

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

June 13, 1996

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

Serial No. 96-301
NO/RPC:vlh
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of May 1996.

Very truly yours,



M. L. Bowling, Manager
Nuclear Licensing & Operations Support

Enclosure

cc: U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, N. W.
Suite 2900
Atlanta, Georgia 30323

Mr. M. W. Branch
NRC Senior Resident Inspector
Surry Power Station

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**VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION
MONTHLY OPERATING REPORT
REPORT No. 96-05**

Approved:



Station Manager

6-12-96

Date

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OPERATING DATA REPORT

Docket No.: 50-280
 Date: 5-03-96
 Completed By: D. Mason
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 1
- 2. Reporting Period:..... May, 1996
- 3. Licensed Thermal Power (MWt):..... 2546
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe):..... 840
- 7. Maximum Dependable Capacity (Net MWe):..... 801

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting Period	744.0	3647.0	205487.0
12. Number of Hours Reactor Was Critical	744.0	3647.0	141697.7
13. Reactor Reserve Shutdown Hours	0.0	0.0	3774.5
14. Hours Generator On-Line.....	744.0	3647.0	139394.0
15. Unit Reserve Shutdown Hours.....	0.0	0.0	3736.2
16. Gross Thermal Energy Generated (MWH).....	1893857.4	9270108.3	325667900.8
17. Gross Electrical Energy Generated (MWH)....	631435.0	3098325.0	106675508.0
18. Net Electrical Energy Generated (MWH).....	610117.0	2998168.0	101454141.0
19. Unit Service Factor.....	100.0%	100.0%	67.8%
20. Unit Availability Factor.....	100.0%	100.0%	69.7%
21. Unit Capacity Factor (Using MDC Net).....	102.4%	102.6%	63.6%
22. Unit Capacity Factor (Using DER Net).....	104.1%	104.3%	62.7%
23. Unit Forced Outage Rate.....	0.0%	0.0%	15.7%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down at End of Report Period, Estimated Date of Start-up: _____

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATING DATA REPORT

Docket No.: 50-281
 Date: 5-03-96
 Completed By: D. Mason
 Telephone: (804) 365-2459

- 1. Unit Name:..... Surry Unit 2
- 2. Reporting Period: May, 1996
- 3. Licensed Thermal Power (MWt): 2546
- 4. Nameplate Rating (Gross MWe):..... 847.5
- 5. Design Electrical Rating (Net MWe):..... 788
- 6. Maximum Dependable Capacity (Gross MWe): 840
- 7. Maximum Dependable Capacity (Net MWe):..... 801

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	YTD	Cumulative
11. Hours In Reporting Period	744.0	3647.0	202367.0
12. Number of Hours Reactor Was Critical	49.7	2866.5	138369.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	328.1
14. Hours Generator On-Line.....	48.9	2859.0	136415.1
15. Unit Reserve Shutdown Hours.....	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH).....	115277.8	7203667.8	319738721.8
17. Gross Electrical Energy Generated (MWH)....	37600.0	2399815.0	104555459.0
18. Net Electrical Energy Generated (MWH).....	36260.0	2323314.0	99433729.0
19. Unit Service Factor.....	6.6%	78.4%	67.4%
20. Unit Availability Factor.....	6.6%	78.4%	67.4%
21. Unit Capacity Factor (Using MDC Net).....	6.1%	79.5%	63.0%
22. Unit Capacity Factor (Using DER Net).....	6.2%	80.8%	62.4%
23. Unit Forced Outage Rate.....	0.0%	3.1%	12.8%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down at End of Report Period, Estimated Date of Start-up: June 9, 1996

26. Unit In Test Status (Prior to Commercial Operation):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: May, 1996

Docket No.: 50-280
 Unit Name: Surry Unit 1
 Date: 5-01-96
 Completed by: Craig Olsen
 Telephone: (804) 365-2155

(1)	(2)	(3)	(4)	(5)
Date	Type	Duration Hours	Reason	Method of Shutting Down Rx
			LER No.	System Code
			Component Code	Cause & Corrective Action to Prevent Recurrence

None During the Reporting Period

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets
 for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source.

UNIT SHUTDOWN AND POWER REDUCTION
 (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: May, 1996

Docket No.: 50-281
 Unit Name: Surry Unit 2
 Date: 5-01-96
 Completed by: Craig Olsen
 Telephone: (804) 365-2155

(1) Date	(1) Type	(2) Duration Hours	(2) Reason	(3) Method of Shutting Down Rx	(4) LER No.	(4) System Code	(5) Component Code	(5) Cause & Corrective Action to Prevent Recurrence
960503	S	N/A	C	1	N/A	N/A	N/A	Scheduled Refueling Outage

(1)
 F: Forced
 S: Scheduled

(2)
 REASON:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Examination
 F - Administrative
 G - Operational Error (Explain)

(3)
 METHOD:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

(4)
 Exhibit G - Instructions for Preparation of Data Entry Sheets
 for Licensee Event Report (LER) File (NUREG 0161)

(5)
 Exhibit 1 - Same Source.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-280
Unit Name: Surry Unit 1
Date: 5-01-96
Completed by: Barry C. Bryant
Telephone: (804) 365-2786

MONTH: May, 1996

<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>	<u>Day</u>	<u>Average Daily Power Level (MWe - Net)</u>
1	819	17	822
2	818	18	820
3	817	19	820
4	824	20	819
5	825	21	818
6	823	22	818
7	823	23	818
8	823	24	816
9	823	25	817
10	822	26	819
11	822	27	820
12	823	28	819
13	823	29	817
14	824	30	811
15	825	31	808
16	823		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-281
 Unit Name: Surry Unit 2
 Date: 5-01-96
 Completed by: Barry C. Bryant
 Telephone: (804) 365-2786

MONTH: May, 1996

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	812	17	0
2	697	18	0
3	2	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: May, 1996

The following chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE:

5/01/96	0000	The reporting period began with the unit operating at 100% power, 850 MWe.
5/31/96	2400	The reporting period ended with the unit operating at 100% power, 850 MWe.

UNIT TWO:

5/01/96	0000	The reporting period began with the unit operating at 100% power, 835 MWe.
5/02/96	1700	Began power reduction from 100%, 845 MWe, to prepare for start of refueling outage.
5/03/96	0052	Generator off-line.
	0142	Reactor manually tripped.
5/31/96	2400	The reporting period ended with the unit at refueling shutdown.

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: May, 1996

- | | | |
|---------------------------|---|---------|
| TM S1-96-07 | Temporary Modification
(Safety Evaluation No. 96-055) | 5-01-96 |
| | <p>Temporary Modification S1-96-07 installed temporary tubing to route the discharge of the Unit 1 recirculating spray heat exchanger (RSHX) service water (SW) radiation monitoring pumps 1-SW-P-5A, 5B, and 5D to a spare penetration on the Unit 2 SW 24 inch pipe in the Unit 2 Safeguards building. This modification supports the implementation of SW system modifications and maintenance activities.</p> <p>The temporary tubing was pressure rated to withstand the RSHX SW radiation monitoring system pump discharge. The RSHX SW radiation monitoring system was tested following installation and removal of the modification and remained operable. Therefore, an unreviewed safety question does not exist.</p> | |
| FS 96-22 | Updated Final Safety Analysis Report Change
(Safety Evaluation 96-059) | 5-02-96 |
| | <p>Updated Final Safety Analysis Report Change 96-22 revised various sections that address fuel handling to accurately reflect current station practices.</p> <p>None of the changes represented significant deviations from the design basis of the fuel handling system and did not reduce the Technical Specifications margin of safety. The changes also did not affect any of the assumptions made in any fuel handling or other design basis accident analyses. Therefore, an unreviewed safety question does not exist.</p> | |
| SE 96-056 | Safety Evaluation | 5-02-96 |
| | <p>Safety Evaluation 96-056 was performed to evaluate the 1996 Unit 2 refueling outage schedule.</p> <p>The evaluation concluded that the refueling outage schedule is acceptable based on a review of (a) the capability to satisfy Cold Shutdown (CSD) and Refueling Shutdown (RSD) critical safety functions for Unit 2, and (b) the effects of Unit 2 outage activities on critical safety functions for Unit 1. Therefore, an unreviewed safety question does not exist.</p> | |
| TM S1-96-006
1-CAL-112 | Temporary Modification
Calibration Procedure
(Safety Evaluation No. 96-038, Revision 1) | 5-02-96 |
| | <p>Temporary Modification (TM) S1-96-006 installed a calibrated resistor in the boron recovery tank (BRT) level instrumentation loop and a chart recorder to allow the BRT level to be more precisely monitored during reactor coolant system draining evolutions. In addition, Calibration Procedure 1-CAL-112, "L-BR103B Boron Recovery Tank B," was temporarily changed to provide instructions for verifying the operability and accuracy of the BRT high level alarm following the installation of the TM.</p> <p>The TM was in place while Unit 2 was shut down and did not affect the boron recovery system or any automatic protection or control functions. Therefore, an unreviewed safety question does not exist.</p> | |

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: May, 1996

TM S1-96-008

Temporary Modification
(Safety Evaluation No. 96-058)

5-02-96

Temporary Modification (TM) S1-96-008 installed an electrical jumper to allow main control room radiation monitor cabinet 1-1 to continue to operate while a lug was replaced on a wire from the load side of the cabinet power fuse.

The TM maintained the radiation monitor system in a fully operable condition. Double verification of jumper installation/removal and post-maintenance testing were performed. Therefore, an unreviewed safety question does not exist.

TM S2-96-005

Temporary Modification
(Safety Evaluation No. 96-060)

5-03-96

Temporary Modification (TM) S2-96-005 installed a mechanical jumper from the end of the Surry Radwaste Facility (SRF) effluent discharge line to a point downstream of the discharge tunnel weir. The TM was installed to preclude the potential for SRF effluent to flow back into the Unit 2 Turbine building while the discharge tunnel was partially drained to facilitate maintenance activities.

The mechanical jumper was inspected for leakage on a daily basis and the length was limited to ensure the discharge tunnel radiation monitors were not bypassed. Therefore, an unreviewed safety question does not exist.

TM S2-96-007
2K-C8
2-OP-CD-001

Temporary Modification
Annunciator Response Procedure
Operating Procedure
(Safety Evaluation No. 96-063)

5-06-96

Temporary Modification (TM) S2-96-007 removed the defective bearing oil temperature sensor from the trip circuit of containment cooling chiller, 2-CD-REF-1, to prevent the occurrence of spurious chiller trips. Annunciator Response Procedure 2K-C8, "2-CD-REF-1 TRBL," and Operating Procedure 2-OP-CD-001, "Chilled Water System (CD)," were changed to facilitate the implementation of the TM.

The TM did not alter any safety-related components and did not affect the operation of the other safety trips associated with 2-CD-REF-1 (e.g., oil differential pressure, refrigerant low pressure, low oil pressure). Bearing oil temperature, as well as other operating parameters, were monitored in accordance with normal operator log requirements. Therefore, an unreviewed safety question does not exist.

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: May, 1996

FS 95-18 **Updated Final Safety Analysis Report Change** 5-09-96
VPAP-0809 **Administrative Procedure**
(Safety Evaluation 96-066)

Updated Final Safety Analysis Report Change 95-18 revised Section 9B.3, "Control of Heavy Loads," to summarize Virginia Power's commitments related to NUREG-0612. Administrative procedure VPAP-0809, "NUREG-0612 Heavy Load Program," was also revised to be consistent with the UFSAR.

The subject changes reflect Virginia Power's commitment to comply with the guidelines of NUREG-0612 which provides overall defense-in-depth at preventing load handling accidents. Therefore, an unreviewed safety question does not exist.

TM S2-96-008 **Temporary Modification** 5-09-96
(Safety Evaluation No. 96-067)

Temporary Modification (TM) S2-96-008 installed temporary strip chart recorders in the instrument circuit of the Unit 2 reactor coolant system (RCS) standpipe to allow monitoring of selected loop parameters.

A local RCS standpipe watch was maintained during the installation and removal of the TM. The TM did not affect the operation or operability of the instrument loop or any associated components. Therefore, an unreviewed safety question does not exist.

TM S2-96-006 **Temporary Modification** 5-13-96
(Safety Evaluation No. 96-064, Revision 1)

Temporary Modification (TM) S2-96-006 installed temporary drinking fountains and associated water lines in the Unit 2 containment for use during the 1996 refueling outage.

The water lines were rated for the pressure and temperature of the domestic water system and were only used while the plant was at Cold Shutdown. The TM did not affect any safety-related system or component. Therefore, an unreviewed safety question does not exist.

TM S2-96-009 **Temporary Modification** 5-13-96
(Safety Evaluation No. 96-068)

Temporary Modification (TM) S2-96-009 installed electrical jumpers to provide control room operators with manual control of the Unit 2 containment sump pumps while modifications to main board rack No. 6 were being implemented. A television camera was also installed at the containment sump to allow the water level to be visually monitored from the control room.

The unit was at cold shutdown and de-fueled while the TM was in place. Double verification of jumper installation/removal and post-maintenance testing were performed. Therefore, an unreviewed safety question does not exist.

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: May, 1996

- TM S2-96-010 **Temporary Modification** 5-13-96
(Safety Evaluation No. 96-069)
- Temporary Modification (TM) S2-96-010 installed electrical jumpers to provide control room operators with manual control of the Unit 2 primary drains transfer pumps while modifications to main board rack No. 6 were being implemented.
- The unit was at Cold Shutdown and de-fueled while the TM was in place. Double verification of jumper installation/removal and post-maintenance testing were performed. Therefore, an unreviewed safety question does not exist.
- TM S2-96-011 **Temporary Modification** 5-14-96
(Safety Evaluation No. 96-070)
- Temporary Modification (TM) S2-96-011 installed electrical jumpers to maintain Unit 2 primary drains transfer tank level indication while modifications to main board rack No. 6 were being implemented.
- The unit was at Cold Shutdown and de-fueled while the TM was in place. Double verification of jumper installation/removal and post-maintenance testing were performed. Therefore, an unreviewed safety question does not exist.
- TM S1-96-011 **Temporary Modification** 5-16-96
(Safety Evaluation No. 96-073)
- Temporary Modification (TM) S1-96-011 installed mechanical jumpers to provide an alternate source of compressed air to the Unit 1 circulating water system vacuum priming and vacuum breaker valves while instrument air system valve 1-IA-1215 was being repaired.
- Testing was performed following the installation and removal of the TM to ensure proper operation of the subject valves. The TM did not affect the three diesel engine-driven emergency service water pumps. Therefore, an unreviewed safety question does not exist.
- TM S2-96-013 **Temporary Modification** 5-19-96
(Safety Evaluation No. 96-077)
- Temporary Modification (TM) S2-96-013 installed temporary drinking fountains and associated water lines in the Unit 2 containment for use during the 1996 refueling outage.
- The water lines were rated for the pressure and temperature of the domestic water system and were only used while the plant was at Cold Shutdown. The TM did not affect any safety-related system or component. Therefore, an unreviewed safety question does not exist.

**FACILITY CHANGES THAT DID NOT
REQUIRE NRC APPROVAL**

MONTH/YEAR: May, 1996

TM S2-96-12 **Temporary Modification** 5-22-96
(Safety Evaluation No. 96-078, Revision 1)

Temporary Modification (TM) S2-96-12 removed inside trip valves 2-DA-TV-200A and 2-DA-TV-200B for maintenance, installed a spool piece to maintain an operable flowpath from the Unit 2 containment sump pumps 2-DA-P-4A/B, and maintained an operable flowpath for the containment sump sample pump. Installation of electrical jumpers allowed 2-DA-P-4A/B to operate with the limit switches for the two trip valves removed.

This activity did not affect the operability of the containment sump pumps, nor the operability of the containment sump sample pump. The unit was at Cold Shutdown, at which time containment integrity is not required by Technical Specifications. The TM was removed prior to exceeding 200°F and the valves were tested following maintenance to verify operability. Therefore, an unreviewed safety question does not exist.

TM S2-96-14 **Temporary Modification** 5-22-96
(Safety Evaluation No. 96-079)

Temporary Modification (TM) S2-96-14 installed electrical jumpers to maintain power to Hi-Consequence Limiting Safeguards (Hi-CLS) system output relays during the replacement of Unit 2 relays 3/4-CLS-1B, 3-CLS-1BM, and CR-CLS-1B1. The TM also disconnected an electrical lead in the train "B" safety injection (SI) rack to prevent an SI initiation from a Hi-CLS.

The activity was performed while Unit 2 was at cold shutdown, or below, when the CLS system was not required. Double verification of the installation/removal of the TMs was performed. Therefore, an unreviewed safety question does not exist.

TM S2-96-15 **Temporary Modification** 5-22-96
(Safety Evaluation No. 96-080)

Temporary Modification (TM) S2-96-15 installed electrical jumpers to maintain power to Hi-Consequence Limiting Safeguards (Hi-CLS) system output relays during the replacement of Unit 2 relays 3/4-CLS-1A, 3-CLS-1AM, and CR-CLS-1A1. The TM also disconnected an electrical lead in the train "A" safety injection (SI) rack to prevent an SI initiation from a Hi-CLS.

The activity was performed while Unit 2 was at cold shutdown, or below, when the CLS system was not required. Double verification of the installation/removal of the TMs was performed. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: May, 1996

- 1-E-0
1-ES-1.1
TM S1-96-09
- Emergency Procedures** 5-03-96
- Temporary Modification**
(Safety Evaluation No. 96-061)
- Temporary Modification (TM) S1-96-09 installed electrical jumpers and switches to provide operators with the ability to reposition motor-operated dampers 1-VS-MOD-100A[B] and 2-VS-MOD-200A[B] in the event of a reduction of air pressure below the 65 psig lock-out point. Emergency Operating Procedures 1-E-0, "Reactor Trip or Safety Injection," and 1-ES-1.1, "SI Termination," were revised to identify the subject switches and their appropriate positions.
- In the unlikely event of a loss of air pressure prior to a safety injection, the subject TMs allow the dampers to be correctly positioned. Therefore, an unreviewed safety question does not exist.
- 2-OP-RT-003
- Operating Procedure** 5-04-96
(Safety Evaluation No. 94-113, Revision 1)
- Operating Procedure 2-OP-RT-003, "Pumping Steam Generators to WNS Using the Blowdown Lines," was temporarily changed to provide instructions for the installation of a temporary drain hose to permit pumping the contents of any one of the steam generators to the waste neutralization sumps.
- This procedure is performed when the unit is at Cold Shutdown and SG water conditions are not acceptable for release to the James River. Only one SG is drained at a time and the draining will be monitored to ensure that any SG tube leakage is identified so that the evolution can be terminated. Therefore, an unreviewed safety question does not exist.
- 0-STD-FP-1996-7751
- Engineering Standard** 5-06-96
(Safety Evaluation Nos. 96-062)
- Engineering Standard, 0-STD-FP-1996-7751, "Surry RCCA and Fuel Assembly Examination Field Procedure," was developed to provide instructions for performing fuel assembly testing in the spent fuel pool. The testing satisfies the requirements of NRC Bulletin 96-01, "Incomplete Control Rod Insertion."
- The testing is performed on one fuel assembly at a time and fuel handling is performed using existing fuel handling equipment and approved procedures. The testing does not adversely affect any safety-related systems or equipment. Therefore, an unreviewed safety question does not exist.
- 1[2]-MOP-36.5
- Maintenance Operating Procedures** 5-09-96
(Safety Evaluation No. 96-065)
- Maintenance Operating Procedures 1[2]-MOP-36.5, "Secondary Plant Air Inleakage Inspection," were revised to provide instructions for the installation and removal of an Orbisphere oxygen monitor and a temporary sample header at the condensate pump discharge drain lines. The data obtained will be used to verify that the existing on-line oxygen monitor is operating properly.
- The procedurally controlled temporary modifications did not adversely affect the operation of any component or system. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: May, 1996

2-OPT-SI-022

Operations Periodic Test Procedure
(Safety Evaluation Nos. 96-045, Revision 1)

5-14-96

Operations Periodic Test Procedure, 2-OPT-SI-022, "SI Accumulator Discharge Check Valve Test with Reactor Head Removed" was revised to provide instructions for verifying that the Safety Injection system accumulator discharge check valves are free to open by discharging the accumulators into an open, de-fueled reactor coolant system.

The Unit will be at Refueling Shutdown and the equipment and systems will be operated within design limits during the performance of this test. To mitigate the potential radiological consequences of nitrogen being injected into the reactor vessel and ultimately into the containment atmosphere, the containment purge system will be in service. Therefore, an unreviewed safety question does not exist.

CAL-817
PT-26.6V
IMP-C-RM-36
0-IPM-RM-G-001
0-IPM-RM-G-002
0-IPM-RM-G-003

Calibration Procedure
Instrumentation Periodic Test Procedure
Instrument Maintenance Procedure
Instrument Preventive Maintenance Procedures
(Safety Evaluation No. 96-075)

5-16-96

Calibration Procedure CAL-817, "Model 942 Log Ratemeter Scintillation Detector Source Calibration," Instrumentation Periodic Test Procedure PT-26.6V, "Victoreen Radiation Monitoring Equipment Background, Heat Trace, Flow Fault Checks and Filter Replacement," Instrument Maintenance Procedure IMP-C-RM-36, "Checking, Repairing or Replacing a Component in the Radiation Monitoring System, and Instrument Preventive Maintenance Procedures 0-IPM-RM-G-001, "Digital Ratemeter Model 942B Process Monitor Calibration," 0-IPM-RM-G-002, "945B Series Area Monitor Ratemeter and Detector Calibration," and 0-IPM-RM-G-003, "Model 943-5 GM Tube Detector Calibration," were revised/developed to provide instructions for the implementation of temporary modifications in the main control room radiation monitor cabinets to prevent various control functions from occurring while the digital ratemeters are removed from service for testing, repair, or calibration.

These activities will not alter the performance characteristics of any safety-related systems or components and only one ratemeter, that involves a control function, will be removed from service at a time. Therefore, an unreviewed safety question does not exist.

FDTP-92-052-3-9

Final Design Test Procedure
(Safety Evaluation No. 96-071)

5-16-96

Final Design Test Procedure FDTP-92-052-3-9 was developed to provide instructions for functionally testing the Station Blackout (SBO) Diesel Generator to verify that the requirements of 10 CFR 50.63 and Regulatory Guide 1.155 are met.

The testing will be conducted with Unit 2 at Refueling Shutdown. The No. 1 and No. 3 Emergency Diesel Generators will be fully operable. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: May, 1996

OPAP-0010

Operations Administrative Procedure
(Safety Evaluation No. 96-072)

5-16-96

Operations Administrative Procedure OPAP-0010, "Tag-Outs," was changed to allow single isolation during the replacement of the rubber expansion joints for service water system motor-operated valves 2-SW-MOV-201A[B]. Although initiated for this specific activity, the procedural change is applicable to other work activities where double isolation is impractical.

The procedure change requires: (a) the single isolation device to be tagged closed and independently verified, (b) the installation of a device to prevent valve stem movement (e.g., locking pin), (c) that single isolation maintenance activities be performed continuously to the extent possible, and (d) that turbine building sump pump maintenance not be scheduled coincident with single isolation circulating/service water work. This change does not affect the Individual Plant Evaluation for Severe Accident Vulnerability since single isolation was assumed. Therefore, an unreviewed safety question does not exist.

1[2]-OPT-FW-008
1[2]-AP-21.01

**Operations Periodic Test Procedures
Abnormal Procedures**
(Safety Evaluation No. 96-074)

5-16-96

Operations Periodic Test Procedures 1[2]-OPT-FW-008, "AFW Check Valve Operability Test," were revised to provide instructions for performing back-leakage testing of the check valves at each individual auxiliary feedwater (AFW) pump discharge and flow verification of unit cross-connect flowpaths. Flow testing of the check valves will be performed in conjunction with the full flow testing to each steam generator (SG) for the motor-driven AFW pumps. Flow testing of the unit cross-connect check valves will require the AFW throttle valves on both units to be closed concurrently to ensure there will be no flow to the operating unit's SGs. Administrative control will be established to re-align the unit cross-connect to provide AFW flow from the non-operating unit to the operating unit within one minute, if necessary. Abnormal Procedures 1[2]-AP-21.01, "Response to AFW Check Valve Backleakage," were also revised to specify the valve alignment options for isolating the AFW motor-operated valves to prevent backleakage through the AFW check valves.

The automatic open signal for the operating unit will remain available and the administrative controls will ensure that AFW is available to the operating unit to perform the decay heat removal safety function. Therefore, an unreviewed safety question does not exist.

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR: May, 1996

2-OPT-SI-002 **Operations Periodic Test Procedure** 5-24-96
(Safety Evaluation No. 96-081)

Operations Periodic Test Procedure 2-OPT-SI-002, "Refueling Test of the Low Head Safety Injection [LHSI] Check Valves to the Cold Legs," was temporarily revised to permit the use of a temporary pressure gauge (for information) in the Unit 2 "B" low head safety injection pump seal cooling system during the performance of the subject procedure.

The temporary pressure gauge was in place while the unit was at Cold Shutdown, or below, when the LHSI system was not required by Technical Specifications. Therefore, an unreviewed safety question does not exist.

2-EPT-0903-01 **Electrical Periodic Test** 5-28-96
(Safety Evaluation No. 95-030, Revision 2)

Electrical Periodic Test 2-EPT-0903-01, "RCP Heat Detector Testing," was revised to provide instructions for implementing temporary modifications to facilitate testing of the continuous fire detectors (CFD) in each reactor coolant pump cubicle. The fire detector element leads and alarm output leads will be lifted and test equipment installed to test the circuit. The fire detector control module is tested by simulating the detector element and verifying the actuation of the alarm to the control box for annunciation in the control room. The procedure also includes instructions for testing the trouble annunciation associated with a loss of power to the CFD control boxes.

System operation is verified upon removal of the test equipment by pushing the local test button on the control panel to ensure circuit continuity, and by monitoring the alarm response in the control room. To ensure that a fire in the reactor coolant pump (RCP) cubicles would be detected during the test, a Technical Specification limiting condition for operation will be entered that requires monitoring of the RCP pump bearing and motor temperature or containment RTDs on an hourly basis. As an alternative, containment walkdowns may be performed once every eight hours or containment temperature may be monitored through the P250 computer every 15 minutes. These measures will ensure that a fire in the RCP cubicles will not go undetected. Therefore, an unreviewed safety question does not exist.

SE 96-083 **Safety Evaluation** 5-31-96

Safety Evaluation 96-083 was performed to evaluate a change in the operating speed of the "B" emergency service water pump (ESWP) from 900 rpm to 853 rpm to reduce pump driver vibrations.

The evaluation concluded that the "B" ESWP is fully capable of providing design basis flow at minimum water level when operating at 853 rpm. Therefore, an unreviewed safety question does not exist.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: May, 1996

None During the Reporting Period

CHEMISTRY REPORT

MONTH/YEAR: May, 1996

Primary Coolant Analysis	Unit No. 1			Unit No. 2		
	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioactivity, $\mu\text{Ci/ml}$	8.14E-1	5.74E-2	6.43E-1	1.34E-1	4.19E-4	2.50E-2
Suspended Solids, ppm	≤ 0.01	≤ 0.01	≤ 0.01	0.250	≤ 0.01	0.085
Gross Tritium, $\mu\text{Ci/ml}$	8.31E-1	7.41E-1	7.79E-1	N/A	N/A	N/A
^{131}I , $\mu\text{Ci/ml}$	4.49E-3	2.65E-3	3.42E-3	1.22E-4	7.78E-5	1.04E-4
$^{131}\text{I}/^{133}\text{I}$	0.53	0.34	0.42	0.07	0.07	0.07
Hydrogen, cc/kg	39.2	34.3	35.9	27.8	2.4	14.2
Lithium, ppm	2.34	2.07	2.21	1.71	0.20	0.61
Boron - 10, ppm*	161.5	149.2	155.3	450.4	4.9	99.1
Oxygen, (DO), ppm	≤ 0.005	≤ 0.005	≤ 0.005	9.0	≤ 0.005	5.1
Chloride, ppm	0.006	0.002	0.004	0.015	≤ 0.001	0.003
pH at 25 degree Celsius	6.71	6.39	6.55	6.80	4.46	5.12

* Boron - 10 = Total Boron x 0.196

Comments:

None

**FUEL HANDLING
UNITS 1 & 2**

MONTH/YEAR: May, 1996

<u>New or Spent Fuel Shipment Number</u>	<u>Date Stored or Received</u>	<u>Number of Assemblies per Shipment</u>	<u>Assembly Number</u>	<u>ANSI Number</u>	<u>Initial Enrichment</u>	<u>New or Spent Fuel Shipping Cask Activity</u>
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No Fuel Received or Stored During the Reporting Period

**DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS**

MONTH/YEAR: May, 1996

None During the Reporting Period