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ROCHESTER GAS AND ELECTRIC CORPORATION 🔹 89 EAST AVENUE, ROCHESTER, N.Y. 14649

LEON D. WHITE, JR. VICE PRESIDENT

TELEPHONE AREA CODE 718 546-2700

Regulatory

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April 25, 1977

Director of Nuclear Reactor Regulation Attention: A. Schwencer, Chief Operating Reactor Branch No. 1 U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Subject: R.E. Ginna Nuclear Power Plant Unit No. 1 Docket No. 50-244

Dear Mr. Schwencer:

This letter is in response to a request from a member of the NRC Staff regarding the Ginna core baffle. The leakage through the "Z" baffle joint at another plant has historically lead to fuel rod failures. During the Ginna 1976 refueling, the fuel assemblies adjacent to the "Z" baffle joints during Cycle 5 were inspected with binoculars and no anomalies were found. In addition, the fuel assemblies adjacent to the "Z" baffle joints during the previous cycle (Cycle 4) were examined by binoculars and were leak tested. The results of the leak testing support the conclusion that Ginna baffle does not have the baffle joint condition seen elsewhere.

Very truly yours,

L. D. White, Jr.



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

LEON D. WHITE, JR. VICE PRESIDENT

TELEPHONE AREA CODE 716 546-2700

April 26, 1977

Director of Nuclear Regulation ATTN: Mr. A. Schwencer, Chief Operating Reactors Branch #1 U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Dear Mr. Schwencer:

Your letter of April 1, 1977 requested that we implement procedures to prevent reactor coolant pump (RCP) starts while in the water solid condition or, in the event such starts could not be avoided, to include procedural steps to ensure that an acceptable reactor coolant system (RCS) temperature profile exists prior to starting a reactor coolant pump.

As discussed in our letter of February 24, 1977, there are times when reactor coolant pump starts are necessary while in the water solid condition. Therefore we will revise our operating procedures to include non-optional prerequisite steps while in a water solid condition to determine the reactor coolant system temperature profile before starting a reactor coolant pump. Appropriate measures will be specified to achieve an acceptable profile prior to starting the pump. In addition, we will conduct a review of our procedures to ensure that they are consistent with our February 24, 1977 submittal.

These procedural revisions and reviews will be completed prior to startup at the end of our present refueling and maintenance outage.

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Sincerely,

L. D. White, Jr.

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