

10 CFR 50.55a

May 24, 2018

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

Limerick Generating Station, Units 1 and 2
Renewed Facility Operating License Nos. NPF-39 and NPF-85
NRC Docket Nos. 50-352 and 50-353

Subject: Supplemental Information - End of Interval Relief Request Associated with the Third Ten-Year Inservice Inspection (ISI) Interval

- References:**
- 1) Letter from J. Barstow (Exelon Generation Company, LLC) to U.S. Nuclear Regulatory Commission, "End of Interval Relief Request Associated with the Third Ten-Year Inservice Inspection (ISI) Interval," dated September 29, 2017
 - 2) Email from V. Sreenivas (U.S. Nuclear Regulatory Commission) to T. Loomis (Exelon Generation Company, LLC), "Limerick Units 1 and 2: Request for Additional Information (RAI): Relief Request I3R-23 Regarding Limited Examination Coverage (EPID: L-2017-LLR-0098)," dated December 19, 2017
 - 3) Letter from J. Barstow (Exelon Generation Company, LLC) to U.S. Nuclear Regulatory Commission, "End of Interval Relief Request Associated with the Third Ten-Year Inservice Inspection (ISI) Interval," dated January 17, 2018
 - 4) Email from V. Sreenivas (U.S. Nuclear Regulatory Commission) to T. Loomis (Exelon Generation Company, LLC), "Limerick: Relief request (RR) I3R-23 to request relief from the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), EPID: L-2017-LLR-0098," dated May 15, 2018

In the Reference 1 letter, in accordance with 10 CFR 50.55a, "Codes and standards," paragraph (g)(5)(iii), Exelon Generation Company, LLC (Exelon), requested relief from the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (B&PV) Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components." This relief request applies to the third ten-year Inservice Inspection (ISI) interval, which concluded on January 31, 2017, for the Limerick Generating Station, Units 1 and 2. The third ten-year ISI interval complied with the ASME Boiler and Pressure Vessel Code, Section XI, 2001 Edition with 2003 Addenda.

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In the Reference 2 email, the U.S. Nuclear Regulatory Commission requested additional information. Reference 3 was our response to that email.

In the Reference 4 email, the U.S. Nuclear Regulatory Commission requested additional information regarding the Emergency Service Water (ESW) and Residual Heat Removal Service Water (RHRSW) system component supports. These component supports (Examination Category F-A, Item Number F1.40) are for the ESW and RHRSW system pumps. Upon further review, Exelon is withdrawing the ESW and RHRSW pump supports from this request.

There are no regulatory commitments in this response.

If you have any questions concerning this response, please contact Tom Loomis at (610) 765-5510.

Respectfully,



David P. Helker
Manager - Licensing & Regulatory Affairs
Exelon Generation Company, LLC

cc: USNRC Region I, Regional Administrator
USNRC Senior Resident Inspector, LGS
USNRC Project Manager, LGS
R. R. Janati, Pennsylvania Bureau of Radiation Protection