UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 27, 2018

Ms. Cheryl A. Gayheart
Regulatory Affairs Director
Southern Nuclear Operating Company, Inc.
P.O. Box 1295, Bin 038
Birmingham, AL 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 – ISSUANCE OF AMENDMENTS RE: TECHNICAL SPECIFICATION TASK FORCE TRAVELER 412, REVISION 3 (EPID L-2017-LLA-0429)

Dear Ms. Gayheart:

The U.S. Nuclear Regulatory Commission (NRC, the Commission) has issued the enclosed Amendment No. 219 to Renewed Facility Operating License No. NPF-2 and Amendment No. 216 to Renewed Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Units 1 and 2, respectively. The amendments are in response to your application dated December 21, 2017.

Specifically, the amendments modify Technical Specification (TS) 3.7.5, "Auxiliary Feedwater (AFW) System," to establish a new Completion Time for the Condition where one steam supply to the turbine driven AFW pump is inoperable concurrent with an inoperable motor driven AFW train. In addition, the amendments specify Conditions and Action requirements: (1) for when two motor driven AFW trains are inoperable at the same time and (2) for when the turbine driven AFW train is inoperable either (a) due solely to one inoperable steam supply, or (b) due to reasons other than one inoperable steam supply.

The changes to the TS are consistent with NRC-approved Technical Specification Task Force (TSTF) Traveler, TSTF-412, Revision 3, "Provide Actions for One Steam Supply to Turbine Driven AFW/EFW [Emergency Feedwater] Pump Inoperable" dated January 10, 2007. NRC approval of TSTF-412, Revision 3, was announced in the *Federal Register* on July 17, 2007, (72 FR 39089).

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Shawn Williams

Shawn A. Williams, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosures:
1. Amendment No. 219 to NPF-2
2. Amendment No. 216 to NPF-8
3. Safety Evaluation

cc w/enclosures: Listserv

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 219
Renewed License No. NPF-2

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Joseph M. Farley Nuclear Plant, Unit 1, (the facility), Renewed Facility Operating License No. NPF-2, filed by Southern Nuclear Operating Company, Inc. (the licensee), dated December 21, 2017, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-2 is hereby amended to read as follows:

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 219, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and
Technical Specifications

Date of Issuance: June 27, 2018

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 216
Renewed License No. NPF-8

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Joseph M. Farley Nuclear Plant, Unit 2, (the facility), Renewed Facility Operating License No. NPF-8, filed by Southern Nuclear Operating Company, Inc. (the licensee), dated December 21, 2017, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

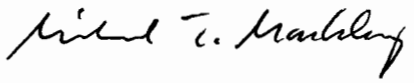
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment, and paragraphs 2.C.(2) of Renewed Facility Operating License No. NPF-8 is hereby amended to read as follows:

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 216, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License
and Technical Specifications

Date of Issuance: June 27, 2018

ATTACHMENT TO JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

LICENSE AMENDMENT NO. 219

TO RENEWED FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

AND LICENSE AMENDMENT NO. 216

TO RENEWED FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following pages of the Renewed Facility Operating Licenses and Appendix "A" Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
License	License
NPF-2, page 4	NPF-2, page 4
NPF-8, page 3	NPF-8, page 3
TSs	TSs
3.7.5-1	3.7.5-1
3.7.5-2	3.7.5-2
3.7.5-3	3.7.5-3
3.7.5-4	3.7.5-4

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 219, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the issuance of the renewed license or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission.

a. Southern Nuclear shall not operate the reactor in Operational Modes 1 and 2 with less than three reactor coolant pumps in operation.

b. Deleted per Amendment 13

c. Deleted per Amendment 2

d. Deleted per Amendment 2

e. Deleted per Amendment 152

Deleted per Amendment 2

f. Deleted per Amendment 158

g. Southern Nuclear shall maintain a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

1) Identification of a sampling schedule for the critical parameters and control points for these parameters;

2) Identification of the procedures used to quantify parameters that are critical to control points;

3) Identification of process sampling points;

4) A procedure for the recording and management of data;

5) Procedures defining corrective actions for off control point chemistry conditions; and

(2) Alabama Power Company, pursuant to Section 103 of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess but not operate the facility at the designated location in Houston County, Alabama in accordance with the procedures and limitations set forth in this renewed license.

(3) Southern Nuclear, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

(4) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(5) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Southern Nuclear is authorized to operate the facility at reactor core power levels not in excess of 2775 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 216 are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

(3) Deleted per Amendment 144
(4) Deleted per Amendment 149
(5) Deleted per Amendment 144

3.7 PLANT SYSTEMS

3.7.5 Auxiliary Feedwater (AFW) System

LCO 3.7.5 Three AFW trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

----------NOTE----------
LCO 3.0.4b is not applicable.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Turbine driven AFW train inoperable due to one inoperable steam supply. OR ----------NOTE---------- Only applicable if MODE 2 has not been entered following refueling. --- One turbine driven AFW pump inoperable in MODE 3 following refueling.	A.1 Restore affected equipment to OPERABLE status.	7 days
B. One AFW train inoperable for reasons other than Condition A.	B.1 Restore AFW train to OPERABLE status.	72 hours

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Turbine driven AFW train inoperable due to one inoperable steam supply. AND One motor driven AFW train inoperable.	C.1 Restore the steam supply to the turbine driven train to OPERABLE status. OR C.2 Restore the motor driven AFW train to OPERABLE status.	24 hours 24 hours
D. Required Action and associated Completion Time of Condition A, B, or C not met. OR Two AFW trains inoperable for reasons other than Condition C.	D.1 Be in MODE 3. AND D.2 Be in MODE 4.	6 hours 12 hours
E. Three AFW trains inoperable.	E.1 ----------NOTE---------- LCO 3.0.3 and all other LCO Required Actions requiring MODE changes are suspended until one AFW train is restored to OPERABLE status. ---------- Initiate action to restore one AFW train to OPERABLE status.	 Immediately

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.7.5.1	----------NOTE---------- AFW train(s) may be considered OPERABLE during alignment and operation for steam generator level control, if it is capable of being manually realigned to the AFW mode of operation. ---------- Verify each AFW manual, power operated, and automatic valve in each water flow path, and in both steam supply flow paths to the steam turbine driven pump, that is not locked, sealed, or otherwise secured in position, is in the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.7.5.2	----------NOTE---------- Not required to be performed for the turbine driven AFW pump until 24 hours after ≥ 1005 psig in the steam generator. ---------- Verify the developed head of each AFW pump at the flow test point is greater than or equal to the required developed head.	In accordance with the Inservice Testing Program.
SR 3.7.5.3	----------NOTE---------- AFW train(s) may be considered OPERABLE during alignment and operation for steam generator level control, if it is capable of being manually realigned to the AFW mode of operation. ---------- Verify each AFW automatic valve that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.7.5.4	----------NOTES---------- 1. Not required to be performed for the turbine driven AFW pump until 24 hours after ≥ 1005 psig in the steam generator. 2. AFW train(s) may be considered OPERABLE during alignment and operation for steam generator level control, if it is capable of being manually realigned to the AFW mode of operation. --- Verify each AFW pump starts automatically on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.7.5.5	Verify the turbine driven AFW pump steam admission valves open when air is supplied from their respective air accumulators.	In accordance with the Surveillance Frequency Control Program

UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 219 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-2

AND

AMENDMENT NO. 216 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-8

SOUTHERN NUCLEAR OPERATING COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-348 AND 50-364

1.0 INTRODUCTION

By letter dated December 21, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17355A177), Southern Nuclear Operating Company, Inc., (SNC, the licensee) submitted a request to change the Joseph M. Farley Nuclear Plant (FNP or Farley), Units 1 and 2, Technical Specifications (TSs).

The licensee proposed to modify TS 3.7.5, "Auxiliary Feedwater (AFW) System," to establish a new Completion Time for the Condition where one steam supply to the turbine driven AFW pump is inoperable concurrent with an inoperable motor driven AFW train. In addition, the licensee proposed specific Conditions and Action requirements: (1) for when two motor driven AFW trains are inoperable at the same time and; (2) for when the turbine driven AFW train is inoperable either (a) due solely to one inoperable steam supply, or (b) due to reasons other than one inoperable steam supply.

The licensee stated that the proposed changes are consistent with U.S. Nuclear Regulatory Commission (NRC)-approved Technical Specification Task Force (TSTF) Traveler, TSTF-412, Revision 3, "Provide Actions for One Steam Supply to Turbine Driven AFW/EFW [Emergency Feedwater] Pump Inoperable" dated January 10, 2007 (ADAMS Accession No. ML070100363). NRC approval of TSTF-412, Revision 3, was announced in the *Federal Register* on July 17, 2007 (72 FR 39089).

2.0 REGULATORY EVALUATION

2.1. Description of Auxiliary Feedwater System

The AFW System consists of two motor driven AFW pumps and one steam turbine driven pump configured into three trains. The pumps are equipped with recirculation lines to prevent pump operation against a closed system. Each motor driven AFW pump is powered from an independent Class 1E power supply and feeds all steam generators through a common header. The steam turbine driven AFW pump receives steam from two main steam lines upstream of the main steam isolation valves. Each of the steam feed lines will supply 100 percent of the requirements of the turbine driven AFW pump. The turbine driven AFW pump supplies a common header capable of feeding all steam generators via S solenoid air operated control valves actuated by the Engineered Safety Feature Actuation System (ESFAS). Thus, the requirement for diversity in motive power sources for the AFW System is met.

The function of the AFW system is to ensure a sufficient supply of cooling water to the Steam Generators when main feedwater is not available. The design of the AFW system ensures that the RCS can be cooled down to less than 350°F from normal operating conditions in the event of any of the following incidents:

- Loss of Normal Feedwater,
- Loss of Offsite Power,
- Feed Line Break,
- Main Steam Line Break,
- Accidental Depressurization of the Steam Generators (SGs),
- SG Tube Rupture,
- High Energy Line Break,
- Small Break Loss-of-Coolant Accident (LOCA),
- Cooldown following a Reactor Trip,
- Station Blackout.

Each motor-driven AFW pump delivers a total of at least 285 gpm [gallons per minute] to all SGs which are at a pressure of 1138 psia [pounds per square inch absolute]. The minimum flow requirement for a motor-driven AFW pump is based on a high energy line break in the steam supply line to the turbine-driven AFW pump. In this scenario, only one motor-driven AFW pump will be the source of AFW. The turbine-driven AFW pump delivers a total of at 350 gpm to all SGs which are at a pressure of 1138 psia. The minimum requirement for the turbine-driven AFW pump is based on a station blackout event. In this scenario, the turbine-driven AFW pump will be the only source of AFW. Additionally, any single AFW pump (turbine or motor-driven) is capable of providing sufficient flow (350 gpm) to all SGs at a pressure of 1020 psia to cooldown the reactor coolant system (RCS) to residual heat removal (RHR) entry conditions during a normal cooldown of the unit (not a reactor trip). For all other incidents listed above, except for the high energy line break in the steam supply to the turbine-driven AFW pump, the station blackout event, and the normal unit cooldown discussed previously, two out of three AFW pumps (motor or turbine-driven combination) are required to satisfy the flow demand.

2.2 Proposed TS Changes

The licensee proposed the following modifications to TS 3.7.5:

- Revise the first part of the description of Condition A for an inoperable turbine-driven AFW pump steam supply to state "Turbine driven AFW train inoperable due to one inoperable steam supply";

- Add a new Condition C for an inoperable turbine-driven AFW pump steam supply concurrent with an inoperable motor-driven AFW pump;

- Revise the description of existing Condition C to exclude the situation where two AFW trains are inoperable due to the described new Condition C and;

  Reletter existing Conditions C and D to Conditions E and D, respectively.

2.3 Regulatory Requirements and Guidance

The NRC staff considered the following regulatory requirements, guidance, and licensing information during its review of the proposed changes:

In Title 10 of the *Code of Federal Regulations* (10 CFR) 50.36, the Commission established its regulatory requirements related to the content of TSs. Pursuant to 10 CFR 50.36(c), TSs are required to include items in the following categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls.

Regulations in 10 CFR 50.36(c)(2) states, in part: "Limiting conditions for operation are the lowest functional capability or performance levels of equipment required for safe operation of the facility. When a limiting condition for operation of a nuclear reactor is not met, the licensee shall shutdown the reactor or follow any remedial action permitted by the technical specifications until the condition can be met."

The NRC staff's guidance for review of the TSs is in NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR Edition," Section 16.0, "Technical Specifications." The U.S. Nuclear Regulatory Commission (NRC) staff has prepared standard technical specifications (STSs) for each of the light–water reactor nuclear steam supply systems and associated balance–of–plant equipment systems. The guidance specifies that the staff review whether content and format of proposed TSs are consistent with the applicable STSs. Where TS provisions depart from the reference TSs, the staff determines whether proposed differences are justified by uniqueness in plant design or other considerations. The applicable current STS for FNP are contained in NUREG-1431, "Standard Technical Specifications – Westinghouse Plants," Revision 4.0. TSTF-412, Revision 3, was approved by the NRC prior to issuance of NUREG-1431, Revision 4.0. TSTF-412, Revision 3, has been incorporated into NUREG-1431, Revision 4.

Regulations in 10 CFR 50, Appendix A, the Commission established regulatory requirements related to Auxiliary Feedwater Systems. General Design Criteria (GDC) 34, "Residual Heat Removal" and GDC 44, "Cooling Water," state that the AFW system is required to assure (1) the capability to transfer heat loads from the reactor system to a heat sink under both normal operating and accident conditions; (2) the redundancy of components for performance of the

safety function under accident conditions, assuming a single active component failure; and (3) the capability to isolate components, subsystems, or piping if required to maintain system safety function. FNP Updated Final Safety Analysis Report, Sections 3.1.30 and 3.1.40 (ADAMS Package No. ML17117A380) contain the licensee's evaluation of the FNP's conformance to GDC 34 and 44, respectively.

3.0 TECHNICAL EVALUATION

The NRC staff evaluated the licensee's application to determine if the proposed changes are consistent with the regulations, guidance, and licensing information discussed in Section 2.3 of this safety evaluation. In determining whether an amendment to a license will be issued, the Commission is guided by the considerations that govern the issuance of initial licenses to the extent applicable and appropriate. Among the considerations are whether the TSs, as amended, would provide the necessary limiting conditions for operation per 10 CFR 50.36(c)(2). In making its determination as to whether to amend the license, the NRC staff considered those regulatory requirements that are automatically conditions of the licenses through 10 CFR 50.54. Where the regulations already condition the licenses, there is no need for a duplicative requirement in the TSs; the regulations provide the necessary reasonable assurance of the health and safety of the public.

3.1 Limiting Condition for Operation (LCO) 3.7.5, Changes to Condition A

The description of Condition A is modified to refer to the inoperability of a turbine driven AFW train due to an inoperable steam supply, instead of referring to the inoperability of a turbine driven AFW pump. This change has the effect of making the description of Condition A train oriented instead of component oriented, consistent with the other Conditions that are included in TS 3.7.5. The train oriented approach is consistent with the preferred approach that is generally reflected in the STS, and therefore, the NRC staff finds the proposed change acceptable. No other changes to the description of Condition A or the associated Required Actions and Completion Times were proposed.

3.2 LCO 3.7.5, New Condition C

A new Condition C with two possible Required Actions (C.1 OR C.2) is proposed for the situation when the turbine driven AFW train is inoperable due to one inoperable steam supply concurrent with one inoperable motor driven AFW train. Required Action C.1 requires restoration of the affected steam supply to operable status within 24 hours. Alternatively, Required Action C.2 requires restoration of the inoperable motor driven AFW train within 24 hours.

The proposed 24 hour Completion Time is applicable to plants that may not provide sufficient flow to the steam generators (SGs) in accordance with accident analyses assumptions if a main steam line break (MSLB) or feedwater line break (FLB) were to occur that renders the remaining steam supply to the turbine driven AFW pump inoperable (a concurrent single failure is not assumed). The STS typically allows a 72 hour or longer Completion Time for Conditions where the remaining operable equipment is able to mitigate postulated accidents without assuming a concurrent single active failure. In FNP's case, a 24 hour Completion Time is proposed by the licensee for the situation where the AFW system would be able to perform its function for most postulated events, and would only be challenged by a MSLB or FLB that renders the remaining operable steam supply to the turbine driven AFW pump inoperable.

The justification for 24 hours for the Completion Time is based on the remaining operable steam supply to the turbine driven AFW pump and the continued functionality of the turbine driven AFW train, the remaining operable motor driven AFW train, and the low likelihood of an event occurring during this 24 hour period that would challenge the capability of the AFW system to provide feedwater to the SGs. The proposed Completion Time for this particular event is consistent with the STS in that the proposed Completion Time is less than the 72 hours that is allowed for the situation where accident mitigation capability is maintained. Therefore, the NRC staff finds that the proposed 24 hour Completion Time is acceptable.

3.3 LCO 3.7.5, Relettered Condition D

The current Condition C is relettered as Condition D. This Condition has been modified to incorporate changes brought on by the addition of new Condition C. The first of the two listed Conditions under Condition D has been modified and now applies to the situation where the Required Action and associated Completion Time of Condition A, B, or C are not met. This section of Condition D is modified to also apply to the new Condition C when the Completion Time that is specified for new Condition C is not met. The NRC staff considers this to be appropriate and consistent with existing STS 3.7.5 requirements to place the plant in a mode where the Condition does not apply when the Required Actions are not met.

The second listed Condition under Condition D (following the first "OR") is modified from "Two AFW trains inoperable" to "Two AFW trains inoperable for reasons other than Condition C." This change is necessary to recognize the situation specified by new Condition C where one motor driven AFW train is allowed to be inoperable at the same time that the turbine driven AFW train is inoperable due to an inoperable steam supply to the pump turbine. Therefore, the NRC staff finds the proposed change to be acceptable.

The Required Actions associated with this Condition were renamed from C.1 AND C.2 to D.1 AND D.2 but not otherwise changed. Required Action D.1 requires the plant to be in Mode 3 in 6 hours, and Required Action D.2 requires the plant to be in Mode 4 in 12 hours. This change is editorial as no other changes are involved. Therefore, the NRC staff finds this proposed change acceptable.

3.4 LCO 3.7.5, Relettered Condition E

Because current Condition C is relettered as Condition D, current Condition D is relettered as Condition E. This change is purely editorial as no other changes are involved. Therefore, the NRC staff finds this proposed change acceptable.

3.5 NRC Staff Conclusion

The proposed changes are consistent with NRC practices and policies as generally reflected in the STS and as reflected by applicable precedents that have been approved. The NRC staff concludes that the requirements of 10 CFR 50.36(c)(2) continue to be met, because the minimum performance level of equipment needed for safe operation of the facility as specified in TS LCO 3.7.5 remains unchanged and appropriate remedial measures are specified if the LCO is not met. The changes to TS LCO 3.7.5 Actions are consistent with the guidance of Section 16.0 of NUREG-0800, in that the proposed changes are generally consistent with the STS incorporated in NUREG-1431. Therefore, the NRC staff finds the proposed changes to TS 3.7.5 acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of Alabama official was notified of the proposed issuance of the amendments on June 1, 2018. On June 1, 2018, the State official confirmed that the State of Alabama had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding published in the Federal Register on February 13, 2018 (83 FR 6234). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: M. Hamm, NRR

Date: June 27, 2018

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 – ISSUANCE OF AMENDMENTS RELATED TO TECHNICAL SPECIFICATION TASK FORCE TRAVELER 412, REVISION 3 (EPID L-2017-LLA-0429) DATED JUNE 27, 2018

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