

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

March 12, 1997

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Serial No. 97-151
NL/RPC
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

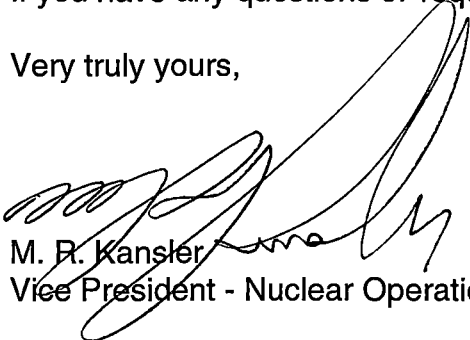
Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of February 1997.

If you have any questions or require additional information, please contact us.

Very truly yours,


M. R. Kansler
Vice President - Nuclear Operations

Enclosure

cc: U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, N.W.
Suite 2900
Atlanta, Georgia 30323

Mr. R. A. Musser
NRC Senior Resident Inspector
Surry Power Station

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VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION
MONTHLY OPERATING REPORT
REPORT NO. 97-02

Approved:


Station Manager

3-15-97
Date

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OPERATING DATA REPORT

Docket No.: 50-280
 Date: 03/04/97
 Completed By: D. K. Mason
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 1
- 2. Reporting Period:..... February, 1997
- 3. Licensed Thermal Power (MWt): 2546
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe): 788
- 6. Maximum Dependable Capacity (Gross MWe):.... 840
- 7. Maximum Dependable Capacity (Net MWe): 801

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting Period	672.0	1416.0	212040.0
12. Number of Hours Reactor Was Critical.....	592.1	1152.0	147986.7
13. Reactor Reserve Shutdown Hours	0.0	0.0	3774.5
14. Hours Generator On-Line	571.1	1130.5	145661.5
15. Unit Reserve Shutdown Hours	0.0	0.0	3736.2
16. Gross Thermal Energy Generated (MWH)	1421706.7	2835195.0	341470638.8
17. Gross Electrical Energy Generated (MWH).....	473530.0	946965.0	111919783.0
18. Net Electrical Energy Generated (MWH).....	457434.0	915225.0	106508974.0
19. Unit Service Factor	85.0%	79.8%	68.7%
20. Unit Availability Factor	85.0%	79.8%	70.5%
21. Unit Capacity Factor (Using MDC Net)	85.0%	80.7%	64.6%
22. Unit Capacity Factor (Using DER Net)	86.4%	82.0%	63.7%
23. Unit Forced Outage Rate.....	15.0%	20.2%	15.3%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Refueling, March 6, 1997, 37 Days

25. If Shut Down at End of Report Period, Estimated Date of Start-up: _____

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATING DATA REPORT

Docket No.: 50-281
 Date: 03-04-97
 Completed By: D. K. Mason
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 2
- 2. Reporting Period:..... February, 1997
- 3. Licensed Thermal Power (MWt): 2546
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe): 788
- 6. Maximum Dependable Capacity (Gross MWe):.... 840
- 7. Maximum Dependable Capacity (Net MWe): 801

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting Period	672.0	1416.0	208920.0
12. Number of Hours Reactor Was Critical.....	608.0	1352.0	144427.6
13. Reactor Reserve Shutdown Hours	0.0	0.0	328.1
14. Hours Generator On-Line	601.1	1345.1	142442.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1516112.5	3409444.4	334883701.2
17. Gross Electrical Energy Generated (MWH)	508440.0	1143970.0	109594769.0
18. Net Electrical Energy Generated (MWH)	492371.0	1106589.0	104298468.0
19. Unit Service Factor	89.4%	95.0%	68.2%
20. Unit Availability Factor	89.4%	95.0%	68.2%
21. Unit Capacity Factor (Using MDC Net)	91.5%	97.6%	63.9%
22. Unit Capacity Factor (Using DER Net)	93.0%	99.2%	63.4%
23. Unit Forced Outage Rate.....	10.6%	5.0%	12.4%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down at End of Report Period, Estimated Date of Start-up: _____ N/A

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

**UNIT SHUTDOWN AND POWER REDUCTION
(EQUAL TO OR GREATER THAN 20%)**

REPORT MONTH: February, 1997

Docket No.: 50-280

Unit Name: Surry Unit 1

Date: 03-05-97

Completed by: M. J. Fanguy

Telephone: (804) 365-2155

(1) Date	(1) Type	(2) Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	(4) LER No.	(4) System Code	(5) Component Code	(5) Cause & Corrective Action to Prevent Recurrence
02/19/97	F	NA	A	NA		AB	EHTR	Start ramp down due to "C" group of Pzr heaters failing.
02/19/97	F	62.7	A	2	97-003	AB	EHTR	Manual Rx trip for same reason.

(1)
F: Forced
S: Scheduled

(2)
REASON:
A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & Licensing Examination
F - Administrative
G - Operational Error (Explain)

(3)
METHOD:
1 - Manual
2 - Manual Scram
3 - Automatic Scram
4 - Other (Explain)

(4)
Exhibit G - Instructions for Preparation of Data Entry Sheets
for Licensee Event Report (LER) File (NUREG 0161)

(5)
Exhibit 1 - Same Source

**UNIT SHUTDOWN AND POWER REDUCTION
(EQUAL TO OR GREATER THAN 20%)**

REPORT MONTH: February, 1997

Docket No.: 50-281
 Unit Name: Surry Unit 2
 Date: 03-05-97
 Completed by: M. J. Fanguy
 Telephone: (804) 365-2155

(1) Date	(1) Type	(2) Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	LER No.	(4) System Code	(5) Component Code	Cause & Corrective Action to Prevent Recurrence
2/18/97	F	70.9	A	2	97-001	TG	JX	Manual Rx trip due to partial loss of power to the EHC panel and #4 governor valve going closed with other starting to go closed.

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets
 for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-280
 Unit Name: Surry Unit 1
 Date: 03-03-97
 Completed by: J. D. Kilmer
 Telephone: (804) 365-2792

MONTH: February, 1997

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	0	17	825
2	132	18	815
3	733	19	755
4	827	20	0
5	826	21	0
6	826	22	169
7	826	23	770
8	825	24	824
9	825	25	825
10	824	26	825
11	825	27	822
12	824	28	813
13	825		
14	825		
15	825		
16	825		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-281

Unit Name: Surry Unit 2

Date: 03-03-97

Completed by: John D. Kilmer

Telephone: (804) 365-2792

MONTH: February, 1997

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	827	17	830
2	827	18	511
3	828	19	0
4	828	20	0
5	828	21	145
6	829	22	782
7	829	23	828
8	829	24	829
9	830	25	828
10	830	26	830
11	830	27	825
12	829	28	827
13	831		
14	831		
15	830		
16	830		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: February, 1997

The following chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE:

02/01/97	0000	Unit 1 begins the month off-line for repairs on "B" Main Steam line.
	2306	Reactor critical.
02/02/97	1410	Unit on-line.
02/03/97	1754	Reactor at 100%, 853 MWe.
02/19/97	2052	Start power reduction at 255 MWe/hr from 855 MWe due to decrease in RCS pressure. "C" group Pressurizer heaters failed causing decrease in RCS pressure.
	2259	Manual Reactor trip from 55%, 430 MWe.
02/22/97	0748	Reactor critical.
	1342	Unit on-line.
02/23/97	1353	Unit 1 at 100%, 850 MWe.
02/27/97	1900	Commence unit coastdown at end of life maintaining Tave median at Tref.
02/28/97	2400	Unit finishes month at 98.5%, 842 MWe.

UNIT Two

02/01/97	000	Unit 2 starts the month at 100%, 855 MWe.
02/18/97	1449	Manual Reactor trip due to partial loss of power to the EHC panel and #4 governor valve going closed with the others starting to go closed.
02/21/97	0650	Reactor is critical.
02/22/97	0553	Unit 2 is at 100%, 855 MWe.
02/28/97	2400	Unit 2 finishes the month at 100%, 855 MWe.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: February, 1997

FS 97-06 **Updated Final Safety Analysis Report Change** 2-7-97
(Safety Evaluation 97-015)

Updated Final Safety Analysis Report Change FS 97-06 revised Section 9.4, "Component Cooling Systems," to correct discrepancies noted during a review of this section of the UFSAR.

This change was administrative in nature and was made to accurately reflect the current plant condition. None of the changes to the UFSAR increases the probability of occurrence or increases the consequence of an accident. Therefore, an unreviewed safety question does not exist.

DCP 97-01 **Design Control Package** 2-11-97
(Safety Evaluation 97-004)

During a run of the 58 B fan, it spuriously tripped on low flow. The reason for this trip was identified as a flow switch (1-VS-FS-117B) being out of calibration. The switch provides the low flow trip for the 58B fan. The flow switch was replaced, however, the replacement switch could not be calibrated. Location of the switch was identified as a contributing factor to the calibration problems since it did not provide a representative flow condition.

The DCP moves the switch to a location having a more representative flow. The function of the flow switch is unchanged. Therefore, an unreviewed safety question did not exist.

FS 97-10 **Updated Final Safety Analysis Report Change** 2-13-97
(Safety Evaluation 97-017)

Updated Final Safety Analysis Report Change 97-10 revised Section 11.2, "Radioactive Waste Systems," to correct a discrepancy identified by Station Deviation S-96-2810. The UFSAR incorrectly describes normal use of the Catalytic Recombiner in Section 11.2.2 and correctly describes its non-use (abandoned in place) in Sections 11.2.5.1 and 11.2.5.3.1.

The implications of operating Surry Power Station Radioactive Waste Systems different than the original intent of the Boron Recovery System and Catalytic Recombiner are bounded by analysis. Liquid releases are bounded by federal regulation. Monitoring release rates and inventories in accordance with Technical Specifications and the Offsite Dose Control Manual guidelines ensure that an unreviewed safety question does not exist.

FS 96-12 **Updated Final Safety Analysis Report Change** 2-17-97
(Safety Evaluation 96-034 Rev. 2)

Updated Final Safety Analysis Report Change FS 96-12 revised various sections that address fuel handling to accurately reflect current station practices.

None of the changes represented significant deviations from the design basis of the fuel handling system and did not reduce the Technical Specifications margin of safety. The changes also did not affect any of the assumptions made in any fuel handling or other design basis accident analyses. Therefore, an unreviewed safety question does not exist.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: February, 1997

- FS 96-22 R1 **Updated Final Safety Analysis Report Change** 2-17-97
(Safety Evaluation 96-059 Rev. 1)
- Updated Final Safety Analysis Report Change FS 96-22 revised various sections that address fuel handling to accurately reflect current station practices.
- None of the changes represented significant deviations from the design basis of the fuel handling system and did not reduce the Technical Specifications margin of safety. The changes also did not affect any of the assumptions made in any fuel handling or other design basis accident analyses. Therefore, an unreviewed safety question does not exist.
- SE 97-018 **Safety Evaluation** 2-17-97
(Safety Evaluation No. 97-018)
- This Safety Evaluation assesses a circulating water pump being out of service for greater than 30 days. Circulating water pumps provide water required to condense turbine exhaust steam and supply water to the Service Water System.
- This condition meets the requirements of Technical Specifications and adequate intake level is maintained by the remaining operable circulating water pumps. Therefore, an unreviewed safety question does not exist.
- SE 97-019 **Safety Evaluation** 2-17-97
(Safety Evaluation No. 97-019)
- This Safety Evaluation assesses equipment abandoned in place. None of the equipment abandoned is used in any of the analyzed transients.
- The abandoned equipment is not used to mitigate or monitor a design basis accident. The Technical Specification margin of safety was not affected by this activity. Therefore, an unreviewed safety question did not exist.
- FS 97-04 **Updated Final Safety Analysis Report Change** 2-17-97
(Safety Evaluation 97-020)
- UFSAR Section 6.3.1.2.1 states that "This system contains the sodium hydroxide solution only while operating during an accident, which is a period of approximately 30 minutes." This statement implies that no sodium hydroxide is present in containment spray piping during normal plant operations. This statement is incorrect due to minor leakage past the chemical addition tank isolation motor-operated valve.
- An engineering evaluation documented in ET MAT 97-001, concludes that the presence of sodium hydroxide in low temperature and low pressure environments will not have any deleterious effects on 304 SS, sensitized or non-sensitized. Therefore, an unreviewed safety question does not exist.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: February, 1997

FS 97-05 **Updated Final Safety Analysis Report Change** 2-18-97
(Safety Evaluation 97-016)

UFSAR Section 4.2 and 9.1 currently provides only normal steady state RCS dissolved hydrogen values of 25-50 cc/Kg. This revision to the UFSAR will expand the range to include startup, shutdown and off-normal operating conditions.

The proposed change will not change the existing normal range for dissolved hydrogen, but will allow operation of the RCS with hydrogen concentrations >5 cc/Kg during off-normal conditions without reactor shutdown. No significant chemistry changes are expected for operation > 5 cc/Kg, and when operation is \leq 15 cc/Kg, station chemistry will increase RCS monitoring to ensure the chemistry remains unaffected. Because RCS chemistry will not change significantly, the probability and/or consequence of accidents will not increase and the probability of a new unreviewed accident is not increased. Therefore, an unreviewed safety question did not exist.

FS 97-07 **Updated Final Safety Analysis Report Change** 2-19-97
(Safety Evaluation 97-023)

UFSAR Chapters 6 and 15 are being revised to reflect the current design of the Surry Containment ground water control equipment, Containment ground water protection methods and liquid level alarms. This revision is based upon an engineering review of the UFSAR, Station Design Drawings, Design Change History, Station Procedures and Regulatory History.

No physical changes to the Station, procedures or methods of operation are involved with this change. The changes made do not affect the safety related systems, components or other items important to safe operation. Therefore, an unreviewed safety question did not exist.

FS 97-01 **Updated Final Safety Analysis Report Change** 2-24-97
(Safety Evaluation 97-026)

UFSAR Section 9B.2, "Control of Heavy Loads," is being revised to include the spent fuel pool transfer canal gate lifting rig.

Adding the spent fuel pool transfer canal gate lifting rig to the UFSAR maintains our commitments to NUREG-0612. The purpose of compliance with NUREG-0612 is to provide defense-in-depth for controlling heavy loads in order to prevent load drops. Therefore, an unreviewed safety question did not exist.

TM S2-97-001 **Temporary Modification** 2-28-97
(Safety Evaluation No. 97-030)

This Temporary Modification (TM) provides a temporary discharge flow path for the Unit 2 Service Water Radiation Monitoring (RM) Pumps while the outlet Motor Operated Valves from the Recirculation Spray Heat Exchangers are removed for inspection, pipe cleaning and recoating.

The Engineering evaluation concludes that the temporary hose and fittings used to redirect RM Pump flow to the proposed spare penetration are of sufficient diameter to provide rated flow and are adequately rated to meet or exceed system design conditions. The temporary installation will be functionally tested to ensure the system operates as designed. The system will also be tested once the temporary installation has been removed. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: February, 1997

01-OP-RC-011 Rev. 3

Operating Procedure
(Safety Evaluation No. 96-095 Rev. 5)

Operating Procedure 01-OP-RC-011 Rev. 3, "Pressurizer Relief Tank Operations," aids in degassing for the Unit 1 Refueling Shutdown. The existing Unit 2 continuous Pressurizer Relief Tank (PRT) to Overhead vent will be secured and a procedurally controlled temporary modification made to vent the Unit 1 PRT. The duration of the Unit 1 PRT vent will be finite and the Unit 2 PRT continuous vent will be re-established. Unit 1's PRT shutdown gases will be processed through the Overhead Gas System in lieu of the designed vent path to the process vent in order to limit station gaseous releases.

Since this process involves an improvement in the processing of gaseous waste and properly utilizes the existing system for processing the gas, there is no unreviewed safety question.

1-MOP-FC-003
0-OP-FC-003

Maintenance Operating Procedure
Operating Procedure
(Safety Evaluation No. 97-022)

2-17-97

Maintenance Operating Procedure 1-MOP-FC-003, "Removal and Installation of the Swing Check Assembly of 1-BR-506," and Operating Procedure 0-OP-FC-003, "Spent Fuel Pit-Makeup From and Transfer To The Boron Recovery Tanks," will allow fluid transfer between the spent fuel pit and the boron recovery tanks (BRT) to support fuel handling operations. The procedure will allow approximately 15,000 gallons of borated water to be stored in the BRTs during fuel handling operation rather than processing this volume as liquid waste. Prior to returning the borated water to the spent fuel pit, a sample will be drawn to ensure that the borated water has a minimum concentration of 2300 ppm which ensures that the Tech Spec limit, as well as administrative limit for Spent Fuel Pit boron concentration, is not violated.

Based on procedural limitations, the presence of the anti-siphon hole, the expected heat load of the Spent Fuel Pit, and the chemistry control, no unreviewed safety question exists.

2-OPT-SI-022

Operations Periodic Test Procedure
(Safety Evaluation Nos. 96-045, rev. 2)

2-17-97

Operations Periodic Test Procedure, 2-OPT-SI-022, "SI Accumulator Discharge Check Valve Test with Reactor Head Removed," was revised to provide instructions for verifying that the Safety Injection System accumulator discharge check valves are free to open by discharging the accumulators into an open, de-fueled reactor coolant system.

The Unit will be at Refueling Shutdown and defueled. Controls are in place to avoid injection of nitrogen from the accumulators into the RCS. Either containment purge will be in operation or the containment hatch properly secured with at least one personnel hatch closed. To mitigate the potential radiological consequences if nitrogen is injected into the reactor vessel and ultimately into the containment atmosphere. The equipment and systems will be operated within design limits during the performance of this test. Therefore, an unreviewed safety question does not exist.

PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: February, 1997

1/2-OP-RC-003

Operating Procedure
(Safety Evaluation No. 97-024)

2/19/97

Operating Procedures 1/2-OP-RC-003, "Reactor Coolant System Vent," contains a procedurally controlled Temporary Modification to defeat the Reactor Coolant (RC) Low Flow Alarms. This is being performed to eliminate nuisance alarms that would distract the operators. The RC Low Flow Alarms will be restored to fully operable following RC flow transmitter venting.

This Temporary Modification will be removed prior to the start of the first RCP during restart from the Unit 1 Refueling Outage. The safety function these alarms provide is not required below 10% power. Therefore, an unreviewed safety question does not exist.

1/2-E-0
1/2-ES-1.1
1/2-FR-S.1

Emergency Procedures
(Safety Evaluation No. 97-025)

2-20-97

A Deviation Report was submitted documenting a condition where the turbine for the Auxiliary Feedwater Pumps could overspeed if the steam admission valves received an open signal within approximately 15 seconds of a normal shutdown. This condition is due to the governor needing time to allow the oil to drain down to reset the low speed stop. The procedures were revised to increase the point at which the turbine is secured. Additionally the procedures place the bench board switch from AUTO to OPEN when the pumps are required to operate.

The changes to the procedures do not alter the Safety Function of the Auxiliary Feedwater Pump as described in the UFASR, Technical Specifications, or the Design Basis Documents. Additionally, these changes do not alter the results of PSAs for any accidents. Therefore, an unreviewed safety question does not exist.

FP-VPA-FR1

Vendor Procedure
(Safety Evaluation No. 97-028)

2-24-97

This safety evaluation address the portion of Vendor Procedure FP-VPA-FR1, "Fuel Assembly Repair for Surry Unit #1," which specifically calls for the temporary replacement of the original equipment fuel elevator basket with a functionally equivalent Westinghouse basket. The temporary modification will be procedurally controlled in accordance with VPAP-1403, "Temporary Modifications."

The replacement basket is functionally equivalent to original equipment with the exception that the Westinghouse basket accommodates fuel assembly reconstitution with irradiated fuel in the basket. Fuel repair procedures will require the mechanical stop to be installed whenever irradiated fuel is in the fuel elevator basket to prevent the lifting of a fuel assembly to unsafe elevations. Fuel handling will be performed in accordance with existing site procedures. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: February, 1997

1-MOP-EP-304

Maintenance Operating Procedure
(Safety Evaluation No. 97-029)

2-26-97

Maintenance Operating Procedure 1-MOP-EP-304, "RSS Transformer A Outage with Backfeed of Transfer Bus D and Emergency Bus 1J via Alternate AC Diesel Bus OL," was developed to provide instructions for the maintenance of Reserve Service Station Transformer 'A' while Bus 1J remains operable. This allows Bus 1J maintenance to be scheduled separately, providing greater flexibility for outage planning and decreases the unavailability of Emergency Diesel Generator 3.

The proposed change does not alter any electrical power system design or operating limit and satisfies the requirements of GDC-17 of 10CFR50 Appendix A. Existing Technical Specification requirements for electrical power system operability continue to be satisfied. Therefore, an unreviewed safety question does not exist.

1[2]-OPT-SI-021

Operations Periodic Test Procedures
(Safety Evaluation Nos. 95-131 rev. 1)

2-26-97

Operations Periodic Test Procedures, 1[2]-OPT-SI-021, "SI Accumulator Discharge Check Valves Partial Open Test," were developed to provide instructions for implementing a temporary modification (TM) to remove the seal-in function for the Safety Injection (SI) accumulator isolation valves during the performance of the subject test. The TM will allow the valves to be partially opened. The procedure revision also includes instructions for verifying that the Safety Injection System accumulator discharge check valves are free to open and exhibit partial stroke operation during a controlled accumulator dump into the closed Reactor Coolant System (RCS).

The Unit will be at cold shutdown and the equipment and systems will be operated within design limits during the performance of this test. Reactivity addition will be precluded by ensuring the accumulator boron concentration is greater than the RCS minimum refueling shutdown margin concentration and by ensuring that the accumulator temperature is greater than the minimum RCS shutdown margin temperature. Therefore, an unreviewed safety question does not exist.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: February, 1997

None During the Reporting Period

CHEMISTRY REPORT

MONTH/YEAR: February, 1997

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioactivity, $\mu\text{Ci/ml}$	2.19E+0	3.95E-1	9.08E-1	3.92E-1	7.63E-3	2.14E-1
Suspended Solids, ppm	0.050	≤ 0.010	0.030	≤ 0.010	≤ 0.010	≤ 0.010
Gross Tritium, $\mu\text{Ci/ml}$	3.16E-2	2.77E-3	2.15E2	3.93E-1	1.89E-1	3.14E-1
^{131}I , $\mu\text{Ci/ml}$	7.62E-1	6.76E-3	1.60E-1	1.52E-4	1.94E-5	5.54E-5
$^{131}\text{I}/^{133}\text{I}$	0.33	0.18	0.29	0.11	0.05	0.08
Hydrogen, cc/kg	36.0	27.5	31.9	30.5	27.3	29.1
Lithium, ppm	2.15	0.58	1.04	2.34	2.01	2.16
Boron - 10, ppm*	124.5	0.3	40.6	262.8	164.6	194.3
Oxygen, (DO), ppm	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005	≤ 0.005
Chloride, ppm	0.003	≤ 0.001	≤ 0.001	0.006	0.002	0.004
pH at 25 degree Celsius	8.71	6.77	7.46	6.45	6.16	6.34

* Boron - 10 = Total Boron x 0.196

Comments:

None

**FUEL HANDLING
UNITS 1 & 2**

MONTH/YEAR: February, 1997

New Fuel Shipment or Cask No.	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
TN 32-02	2-11-97	32	1R2	LM0C1X	3.5993	N/A
			2E9	LM0DED	3.5986	
			3D8	LM0AN3	3.6081	
			3P0	LM05YP	3.6070	
			3P2	LM05XZ	3.6070	
			4D1	LM0AMS	3.5980	
			4D3	LM0MAT	3.6125	
			4D5	LM0AMA	3.6030	
			4D8	LM0AMU	3.6019	
			5D2	LM0AM7	3.6078	
			5D3	LM0AMZ	3.6067	
			5D5	LM0AN6	3.6062	
			5D7	LM0AMS	3.6082	
			5D9	LM0AN1	3.6062	
			6D0	LM0AN2	3.6073	
			6D2	LM0BDG	3.5847	
			6D4	LM0BDH	3.5857	
			J13	LM0359	2.9020	
			J20	LM0353	2.9020	
			J34	LM035J	2.9020	
			N48	NOANSI	2.5560	
			N49	NOANSI	2.5560	
			N51	NOANSI	2.5560	
			N52	NOANSI	2.5560	
			R05	LM00U4	3.0970	
			R06	LM00U3	3.0970	
			R07	LM00U6	3.0970	

**FUEL HANDLING
UNITS 1 & 2**

MONTH/YEAR: February, 1997

New Fuel Shipment or Cask No.	Date Stored or Received	Number of Assemblies per Shipment	Assembly Number	ANSI Number	Initial Enrichment	New or Spent Fuel Shipping Cask Activity
			R11	LM00U5	3.0970	
TN 32-02	2-11-97	32	R14	LM00TW	3.0970	N/A
			R40	LM00UM	3.0970	
			R43	LM00VJ	3.0970	
			R49	LM00VX	3.0970	

**DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS**

MONTH/YEAR: FEBRUARY, 1997

None During the Reporting Period