

Vogle PEmails

From: Comar, Manny
Sent: Tuesday, May 22, 2018 5:21 PM
To: Vogle PEmails
Subject: ACCEPTANCE FOR REVIEW OF SOUTHERN NUCLEAR OPERATING COMPANY'S LICENSE AMENDMENT AND EXEMPTION REQUEST LAR-18-014 (Containment Sump level Instrumentation)
Attachments: Acceptance Letter - VEGP Units 3 and 4 - LAR 18-044.docx

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From: Comar, Manny

Created By: Manny.Comar@nrc.gov

Recipients:

"Vogtle PEmails" <Vogtle.PEmails@nrc.gov>

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May 23, 2018

Mr. Brian H. Whitley, Director
Regulatory Affairs
Southern Nuclear Operating Company, Inc.
42 Inverness Center Parkway
BIN B237
Birmingham, AL 35242

SUBJECT: ACCEPTANCE REVIEW OF SOUTHERN NUCLEAR OPERATING
COMPANY'S REQUEST FOR LICENSE AMENDMENT FOR
THE VOGTLE ELECTRIC GENERATING PLANT, UNITS 3 AND 4:
CONTAINMENT SUMP LEVEL INSTRUMENTATION (LAR 18-014)

Dear Mr. Whitley:

By letter dated April 26, 2018 (Agencywide Documents Access and Management System Accession No. ML18116A138), Southern Nuclear Operating Company (SNC/licensee) submitted a request for a License Amendment Request (LAR) 18-014 for the Combined License (COL) Numbers NPF-91 and NPF-92, for Vogtle Electric Generating Plant (VEGP) Units 3 and 4, respectively.

The requested amendment proposes changes to COL Appendix C, with corresponding changes to the associated plant-specific Tier 1 information, and involves associated Tier 2 information in the Updated Final Safety Analysis Report (UFSAR) (which includes the plant-specific Design Control Document (DCD) Tier 2 information). Pursuant to the provisions of 10 CFR 52.63(b)(1), you also requested an exemption from elements of the design as certified in the 10 CFR Part 52, Appendix D design certification rule for the plant-specific DCD Tier 1 material departures.

The requested amendment proposes changes to COL Appendix C (and plant-specific Tier 1) to reflect a new design of containment sump level sensors which affects the acceptance criterion for the detected containment sump level change test and the associated minimum detectable unidentified leakage rate in plant-specific DCD Tier 2 information.

The purpose of this letter is to provide the results of the NRC staff's acceptance review of this LAR. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Title 10 of the *Code of Federal Regulations* (10 CFR) 50.90, an amendment to the license must fully describe the changes requested, and follow as far as applicable, the form prescribed for original applications. Section 52.79 of the 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application against the regulatory requirements and has concluded that it provides technical information in sufficient detail to enable the NRC staff to

complete its detailed technical review and make an independent assessment regarding the acceptability of the proposed amendment in terms of regulatory requirements and the protection of public health and safety and the environment.

Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may be instances in which issues that impact the NRC staff's ability to complete the detailed technical review are identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the NRC staff's detailed technical review by separate correspondence.

If you have any questions, please contact me at (301) 415-3863 or Manny.Comar@nrc.gov.

Sincerely,

R/A

Manny Comar, Senior Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket Nos.: 52-025
52-026

