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### 2018 Materials Programs Technical Information Exchange Meeting PWROG Materials Committee Update

Chris Wax, PWROG MSC Chair (APS) May 22, 2018 Washington, DC

P R E S S U R I Z E D W A T E R R E A C T O R O W N E R S G R O U P

2018 Materials Programs Technical Information Exchange Meeting **PWR Owners Group MSC Agenda** 

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- PWROG MSC Key Focus Areas for 2018/2019
- Future PWROG Meetings
- MSC PWROG Core/Planning Team Organization and Key Contacts

2018 Materials Programs Technical Information Exchange Meeting **PWROG MSC Key Focus Areas for 2018/2019** <sup>(1/10)</sup>



# Scale and Impact of Uncertainty in Fluence Determinations for Reactor Vessel Internals

- The purpose of this ongoing program is to investigate the factors contributing to the uncertainty in the calculation of the fluence for the reactor vessel internals
  - The project intends to demonstrate that the fluence calculation uncertainty for internals does not change the individual MRP-227 component categorization for defining actions needed to manage material aging in the reactor vessel internals
  - The PWROG plans to submit a topical report (early 2021) to address NRC fluence uncertainty related Regulatory Questions/RAIs

## 2018 Materials Programs Technical Information Exchange Meeting **PWROG MSC Key Focus Areas for 2018/2019** <sup>(2/10)</sup>



#### Qualification/Refinement of Fluence Determination in Non-Traditional Reactor Vessel Beltline Locations

- The purpose of this ongoing program is to collect supplementary neutron fluence/activity measurements in the extended beltline regions of operating PWRs
  - The project intends to provide;
    - $_{\odot}~$  a basis for refinement of the fluence analysis methodology from 2D to 3D modeling
    - a basis for confirmation of the fluence uncertainty in extended beltline locations in a manner similar to Regulatory Guide (RG) 1.190
  - The project will provide a consistent, documented industry position on the extended beltline, and distributed costs, in keeping with the 'Nuclear Promise'.

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# Demonstrate Appendix G Margins for PWR RPV Nozzles and Beltline

- The purpose of this ongoing program is to demonstrate that the RPV nozzle corner pressure-temperature (P-T) limit curves are bounded by the licensed traditional P-T limit curves for the US PWRs
  - The intent of the program is to justify the use of the RPV beltline & flange region as the limiting region to be used for P-T curves thereby demonstrating that current approved methodologies comply with Appendix G

 $\,\circ\,$  WCAP-14040-A , CE NPSD-683-A, and BAW-10046A

 PWROG-15109-NP, "PWR Pressure Vessel Nozzle Appendix G Evaluation" was submitted to the NRC for review in March 2018

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curve as comparing apples and oranges and was uncomfortable with this approach Report now includes originally postulated small flaws 0 and 2.1" flaws (comparable to beltline 1/4T size) using

- Appendix G SIF solution for various limiting geometries
- Also gathered all the US PWR CD curves to show a more convincing story



The report is more robust and should make NRC review

Demonstrate Appendix G Margins for

A thorough review was provided and all

Provided report to EPRI for a 3<sup>rd</sup> party review

PWR RPV Nozzles and Beltline <sup>(2/3)</sup>

comments were addressed

easier

2250 Westinghouse 4-Loop Baseline **Outlet Nozzle Limit** 2000 (Flaw Size = 2.156" Westinghouse 4-Loop Bounding nghouse 4-Loop Baselin 1750 **Outlet Nozzle Limit** Inlet Nozzle Limit (Flaw Size = 2.156) (Flaw Size = 2 156" 1500 Westinghouse 4-Loop Boundin Inlet Nozzle Limit 1250 (psig) (Flaw Size = 2.156 <u>و</u> 1000 Pressur 750 CE System 80 **Outlet Nozzle Limit** (Flaw Size = 2.266" 500 CE System 80 100°F/hour Cooldown Inlet Nozzle Limit 250 (Flaw Size = 2.266") (RTT<sub>0</sub> = 43°F) 1/4T Beltline Size Circular Flaw 0 50 100 150 200 250

Temperature (°F)

Owners



### 2018 Materials Programs Technical Information Exchange Meeting PWROG MSC Key Focus Areas for 2018/2019 (4/10)

2018 Materials Programs Technical Information Exchange Meeting **PWROG MSC Key Focus Areas for 2018/2019** <sup>(5/10)</sup>



### Demonstrate Appendix G Margins for PWR RPV Nozzles and Beltline <sup>(3/3)</sup>

- Final Report Excerpts PWROG-15109-NP
  - The conservatively-derived generic nozzle Appendix G P-T limit curves were compared to all the NRC approved P-T limit curves for the U.S. PWR fleet
  - The results demonstrated that nozzle P-T limit curves were bounded in every case by the NRC approved U.S. PWR P-T limit curves. The nozzle P-T limit curve results are applicable through 60 years of operation
  - With licensee evaluation of SLR or other operational changes, updated nozzle fluence projections can be compared to the values used in this work for applicability. If the projected nozzle corner fluence remains less than the screening criterion of 4.28 x 10<sup>17</sup> n/cm<sup>2</sup>, then this analysis is applicable.
  - Based on the results of this detailed conservative assessment, the current licensed traditional U.S. PWR P-T limit curves that have used the NRC approved methods of developing P-T limit curves (WCAP-14040-A, Rev. 4, CE NPSD-683-A, Rev. 06 and BAW-10046A, Rev. 2) bound the nozzle P-T limit curves

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### **Transitioning RV Integrity to Direct Fracture Toughness**

- The purpose of this ongoing program is to develop an acceptable method for any licensee to use irradiated fracture toughness data to improve or demonstrate margin in P-T curves.
  - The project intends to:
    - Reduce the number of P-T curve submittals by demonstrating margin of existing curves
    - Reduce uncertainty by providing irradiated fracture toughness data on more limiting materials
- Concept presented to NRC in March 2016
  - NRC feedback incorporated and updated plan presented at August 2016 EPRI Materials Reliability Conference

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### Transitioning RV Integrity to Direct Fracture Toughness (2/2)

- Preparation of Topical Report
  - Follows to extent possible previously approved precedents and methods
    - "Safety Evaluation by the Office of Nuclear Reactor Regulation Regarding Amendment of the Kewaunee Nuclear Power Plant License to Include the Use of a Master Curve-Based Methodology for Reactor Pressure Vessel Integrity Assessment," May 2001
    - "Initial RT<sub>NDT</sub> of Linde 80 Weld Materials," BAW-2308, Rev. 2-A, March 2008
    - NRC approved code case N-629
    - o 10CFR50.61a
    - RG 1.99R2 adjustment methods
  - Topical report with basis is under preparation
    - Pre-submittal meeting Spring 2019
    - Submittal planned for Summer 2019

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#### Development of Generic Recommendations to Address ID-Initiated and OD-Initiated SCC of PWR Stainless Steel Pressure Boundary Components

- The purpose of this completed program was to proactively address SCC of pressure boundary stainless steel to ensure it is not a safety concern
  - Several cases of leakage due to SCC of stainless steel
    - MRP-236 OE Report
    - NRC Information Notice 2011-04
- Two reports provided to the PWROG MSC, PWROG-17067-NP and PWROG-17054-NP

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- PWROG-17067-NP, Revision 0, "Resource Guide for Selecting Weld Locations for Inspection to Address ID-Initiated SCC of Stainless Steel Piping"
  - Identified locations of interest by developing a SCC susceptibility screening criteria and then applying it to all stainless steel piping locations with a high consequence of rupture
  - Determined locations of interest are adequately addressed by existing inspection requirements (RI-ISI and/or ASME Section XI Category B-J with one-time GALL inspection for small-bore piping) based on
    - Weld locations of interest are part of the piping lines encompassed by the coverage bounds of the existing requirements
    - OE indicates SCC in the weld locations of interest is rare
    - In the unlikely event of SCC, it would be addressed before becoming a safety concern
  - Resource guide is provided for consideration when selecting weld locations to inspection to satisfy existing inspection requirements



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#### NEI 03-08 Good Practice

SCC

• Ensures a consistent level of awareness is communicated to appropriate plant organizations

• Provides a consistent set of attributes for identifying and responding to

addressed before becoming a safety concern

In the unlikely event of SCC in a high consequence location, it would be

- PWROG-17054-NP, Revision 0, "Long-term Strategy for Identifying ODSCC of Stainless Steel Piping"

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- ODSCC is not a safety concern
  - SCC susceptibility is not prevalent in locations with a high consequence of rupture





2018 Materials Programs Technical Information Exchange Meeting Future PWROG Meetings



- June 12-14, 2018 PWROG General Session
- July 23-24, 2018 Steering and Executive Planning Meeting
- August 13-16, 2018 PWROG Joint PWROG Meetings
- October 2-4, 2018 PWROG General Session
- December 10-14, 2018 PWROG Joint PWROG Meetings



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### **Questions?**

The Materials Committee is established to provide a forum for the identification and resolution of materials issues including their development, modification and implementation to enhance the safe, efficient operation of PWR plants.

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