VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

July 13, 1992

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

Serial No. 92-459
NO/RPC:vlh
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of June 1992.

Very truly yours,

for W. L. Stewart
Senior Vice President - Nuclear

Enclosure

cc: U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, N. W.
Suite 2900
Atlanta, Georgia 30323

Mr. M. W. Branch
NRC Senior Resident Inspector
Surry Power Station

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION
MONTHLY OPERATING REPORT
REPORT NO. 92-06

Approved:

Station Manager

7/10/92
Date

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OPERATING DATA REPORT

Docket No.: 50-280
Date: 07-07-92
Completed By: D. Mason
Telephone: (804) 365-2459

1. Unit Name:..... Surry Unit 1
2. Reporting Period:..... June 1992
3. Licensed Thermal Power (MWt):..... 2441
4. Nameplate Rating (Gross MWe):..... 847.5
5. Design Electrical Rating (Net MWe):..... 788
6. Maximum Dependable Capacity (Gross MWe):.... 820
7. Maximum Dependable Capacity (Net MWe):..... 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

		This Month	YTD	Cumulative
11.	Hours In Reporting Period	720.0	4367.0	171143.0
12.	Number of Hours Reactor Was Critical	720.0	2723.8	110958.0
13.	Reactor Reserve Shutdown Hours	0.0	0.0	3774.5
14.	Hours Generator On-Line	720.0	2668.6	108908.8
15.	Unit Reserve Shutdown Hours	0.0	0.0	3736.2
16.	Gross Thermal Energy Generated (MWH)	1757284.0	5923128.2	253039804.9
17.	Gross Electrical Energy Generated (MWH)	587245.0	1988825.0	82504013.0
18.	Net Electrical Energy Generated (MWH)	558184.0	1885437.0	78259503.0
19.	Unit Service Factor	100.0%	61.1%	63.6%
20.	Unit Availability Factor	100.0%	61.1%	65.8%
21.	Unit Capacity Factor (Using MDC Net)	99.3%	55.3%	59.0%
22.	Unit Capacity Factor (Using DER Net)	98.4%	54.8%	58.0%
23.	Unit Forced Outage Rate	0.0%	6.3%	19.0%

24. Shutdowns Schedule Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down at End of Report Period Estimated Date of Start-up:

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

OPERATING DATA REPORT

Docket No.: 50-281
Date: 07-07-92
Completed By: D. Mason
Telephone: (804) 365-2459

1. Unit Name: ..... Surry Unit 2
2. Reporting Period: ..... June 1992
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 820
7. Maximum Dependable Capacity (Net MWe): 781

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

		This Month	YTD	Cumulative
11.	Hours In Reporting Period	720.0	4367.0	168023.0
12.	Number of Hours Reactor Was Critical	720.0	4367.0	109575.1
13.	Reactor Reserve Shutdown Hours	0.0	0.0	328.1
14.	Hours Generator On-Line	720.0	4367.0	107827.9
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	1738128.6	10569917.3	251448599.4
17.	Gross Electrical Energy Generated (MWH)	571450.0	3515625.0	81949474.0
18.	Net Electrical Energy Generated (MWH)	542322.0	3344095.0	77708033.0
19.	Unit Service Factor	100.0%	100.0%	64.2%
20.	Unit Availability Factor	100.0%	100.0%	64.2%
21.	Unit Capacity Factor (Using MDC Net)	96.4%	98.0%	59.3%
22.	Unit Capacity Factor (Using DER Net)	95.6%	97.2%	58.7%
23.	Unit Forced Outage Rate	0.0%	0.0%	14.8%

24. Shutdowns Schedule Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down at End of Report Period Estimated Date of Start-up: _____

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWN AND POWER REDUCTION
(EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: June 1992

Docket No.: 50-280
Unit Name: Surry Unit 1
Date: 07-07-92
Completed by: Anthony Xenakis
Telephone: (804) 365-2145

Date	(1) Type	Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	LER No.	(4) System Code	(5) Component Code	Cause & Corrective Action to Prevent Recurrence

No entries for this reporting period.

(1)
F: Forced
S: Scheduled

(2)
REASON:
A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & Licensing Examination
F - Administrative
G - Operational Error (Explain)

(3)
METHOD:
1 - Manual
2 - Manual Scram.
3 - Automatic Scram.
4 - Other (Explain)

(4)
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5)
Exhibit 1 - Same Source.

UNIT SHUTDOWN AND POWER REDUCTION
(EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: June 1992

Docket No.: 50-281
Unit Name: Surry Unit 2
Date: 07-07-92
Completed by: Anthony Xenakis
Telephone: (804) 365-2145

Date	(1) Type	Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	LER No.	(4) System Code	(5) Component Code	Cause & Corrective Action to Prevent Recurrence
920612	S	0	B	4	N/A	SJ	RV	Unit power was reduced to 80% to remove the "B" feedwater heater train from service in order to repair a relief valve (2-CN-RV-212B).

(1)
F: Forced
S: Scheduled

(2)
REASON:
A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & Licensing Examination
F - Administrative
G - Operational Error (Explain)

(3)
METHOD:
1 - Manual
2 - Manual Scram.
3 - Automatic Scram.
4 - Other (Explain)

(4)
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5)
Exhibit 1 - Same Source.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-280
Unit Name: Surry Unit 1
Date: 07-07-92
Completed by: M. A. Negron
Telephone: (804) 365-2795

Month: June 1992

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	780	17	775
2	782	18	774
3	781	19	774
4	780	20	774
5	780	21	776
6	779	22	777
7	778	23	776
8	776	24	775
9	775	25	774
10	774	26	768
11	775	27	771
12	775	28	773
13	774	29	772
14	775	30	771
15	773	31	
16	774		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-281
Unit Name: Surry Unit 2
Date: 07-07-92
Completed by: M. A. Negron
Telephone: (804) 365-2795

Month: June 1992

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	770	17	763
2	770	18	762
3	767	19	761
4	769	20	754
5	768	21	758
6	764	22	768
7	757	23	767
8	754	24	767
9	763	25	765
10	762	26	763
11	761	27	756
12	757	28	758
13	591	29	759
14	666	30	757
15	759	31	
16	761		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: June 1992

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

06-01-92	0000	This reporting period started with the Unit operating at 100% power, 820 MWe.
06-30-92	2400	This reporting period ended with the Unit operating at 100% power, 810 MWe.

UNIT TWO

06-01-92	0000	This reporting period started with the Unit operating at 100% power, 815 MWe.
06-12-92	2200	Started ramp down to remove the "B" Feedwater Heater Train from service in order to repair a relief valve (2-CN-RV-212B).
	2354	Stopped ramp at 80% power, 670 MWe.
06-14-92	0927	Started power increase; 80% power, 625 MWe.
	1733	Unit at 100% power, 800 MWe.
06-30-92	2400	This reporting period ended with the Unit operating at 100% power, 795 MWe.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: June 1992

JCO C-92-002 **Justification For Continued Operation** 03-31-92
(Safety Evaluation No. 92-075)

This Justification For Continued Operation (JCO) assessed the acceptability of manual operator action (through annunciator response procedures) in lieu of automatic equipment functions to ensure the operation of radiation monitoring systems in the event of a loss of power.

The assessment concluded that the actions required to mitigate a loss of power to the radiation monitoring system can be satisfactorily accomplished through the actions contained in the annunciator response procedures. Therefore, an unreviewed safety question is not created.

TM S2-92-006 **Temporary Modification** 06-03-92
(Safety Evaluation No. 92-124)

This Temporary Modification (TM) installed a device to block open secondary plant drain system level control valve, 2-SD-LCV-203A, to facilitate the repair of instrument air leaks.

This TM does not affect the capability of the secondary system to provide normal feedwater to the steam generators. Furthermore, it was determined that excessive cooldown accidents due to feedwater malfunctions remain bounding for this activity. Therefore, an unreviewed safety question is not created.

SE-92-126 **Safety Evaluation** 06-04-92

This Safety Evaluation was performed to determine the acceptability of current and proposed procedures which permit administrative control of the emergency diesel generator air start system cross-connect valves 0-EG-15, 1-EG-15, and 2-EG-15 in order to maintain the air start bottle banks pressurized using a single compressor.

The evaluation concluded that this method of pressurizing the air start bottle banks enhances EDG reliability and that the procedural requirements are adequate to ensure the EDG can remain operable while the subject valves are being administratively controlled. Therefore, an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: June 1992

[continued]

SE-92-127 **Safety Evaluation** 06-04-92

This Safety Evaluation was performed to determine the acceptability and limitations for administratively controlling fire pump recirculation valves 1-FP-36 or 1-FP-37 to allow fire protection water to be used for miscellaneous purposes.

It was determined that use of fire protection in accordance with the controls specified in the procedures will ensure adequate fire suppression water capacity is maintained within the bounds of Technical Specifications and the Appendix R report. Furthermore, the effectiveness of the Emergency Plan, Fire Contingency Action procedures, and Pre Fire-Plans (area specific fire fighting guidance) are not affected by this activity. Therefore, an unreviewed safety question is not created.

DR S-92-0930 **Deviation Report** 06-11-92
(Safety Evaluation No. 92-130)

This Safety Evaluation was performed to evaluate Deviation Report S-92-0930 concerning the installation of a mechanical jumper between condensate polishing valves 1-CP-0320 and 2-CP-0600. This modification was made to enable make-up seal water to enter the low pressure side of the condensate polishing instrument air compressor, 2-CP-C-1, since normal make-up is inadequate during periods of heavy loading.

This modification will be used on a temporary basis while water usage is high (as a result of leakage from 2-CP-C-1). This change affects the nonsafety-related condensate polishing system and does not affect safety-related systems. Therefore, an unreviewed safety question was not created.

TM S1-92-030 **Temporary Modification** 06-12-92
(Safety Evaluation No. 92-91)

This Temporary Modification (TM) installed a mechanical jumper between Unit 2 bearing cooling system valve, 2-BC-303, and Unit 1 ventilation system valve, 1-VS-567, to provide a make-up water supply to the chilled water system loop "C". This configuration allows chilled water loop "A" to be isolated and drained so that piping connections associated with the implementation of Design Change Package 90-07-3 can be made.

The appropriate Technical Specification limiting conditions for operation will be adhered to while this TM is in place and a flood watch will be posted in mechanical equipment room number 3 when the jumper is being used. Therefore, an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: June 1992

[continued]

TM S2-92-007 **Temporary Modification** 06-12-92
(Safety Evaluation No. 92-133)

This Temporary Modification installed a mechanical jumper from safety injection system valve, 2-SI-231, located downstream of the accumulator vent containment isolation valves, to gaseous vent system valve, 2-VG-9, which leads to the overhead gas system. This TM is necessary to vent the pressurizer relief tank (PRT) as a result of the failure of the PRT sample system trip valve, 2-SS-TV-204A.

The mechanical jumper (hose) is rated for system pressure and allows PRT gases to vent to the overhead gas system which is designed for processing, holding, and monitoring radioactive gases. The configuration includes the accumulator vent containment trip valves, which receive the same containment isolation signals as the normal line-up through the PRT sample system trip valve. Therefore, an unreviewed safety question is not created.

TM S2-92-008 **Temporary Modification** 06-14-92
(Safety Evaluation No. 92-134)

This Temporary Modification installed a mechanical jumper from Unit 2 safety injection system valve, 2-SI-233, located upstream of the accumulator vent containment isolation valves, to gaseous vent system valve, 2-VG-5, located upstream of the primary drains transfer tank containment isolation valves. This TM is necessary to vent the pressurizer relief tank (PRT) as a result of the failure of the PRT sample system trip valve, 2-SS-TV-204A.

The mechanical jumper (hose) is rated for system pressure and allows PRT gases to vent to the overhead gas system which is designed for processing, holding, and monitoring radioactive gases. The configuration includes the primary drains tank vent containment trip valves, which receive the same containment isolation signals as the normal line-up through the PRT sample system trip valve. Therefore, an unreviewed safety question is not created.

TM S2-92-009 **Temporary Modification** 06-16-92
(Safety Evaluation No. 92-135)

This Temporary Modification installed a tube from the Unit 2 pressurizer cubicle ventilation column to the vicinity of a leaking pressurizer safety valve. This TM directs cooling air towards the valve in order to lower the valve temperature and reduce seat leakage.

An evaluation of this TM determined that the valve body temperature change is small and has a negligible effect on the valve loop seals. The safety valves will remain operable and the setpoints will not be impacted. Therefore, an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: June 1992

[continued]

DR S-92-1067 **Deviation Report** 06-19-92
(Safety Evaluation No. 92-137)

This Safety Evaluation was performed to evaluate Deviation Report S-92-1067 concerning the use of a flood watch in the Units 1 and 2 turbine buildings. In the event of a loss of off-site power, the flood watch is tasked with notifying the main control room if a turbine building flooding event is occurring.

The evaluation determined that the identification of a flood source and the manual actions required in the main control room can be accomplished within the allowed operator response time. Therefore, an unreviewed safety question is not created.

SE-92-138 **Safety Evaluation** 06-19-92

This Safety Evaluation was performed to evaluate the addition of portable air blowers in the Unit 2 pressurizer cubicle. The blowers will be installed to improve air circulation in the pressurizer cubicle space to lower ambient temperatures. The lower ambient temperatures reduce the pressurizer valve body temperatures to assist in maintaining a loop seal, minimizing leakage past the valve seats.

The evaluation determined that the valve body temperature change is small and has a negligible effect on the valve. The safety valves will remain operable and the setpoints will not be impacted. Therefore, an unreviewed safety question is not created.

FS 90-12 **UFSAR Change** 06-25-92
(Safety Evaluation 92-141)

The Updated Final Safety Analysis Report (UFSAR) Section 8.5, "Emergency Power Station" was revised to clarify the heat tracing description and to remove setpoint information.

The change is administrative in nature and provides clarification only. No procedures are affected and no physical modifications are involved. Therefore an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: June 1992

[continued]

TM S2-92-010 **Temporary Modification** 06-25-92
(Safety Evaluation No. 92-142)

This Temporary Modification (TM) installed a device to block open Unit 2 secondary plant drain system level control valve, 2-SD-LCV-202A, to facilitate the repair of instrument air leaks. This configuration will enable the repairs to be made without diverting the discharge of the first point feedwater heater to the condenser, which would result in a loss in plant efficiency.

This TM does not affect the capability of the secondary system to provide normal feedwater to the steam generators. Furthermore, it was determined that excessive cooldown accidents due to feedwater malfunctions remain bounding for this activity. Therefore, an unreviewed safety question was not created.

PROCEDURE OR METHOD OF OPERATION CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: June 1992

1-OP-50.1
1-OP-50.2

Operating Procedures 06-04-92
(Safety Evaluation No. 92-128)

Operating Procedures 1-OP-50.1, " Removal of Bearing Cooling Water From Service," and 1-OP-50.2, "Placing Bearing Cooling Water In Service," were revised to provide instructions for installing a piping jumper from the domestic water system to the central chilled water system as a source of make-up water while the Unit 1 bearing cooling water system is out of service for maintenance. The piping arrangement was reviewed to ensure that the domestic water system would not be contaminated by the bearing cooling system water.

This change does not impact safety-related systems. The cross-connect capability between the nonsafety-related central chillers and the safety-related chillers is presently isolated and a continuous fire watch is posted to ensure a fire does not affect the safety-related chillers. Therefore, the Appendix R back-up capability assumed for the central chillers is not considered in the accident analyses and an unreviewed safety question is not created.

0-TOP-4053

Temporary Operating Procedure 06-11-92
(Safety Evaluation No. 92-131)

Temporary Operating Procedure (TOP) 0-TOP-4053, "Control Room And Relay Room Ventilation System Operation During Chilled Water Loop A Tie-Ins For DCP 90-07-3", was developed to permit two control room envelope air conditioning system chillers to be operated in parallel to a single chilled water loop. This mode of operation is required to allow chilled water loop "A" to be isolated and drained so that piping connections associated with the implementation of Design Change Package 90-07-3 can be made.

During the performance of this TOP, two of the three chillers will be operable and the appropriate Technical Specification limiting conditions for operation will be observed. Therefore, an unreviewed safety question is not created.

2-OPT-CT-201

Operations Periodic Test Procedure 06-17-92
(Safety Evaluation No. 92-136)

Operations Periodic Test Procedure 2-OPT-CT-201, "Containment Isolation Valve Local Leak Rate Testing (Type C Containment Testing)" was revised ("one-time only" change) to provide instructions for Type C testing of Unit 2 safety injection system trip valve 2-SI-TV-201B.

Containment integrity was maintained during the performance of this procedure by safety injection system valve, 2-SI-313, and safety injection system trip valve, 2-SI-TV-201A, which had its leads lifted to prevent inadvertent opening. Therefore, an unreviewed safety question was not created.

PROCEDURE OR METHOD OF OPERATION CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: June 1992
[continued]

TM S1-92-031
1-IPT-CC-RC-T-432
1-IPT-FT-RC-T-432

Temporary Modification
Instrument Periodic Test Procedures
(Safety Evaluation No. 92-140)

06-23-92

This Temporary Modification (TM) changed the Unit 1 reactor coolant system loop "C" hot leg average temperature ($T_{h\text{-}avg}$) summator to calculate $T_{h\text{-}avg}$ by averaging the data from 2 resistance temperature devices (RTD) rather than 3. This change is necessary since one of the RTDs and its corresponding spare have become unreliable. Instrument Periodic Test Procedures 1-IPT-CC-RC-T-432, "Delta T and T_{AVG} Protection Set III Loop T-432 Channel Calibration" and 1-IPT-FT-RC-T-432, "Delta T and T_{AVG} Protection Set III Loop T-432 Functional Test" were temporarily revised to address the TM.

This change has a small effect on the temperature input to the protection system. However, the small change is within the uncertainties of the channel statistical allowance and the Safety Analysis. Therefore, an unreviewed safety question was not created.

2-PT-18.1

Operations Periodic Test Procedure
(Safety Evaluation No. 92-143)

06-30-92

Unit 2 Operations Periodic Test Procedure 2-PT-18.1, "LHSI Pump Test" was revised ("one-time only" change) to permit the use of temporary transmitters to enable the measurement of Low Head Safety Injection Pump pressure during the few seconds following pump start.

The use of the temporary safety-related transmitter does not affect the operation of the subject pump or the ability of the safety injection system to perform its required safety function. Therefore, an unreviewed safety question is not created.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: June 1992

ST-301 **Auxiliary Ventilation Filter Train Test** 06-03-92
(Safety Evaluation No. 92-125)

This special test was performed to collect data with the ventilation system aligned in a safety features mode with four charging pump cubicles dampers open. The test results indicated that operation of the ventilation system with four dampers open does not impact the performance of the system in the safety features mode.

The motor operated dampers (MODs) were maintained open during the test by removing fuses located in the control room. Electricians were stationed to immediately replace the fuses and restore power to the MODs, at the direction of the shift supervisor. The time required to perform this manual action is bounded by the ventilation system refueling mode analysis. Operating fan bearing temperatures were also monitored closely to ensure fan operability was maintained during the test. Therefore, an unreviewed safety question was not created.

CHEMISTRY REPORT

MONTH/YEAR: June 1992

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioact., μCi/ml	3.60E-1	1.84E-1	2.68E-1	2.43E-1	1.42E-1	1.90E-1
Suspended Solids, ppm	≤ 0.1	≤ 0.1	≤ 0.1	≤ 0.1	≤ 0.1	≤ 0.1
Gross Tritium, μCi/ml	4.17E-1	2.49E-1	3.26E-1	6.22E-1	5.25E-1	5.73E-1
I^{131}, μCi/ml	1.25E-3	4.30E-4	5.83E-4	1.18E-3	2.26E-4	3.99E-4
I^{131}/I^{133}	0.11	0.07	0.09	0.12	0.06	0.09
Hydrogen, cc/kg	43.8	28.7	35.0	39.3	22.2	33.2
Lithium, ppm	2.34	2.06	2.18	2.32	2.06	2.16
Boron - 10, ppm*	237.7	231.3	234.3	144.4	126.4	135.5
Oxygen, (DO), ppm	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005
Chloride, ppm	0.006	0.004	0.005	0.005	≤ 0.001	0.002
pH at 25 degree Celsius	6.46	6.26	6.38	6.84	6.61	6.72

* Boron - 10 = Total Boron x 0.196

Comments:

None.

FUEL HANDLING
UNITS 1 & 2

MONTH/YEAR: June 1992

New or Spent Fuel Shipment Number	Date Stored or Received	Number for Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity

No Fuel Stored or Received in June, 1992

DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

MONTH/YEAR: June 1992

None During This Reporting Period.