

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION

MONTHLY OPERATING REPORT

REPORT 88-01

APPROVED: David L. Benson  
STATION MANAGER

8802220166 880131  
PDR ADCK 05000280  
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OPERATING DATA REPORT

DOCKET NO. 50-280  
 DATE 2/5/88  
 COMPLETED BY L. A. Warren  
 TELEPHONE 804-357-3184

OPERATING STATUS

1. Unit Name: Surry I
2. Reporting Period: Jan. 01 thru Jan. 31, 1988
3. Licensed Thermal Power (Mwt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 820
7. Maximum Dependable Capacity (Net MWe): 781
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: \_\_\_\_\_

Notes

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.0</u>	<u>744.0</u>	<u>132456.0</u>
12. Number of Hours Reactor Was Critical	<u>744.0</u>	<u>744.0</u>	<u>85467.4</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>3774.5</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>744.0</u>	<u>83715.2</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>3736.2</u>
16. Gross Thermal Energy Generated (MWH)	<u>1811710.2</u>	<u>1811710.2</u>	<u>194531931.6</u>
17. Gross Electrical Energy Generated (MWH)	<u>614935</u>	<u>614935</u>	<u>62990108.0</u>
18. Net Electrical Energy Generated (MWH)	<u>585817</u>	<u>585817</u>	<u>59741193.0</u>
19. Unit Service Factor	<u>100%</u>	<u>100%</u>	<u>63.2%</u>
20. Unit Available Factor	<u>100%</u>	<u>100%</u>	<u>66%</u>
21. Unit Capacity Factor (Using MDC Net)	<u>100.8%</u>	<u>100.8%</u>	<u>58.3%</u>
22. Unit Capacity Factor (Using DER Net)	<u>99.9%</u>	<u>99.9%</u>	<u>57.2%</u>
23. Unit Forced Rate	<u>0%</u>	<u>0%</u>	<u>17.5%</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
Refueling Outage April 8, 48 Days

25. If Shut Down At End Of Report Period Estimated Date of Startup: \_\_\_\_\_

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATING DATA REPORT

DOCKET NO. 50-281  
 DATE 2/5/88  
 COMPLETED BY L. A. Warren  
 TELEPHONE 804-357-3184

OPERATING STATUS

1. Unit Name: Surry 2
2. Reporting Period: Jan. 01 thru Jan. 31, 1988
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 820
7. Maximum Dependable Capacity (Net MWe): 781
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: \_\_\_\_\_

Notes
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9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.0</u>	<u>744.0</u>	<u>129336.0</u>
12. Number of Hours Reactor Was Critical	<u>744.0</u>	<u>744.0</u>	<u>85410.0</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>328.1</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>744.0</u>	<u>84042.3</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1811065.7</u>	<u>1811065.7</u>	<u>196981428.4</u>
17. Gross Electrical Energy Generated (MWH)	<u>606540.0</u>	<u>606540.0</u>	<u>63979364.0</u>
18. Net Electrical Energy Generated (MWH)	<u>577580.0</u>	<u>577580.0</u>	<u>60654057.0</u>
19. Unit Service Factor	<u>100%</u>	<u>100%</u>	<u>65%</u>
20. Unit Available Factor	<u>100%</u>	<u>100%</u>	<u>65%</u>
21. Unit Capacity Factor (Using MDC Net)	<u>99.4%</u>	<u>99.4%</u>	<u>60.2%</u>
22. Unit Capacity Factor (Using DER Net)	<u>98.5%</u>	<u>98.5%</u>	<u>59.5%</u>
23. Unit Forced Rate	<u>0%</u>	<u>0%</u>	<u>14.3%</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	_____		

25. If Shut Down At End Of Report Period Estimated Date of Startup: \_\_\_\_\_

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-280  
 UNIT NAME Surry 1  
 DATE 02/05/88  
 COMPLETED BY L. A. Warren  
 TELEPHONE 804-357-3184

REPORT MONTH JANUARY 1988

NO.	DATE	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	LICENSEE EVENT REPORT #	System Code <sup>4</sup>	Component Code <sup>5</sup>	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
88-01	1/9/88	S	0.0	B	1				Unit was reduced to 69% power, 572 MW's to allow monthly testing of turbine valves (PT-29.1)

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & License Examination  
 F - Administrative  
 G - Operational Error (Explain)  
 H - Other (Explain)

<sup>3</sup>  
 Method:  
 1 - Manual  
 2 - Manual Scram.  
 3 - Automatic Scram.  
 4 - Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File  
 (NUREG 0161)

<sup>5</sup>  
 Exhibit 1 - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-281  
 UNIT NAME Surry 2  
 DATE 02/05/88  
 COMPLETED BY L. A. Warren  
 TELEPHONE 804-357-3184

REPORT MONTH JANUARY 1988

NO.	DATE	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	LICENSEE EVENT REPORT #	System Code <sup>4</sup>	Component Code <sup>5</sup>	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
88-01	01/23/88	S	0.0	B	1				Unit was reduced to 70% power, 590 MW's to allow monthly testing of turbine valves (PT-29.1)

<sup>1</sup>  
**F: Forced**  
**S: Scheduled**

<sup>2</sup>  
**Reason:**  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & License Examination  
 F - Administrative  
 G - Operational Error (Explain)  
 H - Other (Explain)

<sup>3</sup>  
**Method:**  
 1 - Manual  
 2 - Manual Scram.  
 3 - Automatic Scram.  
 4 - Other (Explain)

<sup>4</sup>  
**Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)**

<sup>5</sup>  
**Exhibit 1 - Same Source**

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-280  
 UNIT Surry 1  
 DATE 2/08/88  
 COMPLETED BY L. A. Warren  
 TELEPHONE 804-357-3184

MONTH JANUARY 1988

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>787</u>	17	<u>791</u>
2	<u>787</u>	18	<u>791</u>
3	<u>788</u>	19	<u>791</u>
4	<u>788</u>	20	<u>791</u>
5	<u>787</u>	21	<u>791</u>
6	<u>787</u>	22	<u>791</u>
7	<u>788</u>	23	<u>791</u>
8	<u>788</u>	24	<u>791</u>
9	<u>728</u>	25	<u>790</u>
10	<u>789</u>	26	<u>791</u>
11	<u>788</u>	27	<u>791</u>
12	<u>789</u>	28	<u>790</u>
13	<u>788</u>	29	<u>791</u>
14	<u>789</u>	30	<u>791</u>
15	<u>789</u>	31	<u>790</u>
16	<u>790</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-281  
 UNIT Surry 2  
 DATE 2/08/88  
 COMPLETED BY L. A. Warren  
 TELEPHONE 804-357-3184

MONTH JANUARY 1988

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>779</u>	17	<u>778</u>
2	<u>778</u>	18	<u>779</u>
3	<u>778</u>	19	<u>778</u>
4	<u>777</u>	20	<u>778</u>
5	<u>778</u>	21	<u>779</u>
6	<u>778</u>	22	<u>778</u>
7	<u>778</u>	23	<u>731</u>
8	<u>779</u>	24	<u>776</u>
9	<u>777</u>	25	<u>777</u>
10	<u>778</u>	26	<u>776</u>
11	<u>778</u>	27	<u>776</u>
12	<u>778</u>	28	<u>777</u>
13	<u>779</u>	29	<u>777</u>
14	<u>778</u>	30	<u>778</u>
15	<u>778</u>	31	<u>779</u>
16	<u>778</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)



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**SUMMARY OF OPERATING EXPERIENCE**

**MONTH/YEAR** January 1988

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

**UNIT ONE**

01-01-88	0000	This reporting period begins with the unit at 100% power, 830MW's.
01-09-88	1000	Commenced power reduction at 150MW/hr. for scheduled testing of turbine valves (PT-29.1).
	1136	Holding power at 69% power, 572 MW's.
	1445	Commenced power increase at 150 MW/hr.
	1845	Unit at 100% power, 830 MW's.
01-31-88	2400	This reporting period ends with the unit at 100% power, 830 MW's.

**UNIT TWO**

01-01-88	0000	This reporting period begins with the unit at 100% power, 820 MW's.
01-23-88	1213	Commenced power reduction at 150 MW's/hr for scheduled testing of turbine valves (PT-29.1).
	1346	Holding power at 70%, 590 MW's.
	1650	Commenced power increase at 150 MW/hr.
	1930	Unit at 100% power, 815 MW's.
01-31-88	2400	This reporting period ends with the unit at 100% power, 815 MW's.

**FACILITY CHANGES REQUIRING NRC APPROVAL**

**MONTH/YEAR JANUARY 1988**

**NONE DURING THIS PERIOD**

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**  
**MONTH/YEAR January 1988**

**EWR 87-331      ADDITION OF FP CARDOX ODORIZER      UNITS 1 & 2      1/29/88**

This engineering work request added an odorizer to the Cardox System to enhance worker safety by providing another means of detecting a discharge of the system.

The addition of odorizers does not effect the function of the Cardox System. Therefore, the enhancement does not impact plant or public health and safety nor result in an unreviewed safety question.

**EWR 87-400      REPLACE MAIN STEAM VALVE (1-MS-104) UNIT 1      1/12/88**

This engineering work request installed an equivalent safety related gate valve to replace 1-MS-104, an isolation globe valve for the auxiliary feedwater pump steam flow transmitter which had a bonnet steam cut.

The replacement valve is of the same pressure class and material as the original valve. Therefore, there are no changes in the probability or consequences of accidents analyzed in the UFSAR, nor has the possibility of a new accident being created. The margin of safety is not affected.

**EWR 87-403      MODIFY FEEDWATER VALVES      UNITS 1 & 2      1/05/88**  
**(1/2-FW-MOV-150A, B/250A, B)**

This engineering work request evaluated modifications to the Unit 1 and 2 feedwater pump discharge valves (1-FW-MOV-150 A/B and 2-FW-MOV-250 A/B) to prevent the operator mounting studs from pulling loose from the operator body.

The installation of: 1) all thread rod studs to ensure thread engagement, 2) one new stud between existing holes for support and 3) a new yoke flange will not result in an unreviewed safety question.

**EWR 87-410      EVALUATE GLAND BOLT MATERIAL CHANGE      1/05/88**  
**(1-CH-163, 164) UNIT 1**

This engineering work request compared chemical and mechanical properties of the old (A193) and the new (A582) gland bolt materials for Westinghouse valves (2" - T58) to determine if the new material was acceptable.

An analysis of the impact of the valve failure as analyzed in UFSAR was made. As the material's properties are equivalent, there is no effect on the UFSAR analyses.

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**  
**MONTH/YEAR January 1988**

<b>N/A</b>	<b>SCAFFOLD REQUEST</b>	<b>1/29/88</b>
	<p>A temporary scaffold platform was constructed in #1 EDG Room to work #1 Diesel Room Dampers.</p> <p>Installation of this temporary scaffolding was reviewed for affect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.</p>	
<b>N/A</b>	<b>SCAFFOLD REQUEST</b>	<b>1/20/88</b>
	<p>A temporary scaffold platform was constructed in Auxiliary Building to work FT-VS-118 and FT-VS-218.</p> <p>Installation of this temporary scaffolding was reviewed for affect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.</p>	
<b>TM S1-88-2</b>	<b>TEMPORARY MODIFICATION</b>	<b>1/29/88</b>
	<p>Temporary modification removed and blanked the piping connection on the PG tank heat exchanger thermal relief valves, 1-RV-BR-114A, B.</p> <p>It was determined that maintaining the PG tank heat exchanger inlet and outlet isolation valves open prevents heat exchanger overpressurization while the thermal relief valves are removed. Therefore, the radioactive liquid release probability is not increased.</p>	
<b>TM S2-88-2</b>	<b>TEMPORARY MODIFICATION</b>	<b>1/07/88</b>
	<p>This temporary procedure closed manual valve 2-SW-117 and placed it under administrative control to prevent over-cooling of lube oil for 2-CH-P-1B while the pump is off.</p> <p>Closure of 2-SW-117 does not constitute an unreviewed safety question because it is under administrative control ensuring it will be opened if needed before operability of any plant equipment is affected.</p>	

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**  
**MONTH/YEAR January 1988**

- TM S2-88-3            TEMPORARY MODIFICATION            1/11/88**
- This temporary modification closed manual valve 2-SW-168 (lube oil cooler SW isolation) under administrative control to prevent over cooling of lube oil for 2-CH-P-1A while the pump is shut down.
- Closure of 2-SW-168 does not constitute an unreviewed safety question since it is under administrative control ensuring it will be opened, if required, before operability of any plant equipment is affected.
- TM S2-88-5            TEMPORARY MODIFICATION            1/26/88**
- This temporary modification placed the emergency borate valve, 2-CH-MOV-2350, under administrative control because the normal boric acid flow path from the blender was inoperable due to a heat tracing failure.
- Placing 2-CH-MOV-2350 under administrative control will ensure two boric acid flow paths are available to the charging pump suction as required.
- TM S2-88-7            TEMPORARY MODIFICATION            1/26/88**
- This temporary modification locked closed 2-RC-133 to isolate containment penetration #57 and the pressurizer vent and vapor space sample.
- The pressurizer vent system is used as a backup bleed path in case of a loss of heat sink transient. However, the preferred path remained available. The pressurizer vent system can also be used to vent non-condensable gases from the pressurizer vapor space. However, the PORV's remain available if this function is required.

**PROCEDURE OR METHOD OF OPERATION CHANGES**  
**THAT DID NOT REQUIRE NRC APPROVAL**  
**MONTH/YEAR January 1988**

**2-TOP-2015            AUXILIARY FEEDWATER CHECK VALVE            1/05/88**  
**LEAKAGE GUIDELINES                            UNIT 2**

This temporary operating procedure isolated steam generator auxiliary feedwater flow paths on Unit 2 to determine potential check valve leakage paths.

Since the flow paths were under administrative control during the temporary operations and since automatic actuation will return the system to the required operating condition, it was determined that an unreviewed safety question was not created.

**2-TOP-2015            AUXILIARY FEEDWATER CHECK VALVE            1/26/88**  
**LEAKAGE GUIDELINE                                UNIT 2**

The temporary operating procedure was revised to extend the implementation date.

The administrative control and automatic actuations remain in place; therefore, the evaluation above remains valid.

**SUADM-ADM-21        STATION PROCEDURES                                1/26/88**

This procedure revision updated the procedure deviation process to reflect the 10CFR50.59 screening and review process.

The updated procedure incorporated more stringent requirements governing procedure deviations than outlined in the administration section of Technical Specification (section 6.4)

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR JANUARY 1988

NONE DURING THIS PERIOD

TESTS AND EXPERIMENTS REQUIRING NRC APPROVAL

MONTH/YEAR JANUARY 1988

NONE DURING THIS PERIOD



OTHER CHANGES, TEST AND EXPERIMENTS

MONTH/YEAR JANUARY 1988

NONE DURING THIS PERIOD

**VIRGINIA POWER**  
**SURRY POWER STATION**  
**CHEMISTRY REPORT**

JANUARY 19 88

PRIMARY COOLANT ANALYSIS	UNIT NO. 1			UNIT NO. 2		
	MAX.	MIN.	AVG.	MAX.	MIN.	AVG.
Gross Radioact., $\mu\text{Ci/ml}$	1.42	7.78E-1	1.24	2.72E-1	1.27E-1	2.03E-1
Suspended Solids, ppm	0.0	0.0	0.0	0.0	0.0	0.0
Gross Tritium, $\mu\text{Ci/ml}$	7.82E-2	3.22E-2	5.43E-2	1.32E-1	2.78E-2	9.42E-2
Iodine <sup>131</sup> , $\mu\text{Ci/ml}$	1.48E-2	6.41E-3	9.87E-3	2.65E-4	7.88E-5	1.40E-4
I <sup>131</sup> / I <sup>131</sup>	0.26	0.12	0.19	0.13	0.06	0.09
Hydrogen, cc/kg	39.3	31.0	33.3	34.7	30.3	32.0
Lithium, ppm	0.66	0.41	0.54	1.05	0.77	0.91
Boron-10, ppm*	37.6	15.1	25.1	87.2	70.2	78.2
Oxygen, (DO), ppm	<0.005	<0.005	<0.005	<0.005	<0.005	<0.005
Chloride, ppm	0.010	0.001	0.003	0.010	0.002	0.003
pH @ 25 degree Celsius	7.05	6.52	6.81	6.70	6.46	6.56

\* Boron-10 = Total Boron X 0.196

REMARKS: Unit 1: No LiOH additions made in January, Deborating Bed placed in service for LiOH removal on 1-04-88 from 1115 to 1247; 1-19-88 from 1250 to 1430; 1-22-88 from 0950 to 1208; 1-29-88 from 1820 to 2031.

Unit 2: No LiOH additions made in January. Cation Bed placed in service for LiOH removal on 1-3-88 from 0515 to 0844; 1-7-88 from 0945 to 1321; 1-9-88 from 1810 to 2239; 1-12-88 from 0925 to 1204; 1-15-88 from 0950 to 1326; 1-19-88 from 1810 to 2040; 1-22-88 from 1535 to 1705.

UNIT 1

## FUEL HANDLING

DATE JAN 1988

NEW OR SPENT FUEL SHIPMENT #	DATE SHIPPED OR RECEIVED	NUMBER OF ASSEMBLIES PER SHIPMENT	ASSEMBLY #	ANSI #	INITIAL ENRICHMENT	NEW OR SPENT FUEL SHIPPING CASK ACTIVITY LEVEL
Cask #6	01-05-88	21	B02	N/A	2.57	N/A
Cask #6	01-05-88	21	B03	N/A	2.57	N/A
Cask #6	01-05-88	21	B05	N/A	2.57	N/A
Cask #6	01-05-88	21	B11	N/A	2.57	N/A
Cask #6	01-05-88	21	B13	N/A	2.57	N/A
Cask #6	01-05-88	21	B37	N/A	2.57	N/A
Cask #6	01-05-88	21	B46	N/A	2.57	N/A
Cask #6	01-05-88	21	B51	N/A	2.57	N/A
Cask #6	01-05-88	21	C05	N/A	3.12	N/A
Cask #6	01-05-88	21	C16	N/A	3.12	N/A
Cask #6	01-05-88	21	C22	N/A	3.12	N/A
Cask #6	01-05-88	21	C23	N/A	3.12	N/A
Cask #6	01-05-88	21	C29	N/A	3.12	N/A
Cask #6	01-05-88	21	C30	N/A	3.12	N/A
Cask #6	01-05-88	21	C31	N/A	3.12	N/A
Cask #6	01-05-88	21	C33	N/A	3.12	N/A
Cask #6	01-05-88	21	C34	N/A	3.12	N/A
Cask #6	01-05-88	21	C39	N/A	3.12	N/A
Cask #6	01-05-88	21	C41	N/A	3.12	N/A
Cask #6	01-05-88	21	C45	N/A	3.12	N/A
Cask #6	01-05-88	21	C46	N/A	3.12	N/A



PROCEDURE REVISIONS THAT CHANGED THE  
OPERATING MODE DESCRIBED IN THE FSAR

MONTH/YEAR JANUARY 1988

NONE DURING THIS PERIOD

DESCRIPTION OF PERIODIC TEST WHICH WERE NOT COMPLETED  
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

MONTH/YEAR JANUARY 1988

NONE DURING THIS PERIOD

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

W. L. STEWART  
VICE PRESIDENT  
NUCLEAR OPERATIONS

February 15, 1988

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

Serial No. 88-075  
NO/DAS:vlh  
Docket Nos. 50-280  
50-281  
License Nos. DPR-32  
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY  
SURRY POWER STATION UNITS 1 AND 2  
MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of January 1988.

Very truly yours,



W. L. Stewart

Enclosure

cc: U. S. Nuclear Regulatory Commission  
101 Marietta Street, N. W.  
Suite 2900  
Atlanta, GA 30323

Mr. W. E. Holland  
NRC Senior Resident Inspector  
Surry Power Station

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