

C 03/14/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)  
DISTRIBUTION FOR INCOMING MATERIAL 50-244

REC: STELLO V                      ORG: WHITE L D                      DOCDATE: 03/08/78  
NRC                                  ROCHESTER GAS & ELEC                      DATE RCVD: 03/08/78

DOCTYPE: LETTER              NOTARIZED: NO                      COPIES RECEIVED  
SUBJECT:    LTR 1 / ENCL 0

FORMATION OF TASK GROUP TO EXAMINE PRIMARY SYSTEM PIPE BREAKS AND  
REASSESS THE ORIGINAL PLANT DESIGN AND ASSURE THE SAFETY OF THE  
TASK GROUP PLANTS SHOULD A PIPE BREAK OCCUR.

PLANT NAME: RE GINNA - UNIT 1                      REVIEWER INITIAL: XRS  
DISTRIBUTOR INITIAL: *ne*

\*\*\*\*\* DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS \*\*\*\*\*

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(DISTRIBUTION CODE A001)

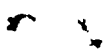
FOR ACTION: BR CHIEF ZIEMANN\*\*LTR ONLY(7)

INTERNAL:	<u>REG FILE**LTR ONLY(1)</u>	NRC PDR**LTR ONLY(1)
	I & E**LTR ONLY(2)	OELD**LTR ONLY(1)
	HANAUER**LTR ONLY(1)	CHECK**LTR ONLY(1)
	EISENHUT**LTR ONLY(1)	SHAO**LTR ONLY(1)
	BAER**LTR ONLY(1)	BUTLER**LTR ONLY(1)
	GRIMES**LTR ONLY(1)	J COLLINS**LTR ONLY(1)
	J. MCGOUGH**LTR ONLY(1)	

EXTERNAL: LPDR'S  
            ROCHESTER, NY\*\*LTR ONLY(1)  
            TIC\*\*LTR ONLY(1)  
            NSIC\*\*LTR ONLY(1)  
            ACRS CAT B\*\*LTR ONLY(16)

DISTRIBUTION: LTR 40              ENCL 0                      CONTROL NBR: 780730343  
SIZE: 1P

\*\*\*\*\* THE END \*\*\*\*\*



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REGULATORY BUCKET FILE COPY

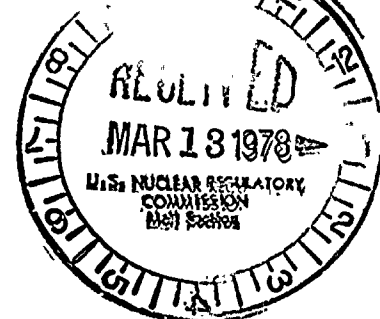
NEW  
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STATE

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

LEON D. WHITE, JR.  
VICE PRESIDENT

TELEPHONE  
AREA CODE 716 576-2700

March 8, 1978



50-244

Director of Nuclear Reactor Regulation  
ATTN: Mr. Victor Stello, Jr., Director  
Division of Operating Reactors  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Dear Mr. Stello:

Your January 25, 1978 letter, which was received by us on February 8, 1978, requested that Rochester Gas and Electric (RG&E) respond within 30 days indicating our intent to proceed with an evaluation of overall asymmetric loss of coolant accident (LOCA) loads.

A task group of utilities with Westinghouse plants, including RG&E, has been formed to examine primary system pipe breaks as outlined in your letter. The group is identifying similarities between Westinghouse plants in order to do consistent evaluations and, where possible, to do analyses that are applicable to more than one specific plant. The task group has met with Westinghouse and the NRC staff during the past several months to develop an evaluation program which is acceptable to both the task group utilities and the NRC staff. The purpose of the evaluation program is to reassess the original plant design and assure the safety of the task group plants should a pipe break occur at specific locations in the reactor coolant system. The evaluation will address those areas of concern identified in your letter. The task group is also working to develop an evaluation program schedule within 90 days as you requested.

Should you have any questions concerning the program, please contact us.

Sincerely yours,

*L.D. White, Jr.*  
L. D. White, Jr.

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