12. A spent fuel cask or heavy loads exceeding 110 percent of the weight of a fuel assembly (not including fuel handling tool) shall not be moved over spent fuel, and only one spent fuel assembly will be handled at one time over the reactor or the spent fuel pit.

13. A spent fuel cask shall not be moved into the Fuel Building unless the Cask Impact Pads are in place on the bottom of the spent fuel pool.

14. Two trains of the control and relay room emergency ventilation system shall be operable. With one train inoperable for any reason, demonstrate the other train is operable by performing the test in Specification 4.20.A.1. With both trains inoperable comply with Specification 3.10.B. *

15. Containment purge shall be filtered through the high efficiency particulate air filters and charcoal absorbers.

B. If any one of the specified limiting conditions for refueling is not met, refueling of the reactor shall cease, work shall be initiated to correct the conditions so that the specified limit is met, and no operations which increase the reactivity of the core shall be made.

C. After initial fuel loading and after each core refueling operation and prior to reactor operation at greater than 75% of rated power, the movable incore detector system shall be utilized to verify proper power distribution.

D. The requirements of 3.0.1 are not applicable. *

* Pending NRC approval

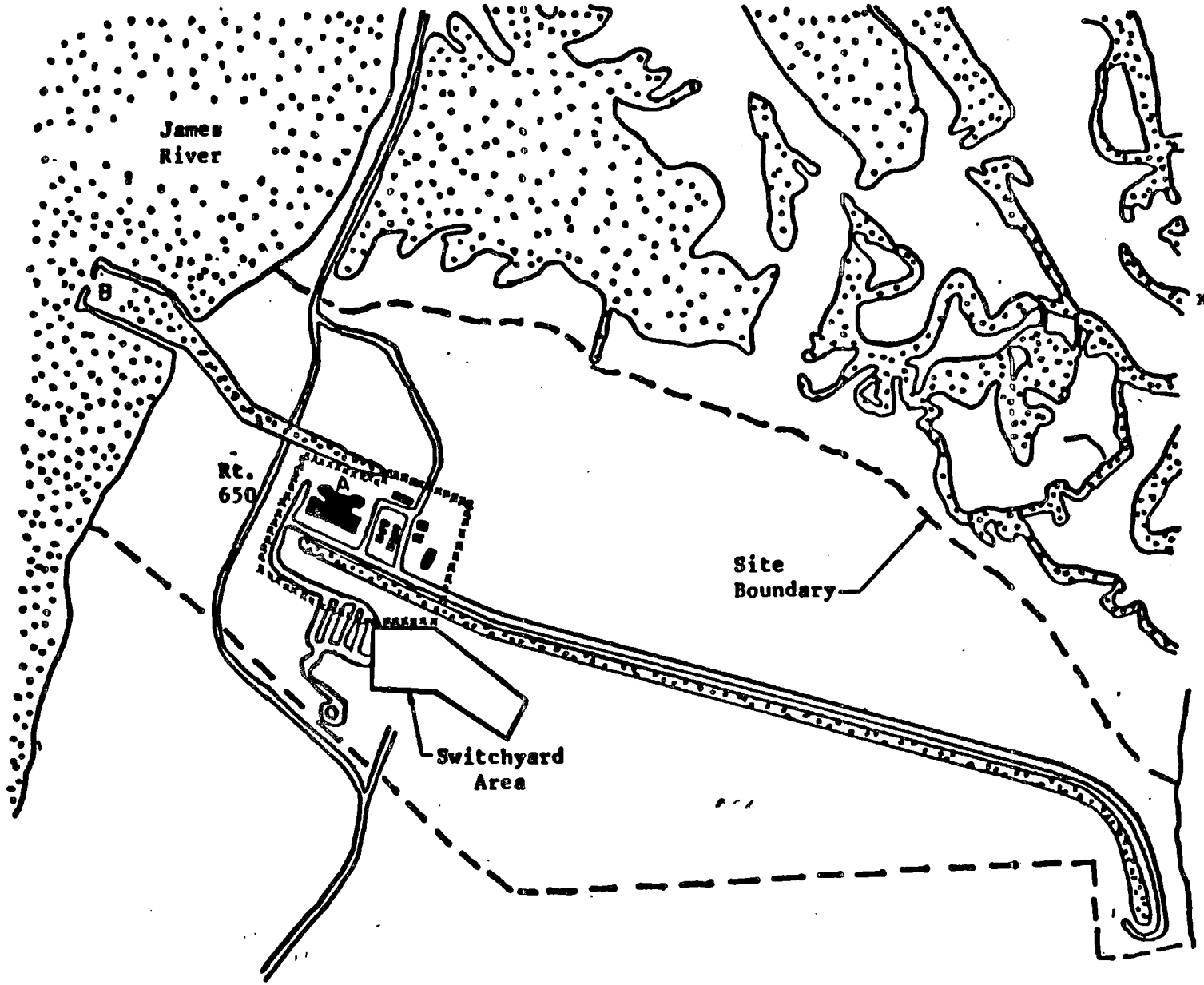
E. The dose rate due to noble gases in gaseous effluents shall be determined continuously to be within the limits of Specification 3.11.B.1 in accordance with the methods and procedures of the ODCM.

The dose rate due to Iodine-131, Tritium, and all radionuclides in particulate form with half life greater than 8 days, in gaseous effluents shall be determined to be within the limits of Specification 3.11.B.1 in accordance with the ODCM by obtaining representative samples and performing analyses in accordance with the sampling and analysis program specified in Table 4.9-2.

F. The concentration of hydrogen or oxygen in the waste gas holdup system shall be determined to be within the limits of Specification 3.11.B.5 by continuously monitoring the waste gases in the waste gas holdup system with the hydrogen or oxygen monitors required operable by Table 3.7-5(b) of Specification 3.7.E.

G. The quantity of radioactive material contained in each gas storage tank shall be determined to be within the limits of Specification 3.11.B.6 at least once per month when the specific activity of the primary reactor coolant is ≤ 2200 μCi/gm dose equivalent Xe-133. Under the conditions which result in a specific activity > 2200 μCi/gm dose equivalent Xe-133, the Waste Gas Decay Tanks shall be sampled once per day.

H. The radiological environmental monitoring samples shall be collected pursuant to Table 4.9-3 from the specific locations given in the table and figure(s) in the ODCM and shall be analyzed pursuant to the requirements of Table 4.9-3, the detection capabilities required by Table 4.9-5.



A Gaseous Release
1. Process Vent- 131 ft.
2. Vent-Vent Stacks-considered ground level release

B Liquid leaves site

xxx Security Fence- Area outside is unrestricted for gaseous effluents

Land Maximum Individual Occupancy within site boundary:
1) Canal Bank Fishing= 160 hr/yr

Liquid Maximum Individual Occupancy within site boundary:
1) Boat Fishing Discharge Canal= 800 hr/yr

0 ½mile

Scale

Figure 5.1-1

Map Defining Unrestricted Areas for Radioactive Gaseous and Liquid Effluents

TS Figure 5.1-1

ATTACHMENT 2

SAFETY EVALUATION

These proposed changes supplement the Radiological Effluent Technical Specifications (RETS) submittal. Added by this proposed change is a Specification concerning filtering during containment purge. This Specification is in the currently approved Surry Technical Specifications, but was inadvertently removed from the Technical Specifications in the RETS submittal. Surveillance requirements of the Waste Gas Decay Tanks are added to Specification 4.9.G. These requirements are a part of Justification No. 15 of the RETS submittal but were mistakenly left out of Specification 4.9.G. Figure 5.1-1 is revised for clarity and a scale is added.

These proposed changes are administrative in nature. These proposed changes do not pose a significant hazards consideration or any danger to the safe operations of the plant and/or public safety.

m12/008