

50-244

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

INCIDENT REPORT

TO:  
Mr. James P. O'Reilly

FROM:  
Rochester Gas & Elec. Corp.  
Rochester, N. Y.  
L. D. White, Jr.

DATE OF DOCUMENT  
7/18/77

DATE RECEIVED  
7/21/77

LETTER  
 ORIGINAL  
 COPY

NOTORIZED  
 UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

1 CC

DESCRIPTION

ENCLOSURE

**ACKNOWLEDGED**

Licensee Event Report (RO 50-244/77-08) on  
7/5/77 concerning the Air Ejector and  
Blowdown radiation monitors alarming.....

PLANT NAME:

Ginna Unit No. 1

(1-P)

(2-P)

RJL 7/21/77

**DO NOT REMOVE**

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:  
W/ 3 CYS FOR ACTION  
LIC ASST.:

Schwartz (4)

INTERNAL DISTRIBUTION

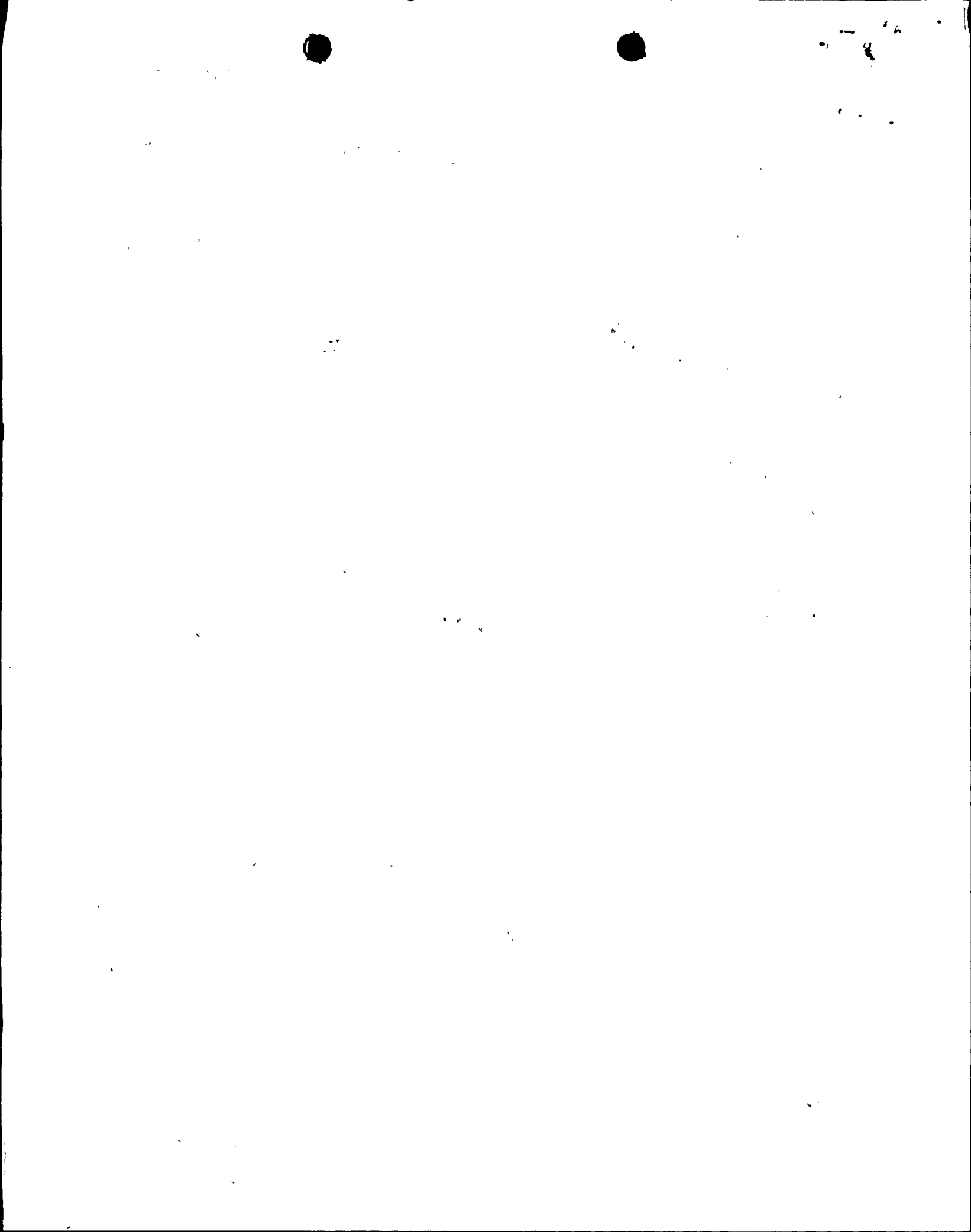
- REG FILE
- NRC PDR
- I & E (2)
- MIPC
- SCHROEDER/IPPOLITO
- HOUSTON
- NOVAK/CHECK
- GRIMES
- KNIGHT
- BUTLER
- HANAUER
- TEDESCO
- EISENHUT
- BAER
- SHAO
- VOLLMER/BUNCH
- KREGER/ J. COLLINS
- ROSA

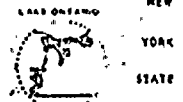
EXTERNAL DISTRIBUTION

LPDR: Rochester, NY  
TIC:  
NSIC:  
ACRS (16) SENT AS CAT. B

CONTROL NUMBER

772020233





ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

LEON D. WHITE, JR.  
VICE PRESIDENT

TELEPHONE  
AREA CODE 716 546-2700

July 18, 1977

Mr. James P. O'Reilly, Director  
U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

**Regulatory Docket File**

Subject: Reportable Occurrence 77-08 (14-day report), "B" Steam  
Generator tube leak  
R. E. Ginna Nuclear Power Plant, Unit No. 1  
Docket No. 50-244

Dear Mr. O'Reilly:

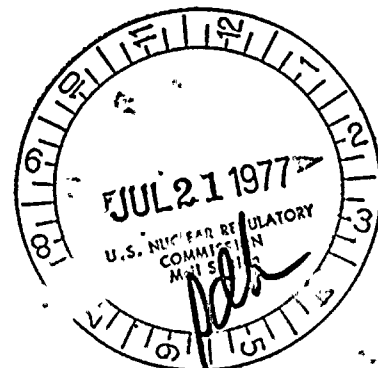
In accordance with Technical Specifications, Article 6.9.2.a, the attached report of Reportable Occurrence 77-08, 14-day, is hereby submitted. Two additional copies of this letter and the attachment are enclosed.

Very truly yours,

L. D. White, Jr.

Attachment

cc: Dr. Ernst Volgenau (40)  
Mr. William G. McDonald (3)



772020233



Handwritten scribbles and marks in the top right corner, possibly including a date or initials.

Handwritten text at the bottom left corner, possibly a date or reference number.

# LICENSEE EVENT REPORT

CONTROL BLOCK: 

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(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME: 

01	N	Y	R	E	G	1
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 7 8 9 14

LICENSE NUMBER: 

0	0	-	0	0	0	0	0	-	0	0
---	---	---	---	---	---	---	---	---	---	---

 15 25

LICENSE TYPE: 

4	1	1	1	1
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 26 30

EVENT TYPE: 

0	1
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 31 32

REPORT TYPE: 

01	CONT
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 7 8

CATEGORY: 

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 57 58

REPORT SOURCE: 

L
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 59 60

DOCKET NUMBER: 

0	5	0	-	0	2	4	4
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 61 68

EVENT DATE: 

0	7	0	5	7	7
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 69 74

REPORT DATE: 

0	7	1	8	7	7
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 75 80

### EVENT DESCRIPTION

02 | During normal operations the Air Ejector and blowdown radiation monitors alarmed. | 7 8 9 80

03 | Analysis of the B Steam Generator blowdown verified Iodine activity from a calculated | 7 8 9 80

04 | leak rate of 0.006 gpm. The leaking tube was located with a hydrostatic test, and | 7 8 9 80

05 | Eddy Current testing was performed to determine if a pattern of degradation existed. | 7 8 9 80

06 | About 300 tubes were examined which included tubes in (cont'd. on attached sheet) | 7 8 9 80

SYSTEM CODE: 

C	B
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 7 8 9 10

CAUSE CODE: 

E
---

 11

COMPONENT CODE: 

H	T	E	X	C	H
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 12 17

PRIME COMPONENT SUPPLIER: 

N
---

 43

COMPONENT MANUFACTURER: 

W	1	2	0
---	---	---	---

 44 47

VIOLATION: 

N
---

 48

### CAUSE DESCRIPTION

08 | The failure of the tube R45C54 and the defect indications in tubes R44C55, 56, 57 and | 7 8 9 80

09 | 58 are postulated to be caused by stress corrosion cracking from the inside surface of | 7 8 9 80

10 | the tubes. These inside diameter cracks are theorized to be (cont'd. on attached sheet) | 7 8 9 80

FACILITY STATUS: 

E
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 7 8 9

% POWER: 

1	0	0
---	---	---

 10 12 13

OTHER STATUS: 

NA
----

 44

METHOD OF DISCOVERY: 

C
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 45

DISCOVERY DESCRIPTION: 

Rad monitors and sample analysis
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 46 80

FORM OF ACTIVITY RELEASED: 

Z
---

 7 8 9

CONTENT OF RELEASE: 

Z
---

 10

AMOUNT OF ACTIVITY: 

NA
----

 44

LOCATION OF RELEASE: 

NA
----

 45 80

### PERSONNEL EXPOSURES

13 | NUMBER: 

0	0	0
---	---	---

 7 8 9 11

TYPE: 

Z
---

 12

DESCRIPTION: 

NA
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 13 80

### PERSONNEL INJURIES

14 | NUMBER: 

0	0	0
---	---	---

 7 8 9 11

DESCRIPTION: 

NA
----

 12 80

### OFFSITE CONSEQUENCES

15 | 

NA
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 7 8 9 80

### LOSS OR DAMAGE TO FACILITY

16 | TYPE: 

Z
---

 7 8 9 10

DESCRIPTION: 

NA
----

 11 80

### PUBLICITY

17 | News release. | 7 8 9 80

### ADDITIONAL FACTORS

18 | 

NA
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 7 8 9 80

19 | 

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 7 8 9 80

Licensee Event Report  
Reportable Occurrence 50-244/77-08

Event Description (cont'd.)

the area of the leaking tube and other selected areas with previous indications. These examinations revealed four additional tubes, in the area of the leaking tube, with similar (ID) indications. These four tubes and the leaking tube were explosively plugged. (Reportable Occurrence 77-08, 14-day).

Cause Description (cont'd.)

caused by a dent at the tube to tube sheet interface which induces stress into the tube wall resulting in cracking. There are not any chemical species postulated to be involved in this cracking phenomena due to the purity of the primary system's water, which flows through the tubes.

Corrective action taken included eddy current examination of surrounding tubes and tubes in areas where previous inservice inspections had revealed small defect indications. There were no other tubes identified of those examined which exhibited further degradation. Therefore, the failed tube and four others with similar (ID) indications were plugged. Further investigation is continuing with Westinghouse and RG&E personnel involvement studying the generic PWR tube failure problem. At the next refueling outage, an inservice inspection of the steam generator tubes will include tubes in the area of this failure.

01 01 31 31 3 18

1977 JUL 21 PM 2 18

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