

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

W. L. STEWART  
VICE PRESIDENT  
NUCLEAR OPERATIONS

December 31, 1984

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
Attn: Mr. Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Serial No. 762  
E&C/TLG:jdm:2009N  
Docket Nos. 50-280  
50-281  
License Nos. DPR-32  
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY  
SURRY POWER STATION UNITS 1 & 2  
STATUS OF NUREG-0737, ITEM II.D.1.2

In our letter dated December 29, 1983 (Serial No. 721), we provided a status update of Surry Power Station's pressurizer safety and relief valve discharge piping evaluation. As stated, the initial preliminary analysis demonstrated the feasibility of maintaining the safety valve loop seals provided the loop seals are maintained at an elevated temperature of approximately 400°F and modifications are made to the piping system supports. We also noted that we were proceeding with the final engineering to accomplish the required modification and that our intentions were to complete the final engineering in 1984.

As you are aware, we have been proceeding with the final engineering to fulfill our commitment and to allow for the implementation of the required modifications. However, due to the coordination and volume of analysis and design engineering involved, in November 1984 we re-evaluated our position regarding completion of the final engineering by the end of 1984. On November 20, 1984 a telephone call was made to the NRC Surry Project Manager, Mr. J.D. Neighbors to discuss the status of our engineering and the potential of not completing all the final engineering packages. At that time we informed Mr. Neighbors that there may be some engineering which could carry over into the first quarter of 1985, and that by December 31, 1984 we would inform the NRC of the engineering status and of the revised completion date for any engineering not completed at that time.

The final engineering has been divided into two parts per unit. Each part will be controlled and worked as a Design Change Package (DCP). Part One for each unit is the "Pressurizer Safety Valve Loop Seal Insulation Ovens." The engineering for this portion consists of the engineering analysis and design for elevating and maintaining the pressurizer safety valve loop seals to a temperature of approximately 400°F and the preparation of the required DCPs.

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The DCPs have been prepared for both units, however, the vendor supplied insulation oven engineering design has not been included. Upon completion of the insulation engineering, it will be incorporated to complete the DCPs.

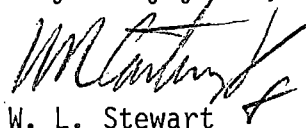
The Unit 2 engineering analysis has been performed and evaluated. The results have demonstrated that the oven design will maintain the bulk of the safety valve loop seal at or above 400°F. However, minor changes and refinements are required in finalizing the insulation design. Upon final design refinements, the insulation design will be incorporated into the DCP and the total DCP will be issued for review and approval. All review comments will be reviewed, addressed and incorporated as appropriate. Upon final resolution of the comments, the DCP will be forwarded to the Station Nuclear Safety and Operating Committee for final approval. This process is scheduled to be completed by March 31, 1985.

The Unit 1 engineering analysis is essentially identical to Unit 2's. Therefore, once the Unit 2 engineering is finalized, the Unit 1 engineering will proceed and upon completion will be incorporated into the Unit 1 DCP. Following review and final resolution of the comments, the DCP will be forwarded to the Station Nuclear Safety and Operating Committee for final approval. This process is also scheduled to be completed by March 31, 1985.

Part Two for each unit is the "Pressurizer Safety Valve Discharge Pipe Support Modifications." The engineering for this portion consists of the engineering analysis and design for the upgrade of the discharge piping supports.

The engineering analysis has been completed, the pipe supports designed, and the DCPs prepared. The final review and approval is presently being performed. All review comments will be reviewed, addressed and incorporated as appropriate. Upon final resolution of the comments, the DCPs will be forwarded to the Station Nuclear Safety and Operating Committee for final approval. This process is scheduled to be completed by March 31, 1985.

Very truly yours,



W. L. Stewart

cc: Mr. James P. O'Reilly  
Regional Administrator  
Region II  
Atlanta, Georgia 30323

Mr. Donald J. Burke  
NRC Resident Inspector  
Surry Power Station