

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

W. L. STEWART
VICE PRESIDENT
NUCLEAR OPERATIONS

January 15, 1986

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Serial No. 85-880
NO/JBL:vlh
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNIT NOS. 1 AND 2
INADEQUATE CORE COOLING INSTRUMENTATION SYSTEM

This letter provides our response to your letter of December 4, 1985, "Proposed Inadequate Core Cooling Instrumentation System (ICCIS) - Surry Power Station Unit Nos. 1 and 2," and addresses aspects of Virginia Electric and Power Company's efforts to meet the requirements of NUREG 0737 Item II.F.2, "Instrumentation for Detection of Inadequate Core Cooling," as discussed in your attached Safety Evaluation.

Your letter indicated that our currently installed Reactor Vessel Level Indicating Systems (RVLIS) for Units 1 and 2 was acceptable upon restoration of the functional operability for the Unit 2 second train prior to the next refueling outage. The Unit 2 second train RVLIS has been functionally operable since the end of the June 1985 refueling outage. We have also revised the Surry Technical Specifications to contain requirements for operability and surveillance of RVLIS. Therefore, per your Safety Evaluation, the RVLIS for both Units 1 and 2 is acceptable.

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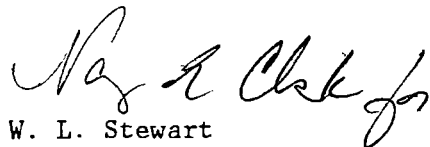
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Add: RSB-Berlinger

As noted in our letters dated July 6, 1984, September 25, 1984, and July 3, 1985, we had originally intended to complete the Control Room Design Review (CRDR) process and update our Emergency Operating Procedures (EOPs) based on the Westinghouse Owners Group (WOG) Emergency Response Guidelines (ERGs), Revision 1, concurrently. However, as documented in our letter dated August 21, 1985, we separated these processes in an effort to better facilitate the timely submittal of our CRDR Final Summary Report. Therefore, the updating of our EOPs will not be concurrent with the CRDR Final Summary Report submittal. We will inform you upon completion of updating the EOPs to Revision 1 of the WOG ERGs.

Due to a change in the Unit 1 outage schedule with a corresponding impact on equipment delivery dates, we plan to submit a letter in the near future requesting an extension to our current implementation schedules for the Upgraded Core Exit Thermocouples and Subcooling Margin Monitor. The request will include sufficient information to justify that such an extension is warranted. No change in the Unit 2 implementation schedule appears to be necessary. We will inform you of the completion of the installation of the Upgraded Core Exit Thermocouples and Subcooling Margin Monitor.

A proposed change to the Surry Technical Specifications will be submitted to add the Core Exit Thermocouple (CET) system to the Accident Monitoring Instrumentation in Tables 3.7-6 and 4.1-2. We will submit the proposed Technical Specification change within 30 days following the completion of the CET upgrade.

Very truly yours,


W. L. Stewart

cc: Dr. J. Nelson Grace
Regional Administrator
NRC Region II

Mr. Roger D. Walker, Director
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NRC Region II

Mr. Donald J. Burke
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Mr. Terence L. Chan
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