit operating at 100% power, 857 MWe.
10/29/96	1443	Revised calorimetric computer program was installed.
10/31/96	2400	The reporting period ended with the unit operating at 100% power, 855 MWe.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: October, 1996

DCP 94-068

Design Change Package (Safety Evaluation 94-045)

Design Change Package 94-068 installed speed sensing panels and new relays to enhance the starting and running overcurrent protection for the Units 1 and 2 reactor coolant pumps (RCP).

The modification provides better motor protection for the RCPs, thereby reducing the possibility of failure. The change does not negatively affect the operation of the RCPs and does not reduce the margin of safety defined in the Technical Specifications. Therefore, an unreviewed safety question does not exist.

DCP 94-069

SE 96-134

Design Change Package (Safety Evaluation 96-017)

Design Change Package 94-069 upgraded a previously installed temporary ventilation duct jumper to permanent plant equipment. When in use, the jumper connects the inlet of the auxiliary building ventilation system filters and the inlet of the Category II iodine filter bank. This alignment is used only during containment purging operations to protect the safety-related filters from potential contaminants originating in containment.

When the subject jumper is in use, the affected unit will be shutdown and no fuel handling in containment will be permitted. The modification does not impair the functionality of safety-related equipment in meeting safety injection demands. Therefore, an unreviewed safety question does not exist.

Safety Evaluation

Safety Evaluation 96-134 assessed the replacement of the existing waste processing stream demineralizer at the Surry Radwaste Facility with a new Thermex Waste Processing System.

The Thermex Waste Processing System utilizes inlet and outlet demineralizers and a reverse osmosis unit. The new system will more efficiently remove high conductivity and/or Antimony wastes from the effluent stream which will result in lower activity releases to the environment and a reduction in the amount of reject water that must be disposed of. Therefore, an unreviewed safety question does not exist.

DCP 95-025

Design Change Package (Safety Evaluation 95-004)

Design Change Package 95-025 added an automatic trip function for the main control room (MCR) ventilation system supply fan 1-VS-AC-4. The new trip function will be automatically initiated following any design basis event requiring control room isolation to reduce the potential for any inleakage past MCR isolation dampers 1-VS-103A and 1-VS-103C. The DCP also installed an indicating light and switch in the MCR to permit 1-VS-AC-4 to be manually started or stopped.

The modification does not affect the operation of the subject isolation dampers and does not negatively impact control room or offsite dose limits. Therefore, an unreviewed safety question does not exist.

10-17-96

10-30-96

10-15-96

10-15-96

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: October, 1996

FS 96-43

Updated Final Safety Analysis Report Change (Safety Evaluation 96-139)

10-31-96

Updated Final Safety Analysis Report Change 96-43 revised Chapter 17, "Quality Assurance," to reflect changes in the nuclear organization and to incorporate editorial changes.

These changes are administrative in nature and do not reduce the effectiveness of the QA Program or reduce the commitments in the NRC approved QA Program. There are no changes to any component, system or structure in the plant, or to any method of operations as a result of these changes. Therefore, an unreviewed safety question does not exist.

PROCEDURE OR METHOD OF OPERATION CHANGES THAT DID NOT REQUIRE NRC APPROVAL

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MONTH/YEAR:	October,	1996
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1-TOP-CH-4052	Temporary Operating Procedure (Safety Evaluation No. 96-132)	10-14-96
	Temporary Operating Procedure 1-TOP-CH-4052, "Backwashing Sample 1-CH-I," was developed to provide instructions for backflushing the sample line exchanger (IX) 1-CH-I-2 to remove any obstruction.	Line for from ion
- .	The backflushing operation requires the installation of a flexible hose and the primary grade water. The temporary modification and valve alignment char procedurally controlled. 1-CH-I-2 will be removed from service during the bac operation and will be flushed and sampled to ensure proper boron concentration being returned to service. Therefore, an unreviewed safety question does not ensure proceeding the back of the service of the service.	e use of nges are ckflushing on before exist.
1[2]-PT-18.8 1[2]-PT-18.8A	Periodic Test Procedures (Safety Evaluation No. 96-135)	10-21-96
· ·	Periodic Test Procedures 1[2]-PT-18.8, "Charging Pump Service Water Perfor and 1[2]-PT-18.8A, "Charging Pump Component Cooling Performance," were r incorporate administrative controls that will ensure the operability of the charging service water and component cooling subsystems when returning a pump to following maintenance.	ormance," evised to ing pump o service
	While testing one of the subject pumps following maintenance, the operable train will be placed in the "Off" mode. This mode of operation defeats the automore capability of the pump thereby preventing an inadvertent autostart should a period of low pressure be experienced. The administrative controls require operator be present to manually start the pump if an actual low pressure condition Therefore, an unreviewed safety question is not created.	in's pump natic start transient that an on exists.
FLOWCALC CALCALC CALCCHK	P-250 Computer Programs	10-22-96
FLOWPRNT 2-OPT-RX-001 2-OPT-RX-002 2-OPT-RX-003	Operations Periodic Test Procedures (Safety Evaluation No. 96-133)	
2-OPT-RX-004	Plant computer (P-250) programs FLOWCALC, CALCALC, CALCCHK, and FLC and Operations Periodic Test Procedures 2-OPT-RX-001, "Reactor Power Ca Using CALCALC Computer Program," 2-OPT-RX-002, "Reactor Power Calorime Steam Flow and P-250 Computer Points (Manual)," 2-OPT-RX-003, "Reactor Calorimetric Using Feed Flow and P-250 Computer Points (Manual)," and 2-OPT "Reactor Power Calorimetric Using Feed Flow with P-250 Out of Service (Manual revised to improve the accuracy and stability of calorimetric calculations.	OWPRNT Ilorimetric tric Using or Power ⁻ -RX-004, al)," were
	The subject changes did not involve any hardware modifications and did r methods of operation or surveillance criteria. The accuracy of the ca calculations remains within that assumed by the safety analysis. There unreviewed safety question does not exist.	not affect llorimetric efore, an

PROCEDURE OR METHOD OF OPERATION CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: October, 1996

0-ECM-2201-01 Electrical Corrective Maintenance Procedure (Safety Evaluation No. 96-136) 10-24-96

Electrical Corrective Maintenance Procedure 0-ECM-2201-01, "ELT Elimination Blackout Testing," was developed to provide instructions for conducting a discharge test of Appendix R emergency lights without the normal lighting in order to verify and, as necessary, adjust the emergency lighting.

This procedure may be implemented while the units are at power and will be performed for only one common fire area at a time or one fire area on each unit at a time. Battery powered lanterns are provided in the Appendix R cabinet for use by Operations personnel during the test and for the 24 hour post test period in which the emergency lighting batteries are being recharged. This activity will not affect the probability of occurrence or the consequences of any previously analyzed accidents. Therefore, an unreviewed safety question does not exist.

Temporary Operating Procedure (Safety Evaluation No. 96-137)

Temporary Operating Procedure 1-TOP-4076, "Systematic Sampling of the Component Cooling [CC] System," was developed to provide instructions for sampling the Unit 1 CC system to determine the source of in-leakage and radioactive contamination.

The CC sample is obtained at the equalizing valves for the reactor coolant pump thermal barrier discharge flow transmitters. When the equalizing valves are open, the CC thermal barrier discharge trip valves high flow trip function is defeated. To address this loss of function, the TOP establishes administrative controls to ensure that the trip valves are manually closed, if required. Therefore, an unreviewed safety question does not exist.

Maintenance Operating Procedure (Safety Evaluation No. 96-138) 10-28-96

10-28-96

Maintenance Operating Procedure 2-MOP-HT-001, "Removal from Service and Return to Service of Heat Tracing Panel 2B3," was developed to provide instructions for removing heat tracing panel (HTP) 2B3 from service and returning the panel to service. The procedure also describes the required actions to be taken when the auxiliary ventilation system flow indicators that are powered by HTP 2B3 are inoperable.

The performance of the subject procedure, including the alternate methods of monitoring auxiliary ventilation system flow, will not affect the operation or control of the auxiliary ventilation system and will not affect the margin of safety as defined by the Technical Specifications. Therefore, an unreviewed safety question does not exist.

1-TOP-4076

2-MOP-HT-001

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TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

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MONTH/YEAR: October, 1996

None During the Reporting Period

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CHEMISTRY REPORT

MONTH/YEAR: October, 1996

	Unit No. 1					
Primary Coolant Analysis	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioactivity, μCi/ml	7.60E-1	4.87E-1	6.20E-1	1.55E-1	9.53E-2	1.30E-1
Suspended Solids, ppm		_	_	_	_	
Gross Tritium, uCi/ml	4.80E-1	4.05E-1	4.35E-1	7.03E-1	5.82E-1	6.47E-1
μ ¹³¹ , μCi/ml	2.23E-2	6.46E-3	8.51E-3	5.86E-5	3.27E-5	4.36E-5
131 _{/l} 133	0.53	0.37	0.45	0.10	0.06	0.08
Hydrogen, cc/kg	41.7	37.4	39.3	35.0	27.8	31.8
Lithium, ppm	2.27	2.03	2.13	2.32	2.08	2.22
Boron - 10, ppm*	84.7	66.2	75.3	229.1	216.8	222.9
Oxygen, (DO), ppm	≤0.005	≤0.005	≤0.005	≤0.005	≤0.005	≤0.005
Chloride, ppm	0.003	0.001	0.002	0.005	0.004	0.005
pH at 25 degree Celsius	7.10	6.61	6.87	6.59	5.97	6.20

* Boron - 10 = Total Boron x 0.196

Comments:

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None

FUEL HANDLING UNITS 1 & 2

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MONTH/YEAR: October, 1996

New or Spent Fuel Shipment Number	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
New Fuel Unit 1 Batch 17						
Shipment 1	10/22/96	12	27B	LM13U4	3.7970	14.63 Ci
			25B	LM13U2	3.8193	
			29B	LM13U6	3.8012	
			32B	LM13U9	3.8215	
			08B	LM13TK	3.8066	
			28B	LM13U5	3.7990	
			07B	LM13TJ	3.8095	
	,		43B	LM13UL	4.0132	
			30B	LM13U7	3.7998	
			26B	LM13U3	3.7988	
			33B	LM13UA	4.0114	
			42B	LM13UK	4.0117	
New Fuel Unit 1						
Batch 17 Shipment 2	10/24/96	12	39B	LM13UG	3.9968	14.92 Ci
			34B	LM13UB	4.0109	
			10B	LM13TM	3.8100	
			44B	LM13UM	4.0069	
			48B	LM13UR	4.0073	
			45B	LM13UN	4.0003	÷
			11B	LM13TN	3.8010	
			12B	LM13TP	3.8143	
			55B	LM13UY	4.0169	
			09B	LM13TL	3.8101	
			46B	LM13UP	3.9994	
			49B	LM13US	3.9959	

FUEL HANDLING UNITS 1 & 2

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MONTH/YEAR: October, 1996

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New or Spent Fuel Shipment Number	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
New Fuel Unit 1 Batch 17						
Shipment 3	10/29/96	12	54B	LM13UX	4.0205	14.83 Ci
			56B	LM13UZ	4.0133	
			22B	LM13TZ	3.8221	
			19B	LM13TW	3.8201	
	•		18B	LM13TV	3.8134	
			01B	LM13TC	3.8023	
			51B	LM13UU	4.0159	
			53B	LM13UW	3.9959	
			13B	LM13TQ	3.8083	
			14B	LM13TR	3.8059	,
			50B	LM13UT	4.0022	
			58B	LM13V1	4.0123	
New Fuel Unit 1 Batch 17						
Shipment 4	10/31/96	12	06B	LM13TH	3.8092	14.66 Ci
			60B	LM13V3	4.0198	
			47B	LM13UQ	3.9983	
•			31B	LM13U8	3.8041	
			23B	LM13U0	3.8171	
			24B	LM13U1	3.8186	
			20B	LM13TX	3.8089	
			57B	LM13V0	4.0164	
			02B	LM13TD	3.8022	
			03B	LM13TE	3.8038	
			05B	LM13TG	3.8144	
			04B	LM13TF	3.8108	



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DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

MONTH/YEAR: October, 1996

None During the Reporting Period