

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

W. L. STEWART  
VICE PRESIDENT  
NUCLEAR OPERATIONS

March 2, 1983

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
Attn: Mr. Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Serial No. 085A  
NO/ERS, Jr.:acm  
Docket Nos. 50-280  
50-281  
License Nos. DPR-32  
DPR-37

Gentlemen:

TECHNICAL EVALUATION REPORTS  
ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT  
SURRY POWER STATION UNITS 1 AND 2

Your letter of January 26, 1983 requested additional information concerning equipment qualification. In response to that letter, we supplied justification for continued operation in our letter of February 23, 1983. This letter supplements our previous letter with additional information concerning reactor coolant system wide range pressure.

This parameter serves as an input to the subcooling meter. This device was installed in response to NUREG-0737 and provides information related to the margin of subcooling in the reactor coolant system. This device does not perform automatic actuations of safety equipment. Before this device was installed, operations personnel used pressure and temperature of the reactor coolant system to ascertain the thermodynamic condition of the reactor coolant.

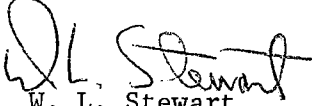
Currently installed instrumentation outside the containment with remote indication in the control room and with a range of 0 to 3000 psig could be used to infer the reactor coolant system margin to saturation. One pressure transmitter is located near the boron injection tank and the other is located on the charging/high head safety injection pump discharge header. Within the capacity/head curve for this pump, these pressure indications less friction losses in the piping represent the reactor coolant system pressure. With the pumps at runout flow, the coolant system integrity has degraded to a point where margin to subcooling cannot be maintained by the charging pumps. Other safety systems including accumulators and low head safety injection pumps are operating to cool the reactor core and repressurizing the reactor coolant system is not likely to occur.

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For Surry Unit 1, qualified equipment is being installed during the current outage. For Surry Unit 2, qualified equipment will be installed during the outage planned to begin on May 1, 1983. The probability of an event occurring prior to having a fully qualified indication of wide range pressure available is very low for Surry Unit 2. Alternate means of assessing reactor coolant system pressure exist. Continued operation of Surry Unit 2 is justified.

Very truly yours,

  
W. L. Stewart

Attachment 1: Unit 1 Response  
Attachment 2: Unit 2 Response

cc: Mr. James P. O'Reilly  
Regional Administrator  
Region II