

Attachment 2
Proposed Technical Specifications
Surry Power Station

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f. Responsibilities

The SNSOC shall be responsible for:

1. Review of a) all new normal, abnormal, and emergency operating procedures and all new maintenance procedures, b) all procedure changes that require a safety evaluation, and c) any other procedures or changes thereto as determined by the Station Manager which affect nuclear safety.
2. Review of all new test and experiment procedures that affect nuclear safety.
3. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
4. Review of proposed changes to Technical Specifications and shall submit recommended changes to the Station Manager.
5. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and to the Management Safety Review Committee.
6. Review of all Reportable Events and special reports submitted to the NRC.
7. Review of facility operations to detect potential nuclear safety hazards.
8. Performance of special reviews, investigations or analyses and report thereon as requested by the Chairman of the SNSOC or Station Manager.

Amendment Nos.

e. Meeting Frequency

The MSRC shall meet at least once per calendar quarter.

f. Quorum

The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

g. Review

The MSRC shall be responsible for the review of:

1. Safety evaluations as programmatically discussed in the Updated Final Safety Analysis Report for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to assess the effectiveness of the safety evaluation program and to verify that the reviewed actions did not constitute an unreviewed safety question.
2. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
3. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
4. Proposed changes to Technical Specifications or the Operating Licenses.

5. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
6. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
7. Events requiring written notification to the Commission.
8. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
9. A representative sample of reports and meeting minutes of the SNSOC.

h. Audits

Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

1. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
2. The performance, training and qualifications of the entire facility staff at least once per 12 months.
3. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety at least once per 6 months.

Amendment Nos.

2. The requirements of 6.4.B.1 above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Supervisor on duty and/or the senior station individual assigned the responsibility for health physics and radiation protection.
 3. Written procedures shall be established, implemented, and maintained covering the activities referenced below:
 - a. Process Control Program implementation.
 - b. Offsite Dose Calculation Manual implementation.
- C. All procedures described in 6.4.A and 6.4.B shall be reviewed and approved by the Station Nuclear Safety and Operating Committee (SNSOC) prior to implementation. Subsequent procedure changes that require a safety evaluation shall also be reviewed and approved by SNSOC prior to implementation. All other changes shall be independently reviewed and approved as programmatically discussed in the Updated Final Safety Analysis Report.

- D. All procedures described in Specifications 6.4.A and 6.4.B shall be followed.
- E. Deleted |
- F. Deleted |
- G. In cases of emergency, operations personnel shall be authorized to depart from approved procedures where necessary to prevent injury to personnel or damage to the facility. Such changes shall be documented, reviewed and approved by the Station Nuclear Safety and Operating Committee.

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Amendment Nos.

Attachment 3

**Proposed Technical Specifications
North Anna Power Station Unit 1**

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all new procedures required by Specifications 6.8.1 and 6.8.2, 2) all procedure changes that require a safety evaluation, 3) all programs required by Specification 6.8.4 and changes thereto, and 4) any other procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Station Manager.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and the MSRC.
- f. Review of all REPORTABLE EVENTS and Special Reports.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Station Manager.
- i. Deleted.
- j. Deleted.

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.2.2 The MSRC shall be composed of the MSRC Chairman and a minimum of four MSRC members. The Chairman and all members of the MSRC shall have qualifications that meet the requirements of Section 4.7 of ANSI/ANS 3.1-1979 Rev. 1 (Draft).

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the MSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in MSRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants should be utilized as determined by the MSRC Chairman to provide expert advice to the MSRC.

MEETING FREQUENCY

6.5.2.5 The MSRC shall meet at least once per calendar quarter.

QUORUM

6.5.2.6 The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7 The MSRC shall be responsible for the review of:

- a. Safety evaluations as programmatically discussed in the Updated Final Safety Analysis Report for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to assess the effectiveness of the safety evaluation program and to verify that the reviewed actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.

ADMINISTRATIVE CONTROLS

- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. A representative sample of reports and meetings minutes of the SNSOC.

AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50.
- e. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
- f. The Fire Protection Program and implementing procedures.
- g. An independent fire protection and loss prevention inspection and audit shall be performed utilizing an outside qualified fire consultant.
- h. The Radiological Environmental Monitoring Program and the results thereof.

ADMINISTRATIVE CONTROLS

- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each new procedure of 6.8.1 above, except 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved by the SNSOC prior to implementation and reviewed periodically as set forth in administrative procedures. Procedures of 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved as set forth in the facility's Security Plan, Emergency Plan, and section 6.5.1.6.m of the Technical Specifications, respectively.

6.8.3 Procedure changes that require a safety evaluation shall also be reviewed and approved by SNSOC. All other changes shall be independently reviewed and approved as programmatically discussed in the Updated Final Safety Analysis Report.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the recirculation spray, safety injection, chemical and volume control, gas stripper, and hydrogen recombiners. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

Attachment 4

**Proposed Technical Specifications
North Anna Power Station Unit 2**

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all new procedures required by Specifications 6.8.1 and 6.8.2, 2) all procedure changes that require a safety evaluation, 3) all programs required by Specification 6.8.4 and changes thereto, and 4) any other procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Station Manager.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and the MSRC.
- f. Review of all REPORTABLE EVENTS and Special Reports.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Station Manager.
- i. Deleted.
- j. Deleted.

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.2.2 The MSRC shall be composed of the MSRC Chairman and a minimum of four MSRC members. The Chairman and all members of the MSRC shall have qualifications that meet the requirements of Section 4.7 of ANSI/ANS 3.1-1979 Rev. 1 (Draft).

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the MSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in MSRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants should be utilized as determined by the MSRC Chairman to provide expert advice to the MSRC.

MEETING FREQUENCY

6.5.2.5 The MSRC shall meet at least once per calendar quarter.

QUORUM

6.5.2.6 The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7 The MSRC shall be responsible for the review of:

- a. Safety evaluations as programmatically discussed in the Updated Final Safety Analysis Report for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to assess the effectiveness of the safety evaluation program and to verify that the reviewed actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.

ADMINISTRATIVE CONTROLS

- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. A representative sample of reports and meetings minutes of the SNSOC.

AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50.
- e. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
- f. The Fire Protection Program and implementing procedures.
- g. An independent fire protection and loss prevention inspection and audit shall be performed utilizing an outside qualified fire consultant.
- h. The Radiological Environmental Monitoring Program and the results thereof.

ADMINISTRATIVE CONTROLS

- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each new procedure of 6.8.1 above, except 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved by the SNSOC prior to implementation and reviewed periodically as set forth in administrative procedures. Procedures of 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved as set forth in the facility's Security Plan, Emergency Plan, and section 6.5.1.6.m of the Technical Specifications, respectively.

6.8.3 Procedure changes that require a safety evaluation shall also be reviewed and approved by SNSOC. All other changes shall be independently reviewed and approved as programmatically discussed in the Updated Final Safety Analysis Report.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the recirculation spray, safety injection, chemical and volume control, gas stripper, and hydrogen recombiners. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

Attachment 5
Significant Hazards Consideration

Significant Hazards Consideration

Virginia Electric and Power Company has reviewed the proposed Technical Specification changes against the criteria of 10 CFR 50.92 and has concluded that the changes as proposed do not pose a significant hazards consideration. Consequently, the elimination of the SNSOC review of procedure changes that do not require a safety evaluation, revising the wording for approval of procedure changes, and the modification of the MSRC's duties regarding their review of safety evaluations and SNSOC meeting minutes and reports will not:

1) Involve a significant increase in the probability or consequences of an accident previously evaluated. As administrative changes, the proposed Technical Specifications changes have no direct or indirect effect on accident precursors. No plant modifications are being implemented and operation of the plant is unchanged. SNSOC review of new procedures and procedure changes that require a safety evaluation ensures that activities that could affect nuclear safety are being properly reviewed. The MSRC's overview of representative samples of safety evaluations and SNSOC meeting minutes and reports based on performance ensures these programs are being properly implemented and nuclear safety is not being compromised; or

2) Create the possibility of a new or different kind of accident from any accident previously evaluated since physical modifications are not involved and systems and components will be operated as before the change. The proposed changes are wholly administrative in nature and have no impact on plant operations or accident considerations. These changes modify the scope of SNSOC review of procedure changes and MSRC's review functions concerning safety evaluations and SNSOC meeting minutes and reports. Procedure changes will continue to receive management review in accordance with administrative procedures, however, only changes that require a safety evaluation will require SNSOC approval. MSRC review of representative samples of safety evaluations and SNSOC meeting minutes and reports based on performance will continue to provide adequate assurance that nuclear safety is being properly considered; or

3) Involve a significant reduction in a margin of safety as defined in the basis of any Technical Specification since the responsibilities of the SNSOC and MSRC are not addressed by the existing Technical Specification Bases, nor are review requirements

for procedures. The proposed changes are administrative in nature and have no impact on, nor were they considered in, existing UFSAR accident analyses. Safety significant procedure changes, i.e., changes that require a safety evaluation to be prepared, will continue to be reviewed by SNSOC, as will new procedures. Procedure changes still require cognizant management approval and preparation of an activity screening to determine whether or not the change impacts nuclear safety. This ensures activities important to nuclear safety are being appropriately reviewed. The effectiveness of the safety evaluation program, and the thoroughness of SNSOC meetings and reports will be assured through the MSRC's plant overview function which is based on observed performance.