

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Surry Power Station, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 8 0	PAGE (3) 1 OF 0 3
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TITLE (4)
Missed Surveillance Due to Personnel Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
07	14	88	88	027	000	08	03	88			05000

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 000	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.38(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.38(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)								

LICENSEE CONTACT FOR THIS LER (12)

NAME D. L. Benson, Station Manager	TELEPHONE NUMBER
	AREA CODE: 8 0 4 3 5 7 - 3 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)														
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)	EXPECTED SUBMISSION DATE (15)	MONTH DAY YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO		

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 14, 1988 at 1245 hours, with Unit 1 at 10E-7 amp intermediate range and Unit 2 at 100% power, it was determined the Technical Specification that required sampling had not been performed within the required time frame. Technical Specification Table 3.7-5(a) requires that the service water side of the component cooling (CC) water heat exchangers (EHS-HX) be sampled and analyzed for radioactive contamination every 12 hours. Due to an error made by the technician performing the analysis, one sample for the "B" and "C" CC heat exchangers was discarded without the proper analysis being performed. This resulted in the samples being performed approximately 18 hours apart. Radiation levels of samples prior to and following the missed samples were within specified limits.

Samples of the "B" and "C" heat exchanger service water were immediately obtained and analyzed. The analysis of samples were found to be within acceptable limits. Personnel were reeducated in the importance of thorough analysis.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		— 0	2 7	— 0	0	0 2	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

1.0 Description of the Event

On July 14, 1988 at 1245 hours, with Unit 1 at 10E-7 amp intermediate range and Unit 2 at 100% power, it was determined the Technical Specification that required sampling had not been performed within the required time frame. Technical Specification Table 3.7-5(a) requires that the service water (SW) side of the component cooling (CC) water heat exchangers (E11S-HX) be sampled and analyzed for radioactive contamination every 12 hours. Due to an error made by the technician performing the analysis, one sample for the "B" and "C" CC heat exchangers was discarded without the proper analysis being performed. This resulted in the samples being performed approximately 18 hours apart. Radiation levels of samples prior to and following the missed samples were within specified limits.

2.0 Safety Consequences and Implications

The component cooling system is an intermediate cooling system which serves both reactor units. It transfers heat from heat exchangers containing reactor coolant, other radioactive liquids, and other fluids to the service water system.

Since there was no increase in the circulation water discharge radiation monitor (indication of CC to SW tube integrity) during this event, the health and safety of the public were not affected.

3.0 Cause

The service water contamination analysis procedure consists of three steps: (1) place sample in detector, (2) erase old data/start new data acquisition, and (3) perform analysis on new data. In this event, the technician performing the procedure neglected to perform step 2. Step 3 was therefore performed on old data and proper analysis of the new samples was not obtained.

4.0 Immediate Corrective Action(s)

Samples of the "B" and "C" heat exchanger service water were immediately obtained and analyzed. The analysis of samples were found to be within acceptable limits.

5.0 Additional Corrective Action(s)

None.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

6.0 Action(s) Taken to Prevent Recurrence

Personnel were reinstructed in the importance of thorough analysis.

7.0 Similar Events

None.

8.0 Manufacturer/Model Number

N/A

VIRGINIA ELECTRIC AND POWER COMPANY
Surry Power Station
P. O. Box 315
Surry, Virginia 23883

August 3, 1988

U.S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Serial No.: 88-035
Docket No.: 50-280
Licensee No.: DPR-32

Gentlemen:

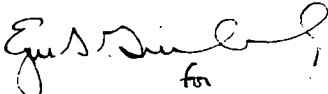
Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

REPORT NUMBER

88-027-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,



for
David L. Benson
Station Manager

Enclosure

cc: Dr. J. Nelson Grace
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

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