

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>Surry Power Station, Units 1 &amp; 2</b>	DOCKET NUMBER (2) <b>0 5 0 0 0 2 8 0</b>	PAGE (3) <b>1 OF 0 3</b>
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TITLE (4)  
**Required Surveillance Not Performed Due To Human Error**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
07	17	87	87	035	00	07	23	87			05000

OPERATING MODE (8) <b>N</b>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) <b>1 0 0</b>	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(e)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.38(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.38(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)								

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME <b>D. L. Benson, Station Manager</b>	AREA CODE <b>8104</b>	NUMBER <b>357</b>	EXTENSION <b>-13184</b>

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO						

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 17, 1987, with Units 1 and 2 at 100% power, it was determined that the Emergency Diesel Generator (EDG) {EIIS-DG} degraded/undervoltage protection {EIIS-EB} surveillance test was not being performed. This test demonstrates that the degraded/undervoltage protection is defeated when the EDG is the sole source of power to an emergency bus, and that the protection is reinstated when the diesel output breaker is open. Technical Specification 4.6.1.b requires that this test be performed during each refueling shutdown. Failure to perform the diesel degraded/undervoltage surveillance as required by Technical Specifications was caused by a failure to incorporate the requirements of a Technical Specification amendment into the existing test program. This was determined to be a reportable occurrence by the Station Nuclear Safety and Operating Committee on December 4, 1987. Special Tests for functional testing of the enable/disable interlocks were performed and completed on the emergency buses in September, 1987. This testing verified that the relays are fully operable and that the protection matrix functions as designed. The refueling Periodic Tests will be revised to include testing of the enable/disable interlock. The revisions will be completed prior to the next refueling outage.

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PDR ADOCK 05000280  
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*Handwritten initials:* JLR  
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		87	035	00	2	OF 03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

1.0 Description of the Event

On July 17, 1987, with Units 1 and 2 at 100% power, it was determined that the Emergency Diesel Generator (EDG) {EIIS-DG} degraded/undervoltage protection {EIIS-EB} surveillance test was not being performed. This test demonstrates that the degraded/undervoltage protection is defeated when the EDG is the sole source of power to an emergency bus, and that the protection is reinstated when the diesel output breaker is open. Technical Specification 4.6.1.b requires that this test be performed during each refueling shutdown. This was determined to be a reportable occurrence by the Station Nuclear Safety and Operating Committee on December 4, 1987.

2.0 Safety Consequences and Implications

The tests specified in Technical Specification Section 4.6 are designed to demonstrate that the EDGs will provide power for operation of essential safeguards equipment. They also assure that the EDG system controls and the control systems for the safeguards equipment will function automatically in the event of a loss of normal station service power. Technical reviews have been performed to determine the effect of failure of an untested circuit. It was determined that the impact on safety was minimal in that no single failure could cause a loss of the safety function. Therefore, the health and safety of the public were not affected.

3.0 Cause

Failure to perform the diesel degraded/undervoltage surveillance as required by Technical Specifications was caused by a failure to incorporate the requirements of a Technical Specification amendment into the existing test program.

4.0 Immediate Corrective Action(s)

Special Tests (1-ST-206, 2-ST-206) for functional testing of the enable/disable interlocks were performed and completed on the emergency buses in September, 1987. This testing verified that the relays are fully operable and that the protection matrix functions as designed.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

5.0 Additional Corrective Action(s)

None required.

6.0 Actions Taken to Prevent Recurrence

The refueling Periodic Tests (1, 2-PT-18.2A, B) will be revised to include testing of the enable/disable interlock. The revisions will be completed prior to the next refueling outage.

7.0 Similar Events

None.

8.0 Manufacturer/Model Number

N/A

VIRGINIA ELECTRIC AND POWER COMPANY  
Surry Power Station  
P. O. Box 315  
Surry, Virginia 23883

December 31, 1987

U.S. Nuclear Regulatory Commission  
Document Control Desk  
016 Phillips Building  
Washington, D.C. 20555

Serial No.: 87-043  
Docket No.: 50-280  
Docket No.: 50-281  
Licensee No.: DPR-32  
Licensee No.: DPR-37

Gentlemen:

Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Units 1 and 2.

REPORT NUMBER

87-035-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

*David L. Benson*

David L. Benson  
Station Manager

Enclosure

cc: Dr. J. Nelson Grace  
Regional Administrator  
Suite 2900  
101 Marietta Street, NW  
Atlanta, Georgia 30323

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