

**Virginia Electric and Power Company
Surry Power Station
5570 Hog Island Road
Surry, Virginia 23883**

September 20, 1996

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Serial No.: 96-489
SPS:mdk
Docket No.: 50-281
License No.: DPR-37

Dear Sirs:

Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to Surry Power Station Units 1 and 2.

REPORT NUMBER

50-280/50-281/96-007-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,



D. A. Christian
Station Manager

Enclosure

pc: Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

R. A. Musser
NRC Senior Resident Inspector
Surry Power Station

9609240390 960919
PDR ADDCK 05000280
S PDR

IE22/1

LICENSING EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

SURRY POWER STATION, Unit 1

DOCKET NUMBER (2)

05000 - 280

PAGE (3)

1 OF 3

TITLE (4)

Fire Watch Patrol Inspection Frequency Exceeds One Hour

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
08	21	96	96	-- 007 --	00	09	19	96	Surry Unit 2	05000 - 281
									FACILITY NAME	DOCKET NUMBER
										05000 -

OPERATING MODE (9) N POWER LEVEL (10) 100%	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR: (Check one or more) (11)									
		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(c)		
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)		
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER		
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)			(Specify in Abstract below and in Text, NRC Form 366A)	
	20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
	20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)					

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Including Area Code)
D. A. Christian, Station Manager	(804) 357-3184

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE)	X	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On August 21, 1996, with Unit 1 and Unit 2 both at 100% power, fire watch patrols failed to complete fire detection zone inspections within the Technical Specifications required time period. With a fire detection panel inoperable, fire watch patrols were required to perform inspections of the affected fire detection zones "at least once per hour" in accordance with Technical Specification 3.21.B.1. The required fire detection zone inspections were performed however, the inspection period between some individual fire detection zones exceeded the allowed time period when a fire watch patrol was detained by Security after a security alarm was inadvertently activated. Required fire suppression equipment remained operable. A fire has not occurred. The health and safety of the public were not affected by this event. This event was caused by inadvertent activation of a security alarm. To prevent recurrence, fire watch training is being changed to emphasize compliance with Security requirements and Technical Specifications fire watch requirements. This report is being made pursuant to 10CFR50.73(a)(2)(i)(B), for any operation prohibited by Technical Specifications.

LICENSING EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
SURRY POWER STATION, Unit 1	05000 - 280	96	- 007 -	0	2 OF 3

TEXT (If more space is required, use additional copies of NRC Form 368A) (17)

1.0 DESCRIPTION OF THE EVENT

At 1838 hours on August 21, 1996 a fire detection panel [EIS:IC] was declared inoperable. Fire watch patrols were initiated within one hour due to the inoperable fire detection panel, as required by Technical Specifications 3.21.B.1. Fire watch patrols were required to inspect the affected fire detection zones "at least once per hour" while the fire detection panel was inoperable. Each fire detection zone inspection was performed while the fire detection panel was inoperable. However, one fire watch patrol was detained by Security when a security access alarm was inadvertently activated. The fire detection zone inspections were documented on Fire Watch Tour Documentation sheets. Required fire suppression equipment remained operable throughout this event. A fire did not occur.

A review of the Security Incident Report, Fire Watch Tour Documentation sheets, and personnel interviews, determined that a fire watch patrol had entered and inspected a fire detection zone and upon exiting the fire detection zone a security access alarm was inadvertently activated. The security access alarm occurred at 0029 hours when the fire watch patrol opened the door to exit the area prior to receiving positive egress indication on the security card reader. Security contacted the fire watch patrol using the Station intercommunication system. The fire watch patrol remained at the access door until a security officer arrived and investigated the incident as required by security procedures. The fire watch patrol was detained for approximately 13 minutes while the incident investigation was conducted. The fire watch patrol was advised by the security officer of the proper procedures and use of the security card reader. Following the security investigation, the fire watch patrol immediately proceeded to complete the required inspections. However, as a result of being detained during the security incident, some fire detection zone inspections exceeded the Technical Specifications required frequency of "at least once per hour." Three fire detection zones were inspected 7 minutes late, two fire detection zones were inspected 11 minutes late, and three fire detection zones were inspected 13 minutes late. All other inspections were conducted in compliance with Technical Specifications and security requirements.

Although each required inspection was performed, some inspections exceeded the Technical Specifications required frequency of "at least once per hour." Therefore, this event is being reported in accordance with 10 CFR 50.73(a)(2)(i)(B), for any operation prohibited by Technical Specifications.

LICENSING EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

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FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)			PAGE (3)
SURRY POWER STATION, Unit 1		05000 - 280		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3 of 3
				96	- 007 -	0	

TEXT (if more space is required, use additional copies of NRC Form 366A) (17)

2.0 SAFETY CONSEQUENCES AND IMPLICATIONS

There are no safety consequences or implications associated with this event. The fire watch patrols were initiated within the Technical Specifications required time period of one hour. The fire detection zones requiring inspection were inspected by the fire watch patrols. The required number of inspections were performed. A fire did not occur. Fire watch personnel had received fire watch training prior to the event and remained qualified to detect, report, and extinguish fires. Fire watch personnel had been briefed regarding their duties, responsibilities, and assignments regarding fire detection zone inspections, prior to the event. Required fire suppression equipment remained operable throughout this event.

3.0 CAUSE

This event was caused by inadvertent activation of a security alarm due to personnel error.

4.0 IMMEDIATE CORRECTIVE ACTIONS

The Senior Reactor Operator instructed the fire watch personnel that fire detection zone inspections must be performed within Technical Specifications requirements and Security requirements. Fire watch personnel were also instructed to inform Security that they are performing the duties of a fire watch if a security alarm is activated during the performance of a fire watch patrol.

5.0 ADDITIONAL CORRECTION ACTION

The fire detection panel was returned to service at 0555 hours on August 22, 1996.

6.0 ACTIONS TO PREVENT RECURRENCE

Fire watch training and associated lesson plans will be enhanced to emphasize compliance with the Technical Specifications requirements and Security requirements.

7.0 SIMILAR EVENTS

There were no similar events identified.

8.0 ADDITIONAL INFORMATION

None.

ATTACHMENT 3

(Page 2 of 2)

LER Checklist

LER Narrative (continued)

Initials/Date

Manufacturer and Model No. of each failed component *None*

MDK 9-19-96

Assessment of the safety significance and implications of the event

MDK 9-19-96

Description of corrective actions

MDK 9-19-96

Reference to previous similar events *None*

MDK 9-19-96

Status of unaffected unit (if applicable) *Both Units Affected*

MDK 9-19-96

Total accumulated ECCS actuations to date, per unit (if applicable) *Not Applicable*

MDK 9-19-96

LER Abstract and Narrative

No typographical errors or misspelled words

MDK 9-19-96

Statements of facts verified for accuracy, documented and contained in LER permanent file.

MDK 9-19-96

LER Processing

NPRDS Reporter contacted for component failures or malfunctions to determine NPRDS reportability *Not a component failure event*

MDK 9-19-96

Draft LER reviewed prior to distribution by Supervisor Licensing

MDK 9-19-96

LER distributed to the plant for comments

MDK 9-19-96

Comments received and incorporated/resolved

MDK 9-19-96

LER approved by SNSOC (9-18-96) SNSOC Meeting # 960918 -1607

MDK 9-19-96

LER signed by Station Manager 9-18-96

MDK 9-19-96

Date inserted on LER submittal letter and NRC Form 366, Item 7 (Report Date) 9-19-96

MDK 9-19-96

LER submitted to NRC

____/____

LER copies transmitted to distribution

____/____

LER package sent to Station Records

____/____

LER database has been updated

MDK 9-19-96

REVIEWER/DATE:

____/____

NOTE: This completed form must be included in the LER package

*Tom
Please handle
these*

ATTACHMENT 3

(Page 1 of 2)

LER Checklist

Initials/Date

LER Data

DR Number: 596-1799 MDKJ 9-19-96
 LER Number: 50-280/96-007-00 and 50-281/96-007-00 MDKJ 9-19-96
 LER Assigned to: Garber / Kocmarcik MDKJ 9-19-96
 Letter Serial Number: 96-489 (optional) MDKJ 9-19-96

LER Abstract

All the Form 366 Items except date of submittal are filled out MDKJ 9-19-96
 Abstract less than 1400 characters, including spaces *Submittal date included MDKJ* MDKJ 9-19-96
 Description of major occurrences MDKJ 9-19-96
 Component/system failures None MDKJ 9-19-96
 Description of corrective actions taken or planned MDKJ 9-19-96
 Assessment of the safety significance and implications of the event MDKJ 9-19-96

LER Narrative

10 CFR reportability requirement or "OTHER" box checked MDKJ 9-19-96
10 CFR 50.73(a)(2)(i)(B)
 Plant operating conditions prior to event MDKJ 9-19-96
 Structures/components/systems inoperable prior to event MDKJ 9-19-96
 Dates and approximate times of occurrences MDKJ 9-19-96
 Cause of each component failure/personnel error MDKJ 9-19-96
 Failure mode/mechanism/effect of failed components None MDKJ 9-19-96
 EIS component function identifiers/system names MDKJ 9-19-96
 A list affected items for multiple functional component failures None MDKJ 9-19-96
 Elapsed time of inoperability of failed train of safety systems MDKJ 9-19-96
 Component/system failure/procedural error discovery method None MDKJ 9-19-96
Personnel Error discovery by Security Alarm
 Operator actions which affected course of events MDKJ 9-19-96
 Automatically or manually initiated ESF actuations None MDKJ 9-19-96