

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

W. L. STEWART
VICE PRESIDENT
NUCLEAR OPERATIONS

April 14, 1983

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Serial No. 001B
NO/WDC:acm
Docket No. 50-280
License No. DPR-32

Gentlemen:

SUPPLEMENT TO RELIEF REQUEST FROM ASME XI REQUIREMENT
SURRY POWER STATION UNIT NO. 1 10 YEAR INSERVICE INSPECTION

Surry Power Station Unit 1 is currently performing its final inservice inspections for its first 10 year interval as required by ASME Section XI, 1974 Edition, Summer 1975 addenda (first 10 year interval governing code). Pursuant to 10 CFR 50.55a paragraph g(5) and as a supplement to our letter dated January 7, 1983 (Serial No. 001), relief is requested from certain first interval, third period inspection requirements delineated in ASME Section XI. The following basis is provided.

During the Surry Unit 1 steam generator replacement (9/14/80 - 7/6/81 inclusive) extensive preservice examinations were conducted to the requirements of IWB-2100 (74-S75 Code). Specific identification of components affected and the dates of the preservice examinations are provided on Enclosure 1. Due to the close proximity of the preservice examinations to the end of the third period, it is considered improbable that significant or relevant indications would be identified with an additional inspection in the third period.

Alternatively hydrostatic testing in accordance with IWB-5222 (77-S79 Code) is presently planned to fulfill the first interval requirements. All Class I components shall be visually examined under hydrostatic test pressure. In addition Surry Unit 1 has currently adopted the 1980 Edition, Winter 1980 addenda version of ASME Section XI. Inspections required by this code as supplemented by the Station's submitted relief requests will be conducted for the first period of the second interval.

A047

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PDR ADOCK 05000280
PDR

VIRGINIA ELECTRIC AND POWER COMPANY TO Harold R. Denton

It is our contention that approval of this relief request will not compromise the intended surveillance requirements of the code, but allow continued inspection under the rules of the 1980 Edition of ASME Section XI. This relief would eliminate the impractical duplication of simultaneous inspection under two different codes.

Very truly yours,

W. L. Stewart
W. L. Stewart

Enclosure

cc: Mr. James P. O'Reilly
Regional Administrator
Region II

Mr. D. J. Burke
NRC Resident Inspector
Surry Power Station

Mr. J. Don Neighbors
NRC Project Manager - Surry
Operating Reactors Branch No. 1
Division of Licensing

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LOSURE 1

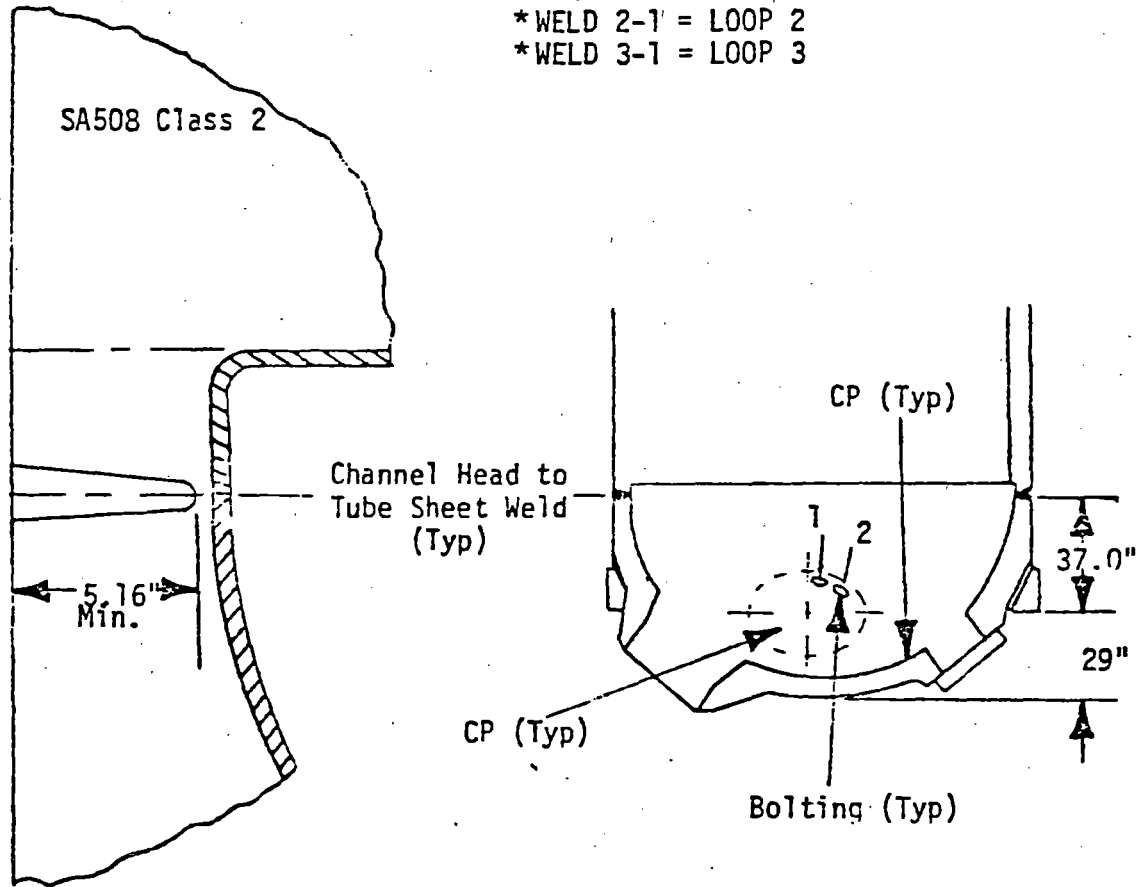
CATEGORY	ITEM	COMPONENT IDENTIFICATION	STATION ISO (ENCL.)	WELD NUMBER (IF APPLICABLE)	PRESERVIVE AND DATE	REQUIREMENT 3RD PERIOD 1ST INTERVAL	PLANNED 1ST PERIOD 2ND INTERVAL
B-F	B3.3	'B' Steam generator safe end welds	VPA-1-4200	05DM	Vol - 4/9/81 Sur.- 4/9/81 VT - 4/9/81	Volumetric Surface	Volumetric Surface (Code percentage)
				06DM	Vol - 5/13/81 Sur.- 5/13/81 VT. - 5/13/81	Volumetric Surface	Volumetric Surface (Code percentage)
B-I-2	B3.8	Clad Surfaces on all three steam generators	VPA-1-3100	'A' CP-1 CP-2	VT - 4/2/81 VT - 4/2/81	Visual	Visual (Code percentage)
				'B' CP-3 CP-4	VT - 4/2/81 VT - 4/2/81	Visual	Visual (Code percentage)
				'C' CP-5 CP-6	VT - 4/7/81 VT - 4/7/81	Visual	Visual (Code percentage)
B-J	B4.8	Socket welds on charging, reactor coolant, and safety injection systems	N/A	N/A	46 preservice surface examinations conducted (SGRP)	Requires 4 additional welds to those already inspected (Surface)	Surface (Code percentage)

ILLUSTRATIVE ONLY

VPA-1-3100
1980

STEAM GENERATOR

- * WELD 1-1 = LOOP 1
- * WELD 2-1 = LOOP 2
- * WELD 3-1 = LOOP 3



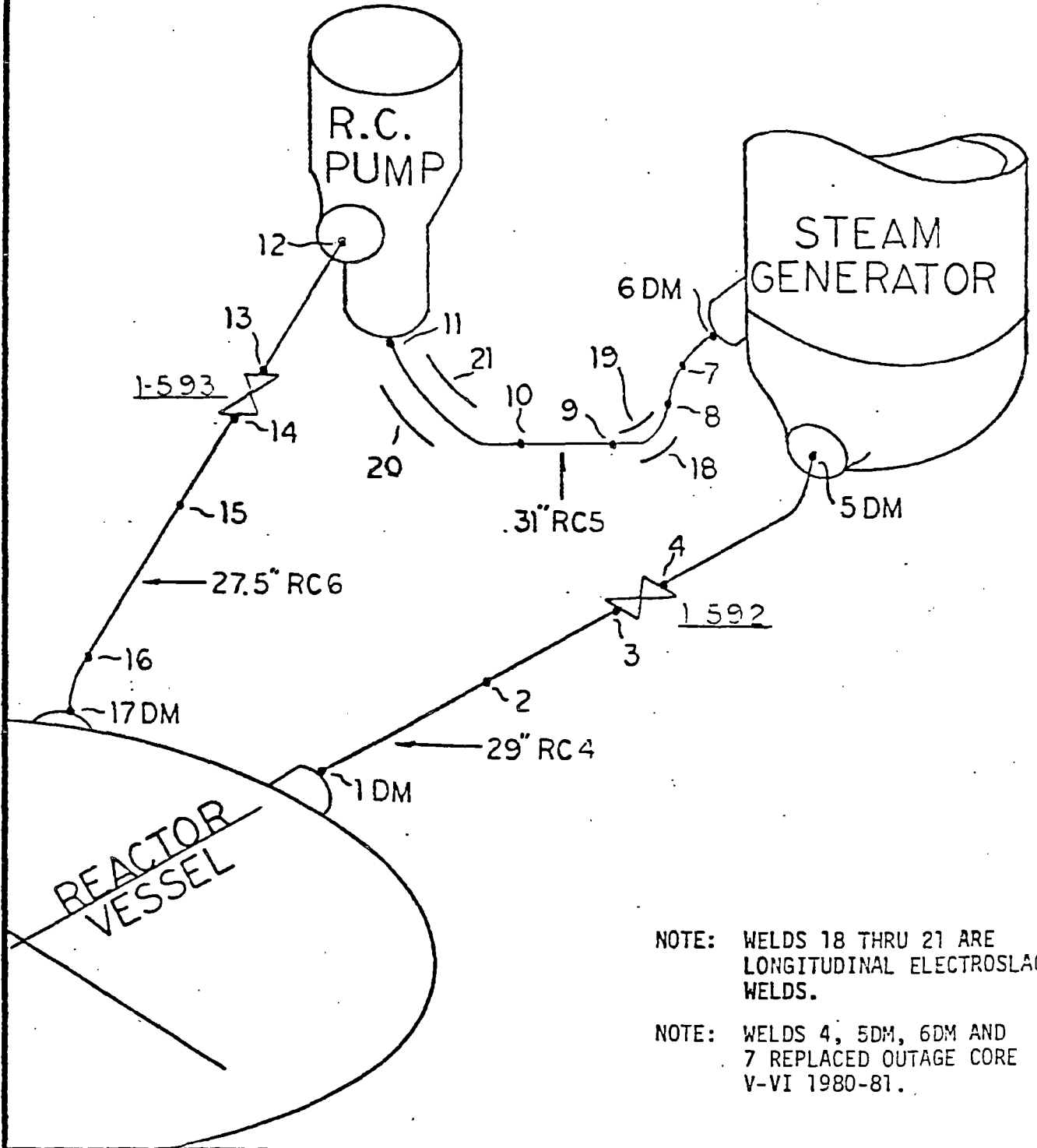
	<u>S.G.</u>	<u>Manway</u>	<u>Bolting</u>	<u>CP#</u>
(1-RC-E-1A)	1	Hotside	A1 to A16	1
(1-RC-E-1A)	1	Coldside	A17 to A32	2
(1-RC-E-1B)	2	Hotside	B1 to B16	3
(1-RC-E-1B)	2	Coldside	B17 to B-32	4
(1-RC-E-1C)	3	Hotside	C1 to C-16	5
(1-RC-E-1C)	3	Coldside	C-17 to C-32	6

* Replacement Weld installed Outage Core V-VI 1980-81.

VPA-1-4200
1980

LOOP 2 REACTOR COOLANT PIPE

FORM 462



NOTE: WELDS 18 THRU 21 ARE LONGITUDINAL ELECTROSLAG WELDS.

NOTE: WELDS 4, 5DM, 6DM AND 7 REPLACED OUTAGE CORE V-VI 1980-81.

VPA-4100
1980

LOOP 1 REACTOR COOLANT PIPE

FORM 4644f

