VIRGINIA ELECTRIC AND POWER COMPANY

REVISIONS TO
SURRY POWER STATION
EMERGENCY PLAN IMPLEMENTING PROCEDURES

Enclosed are recently revised pages to the Surry Power Station Emergency Plan Implementing Procedures. Please take the following actions in order to keep your manual updated with the most recent revisions.

EPIP-1.01 EMERGENCY MANAGER CONTROLLING PROCEDURE

Remove and Destroy	Enter
Page 1 of 8, Rev. 09 and page 6 of 8, Rev. 08; Pages 6 of 39, 7 of 39, 8 of 39 and 9 of 39 of Attachment 1, all Rev. 08.	Pages 1 of 8 and 6 of 8; Pages 6 of 39, 7 of 39, 8 of 39 and 9 of 39 of Attachment 1 all Rev. 10. Dated 02-02-84

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A045

NO. 97887230
REV. A

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION

EMERGENCY PLAN IMPLEMENTING PROCEDURE

NUMBER	PROCEDURE TITLE	REVISION
EPIP-1.01	EMERGENCY MANAGER CONTROLLING PROCEDURE (With 1 Attachment)	10 PAGE 1 of 8

PURPOSE

To initially assess a potential emergency condition and initiate corrective actions.

USER

Shift Supervisor or Station Emergency Manager

ENTRY CONDITIONS

Any one of the following :

1. Another station procedure directs initiation of this procedure

OR

2. A potential emergency condition is reported to the Shift Supervisor.

REVISION RECORD

REV. 05	PAGE(S): 1 of 8, Att. 1 page 4, 22, and 23 of 39	DATE:03-10-83
REV. 06	PAGE(S): 1 of 8 and 4 of 8	DATE:03-11-83
REV. 07	PAGE(S): 1 0f 8, Att. 1 page 13 of 39	DATE:03-17-83
REV. 08	PAGE(S): 1, 5, 6 & 8; Att. 1, pgs. 2 through 13, 15, 17, 18, 20, and 23 through 38	DATE:11-10-83
REV. 09	PAGE(S): 1 of 8 and 5 of 8	DATE:01-13-84
REV. 10	PAGE(S): 1, 6; Att. 1, pgs. 6, 7, 8 and 9	DATE: FEB 0 2 1984

APPROVAL RECOMMENDED	APPROVED	DATE
QC REVIEW	CHAIRMAN STATION NUCLEAR SAFETY AND OPERATING COMMITTEE	FEB 2 1984

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
5.	(CONTINUED)	
	b) IF required, assign appropriate individual as interim Emergency Communicator	
	c) IF required, direct senior H.P. personnel on site to initiate EPIP-4.01, Radiological Assessment Director Controlling Procedure	
6.	DETERMINE EPIPS:	
	a) Event classification - NOTIFICATION OF UNUSUAL EVENT	a) GO TO Step 6b of this instruction.
	1) GO TO EPIP-1.02, Response to Notification of Unusual Event	
	b) Event classification - ALERT	b) GO TO Step 6c of this instruction.
	1) GO TO EPIP-1.03, Response to Alert	
	c) Event classification - SITE AREA EMERGENCY	c) GO TO Step 6d of this instruction.
	1) GO TO EPIP-1.04, Response to Site Area Emergency	
	d) Event classification - GENERAL EMERGENCY	
	1) GO TO EPIP-1.05, Response to General Emergency	
7.	SECURE FROM EVENT:	
	a) Notify involved station personnel that emergency condition does not exist	

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB B) REACTOR COOLANT SYSTEM EVENT	10
ATTACHMENT 1		PAGE 6 of 39

	CONDITION/ APPLICABILITY	INDICATION	CLASSIFICATION
1.	Safety Limit-RCS Temperature/Pressure curve exceeded POWER & HSB	Limits of T.S. Fig. 2.1-1 - EXCEEDED	Notification of Unusual Event
2.	RCS overpressure ALL CONDITIONS	2735 psig RCS Pressure limit-EXCEEDED	Notification of Unusual Event
3.	Rx overpower POWER	118% Rx thermal power limit - EXCEEDED	Notification of Unusual Event
4.	RCS leak rate exceeds 300 gpm ABOVE CSD CONDITION	EP-2.00, Loss of Reactor Coolant - IMPLEMENTED AND Pressurizer level can not be maintained with two (2) Charging/SI Pumps in operation	Site Area Emergency
5.	RCS leak rate exceeds 50 gpm ABOVE CSD CONDITION	Unit in HSD or lower condition as a result of actions required IAW AP-16, Excessive Primary Plant Leakage AND RCS inventory balance indicates leakage-GREATER THAN 50 gpm	Alert

	CONDITION/ APPLICABILITY	INDICATION	CLASSIFICATION
6.	RCS leak rate requiring plant shutdown IAW T.S. 3.1.C POWER & HSB	Unit in HSD or lower as a result of any of the following: a) Unidentified RCS leakage-GREATER THAN 1 gpm OR b) Identified leakage-GREATER THAN 10 gpm OR c) Non-isolable fault of RCS pressure boundary	Notification of Unusual Event
7.	RCP locked rotor leading to fuel damage POWER	All the following exist: a) Flow in one or more RC loops - LESS THAN 90% AND b) RCP trip caused by Phase Overcurrent Relay - ACTUATION AND c) High Range Letdown Radiation Monitor indication increases to-GREATER THAN 5×10^5 cpm	Alert

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB B) REACTOR COOLANT SYSTEM EVENT	10
ATTACHMENT 1		PAGE 8 of 39

	CONDITION/ APPLICABILITY	INDICATION	CLASSIFICATION
8.	Gross Primary to Secondary leakage with loss of offsite power ABOVE CSD CONDITION	EP-4.00, Steam Generator Tube Rupture - IMPLEMENTED with SI in progress AND Condenser Air Ejector Exhaust - DIVERT to Containment AND Loss of offsite power indicated by zero volts on voltmeters for 4160V Buses D, E & F	Site Area Emergency
9.	Excessive Primary to Secondary leakage with loss of offsite power ABOVE CSD CONDITION	Unit in HSD or lower condition as a result of actions required by T.S. 3.1.C.6 AND Loss of offsite power indicated by zero volts on voltmeters for 4160V buses D, E & F.	Alert

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB B) REACTOR COOLANT SYSTEM EVENT	10
ATTACHMENT 1		PAGE 9 of 39

	CONDITION/ APPLICABILITY	INDICATION	CLASSIFICATION
10.	Gross Primary to Secondary leakage ABOVE CSD CONDITION	EP-4.00, Steam Generator Tube Rupture - IMPLEMENTED with SI in progress AND Condenser Air Ejector Exhaust - DIVERT to Containment	Alert
11.	Primary to Secondary leakage-GREATER THAN 1 gpm ABOVE CSD CONDITION	Unit in HSD or lower condition as a result of actions required by T.S. 3.1.C.6	Notification of Unusual Event
12.	Loss of 2 of 3 fission product barriers with potential loss of 3rd barrier ALL CONDITIONS	Any two of a),b) or c) exist and the third is imminent a) Fuel clad integrity failure as indicated by any of the following: 1) RCS specific activity - GREATER THAN OR EQUAL TO 300.0 uCi/gram dose equivalent I-131. 2) 5 or more core exit thermocouples reading-GREATER THAN 1200°F OR b) Loss of RCS integrity as indicated by any of the following:	General Emergency

GOV 02-54

VIRGINIA ELECTRIC AND POWER COMPANY

RICHMOND, VIRGINIA 23261

February 17, 1984

Mr. James P. O'Reilly
Regional Administrator
U.S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. 081
NO/REB/vgv:EM1-17
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Dear Mr. O'Reilly:

REVISIONS TO
SURRY POWER STATION
EMERGENCY PLAN IMPLEMENTING PROCEDURES

Pursuant to 10CFR 50, Appendix E, paragraph 50.54 (q), revisions to the Surry Power Station Emergency Plan Implementing Procedures, numbers, and subjects as listed on enclosures are submitted.

Sincerely,

W. L. Stewart

Enclosures

cc: Document Control Desk (2)

w/o Enclosures

cc: Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1

Mr. D. J. Burke - (NRC - Surry)

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