VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

R. H. LEASBURG
VICE PRESIDENT
NUCLEAR OPERATIONS

July 1, 1982

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. Darrell G. Eisenhut, Director
Division of Licensing Commission
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Serial No. 392
NO/DWS,JR.:acm
PSEC/DPB
Docket Nos. 50-338
50-339
50-280
50-281
License Nos. NPF-4
NPF-7
DPR-32
DPR-37

Gentlemen:

RESPONSE TO NUREG 0737 POST-TMI REQUIREMENT-ITEM II.D.1
RELIEF, SAFETY, BLOCK VALVE TEST AND DISCHARGE PIPING ANALYSIS REQUIREMENTS
PLANT SPECIFIC REPORT
NORTH ANNA 1 & 2 AND SURRY 1 & 2

In accordance with the requirements of Item II.D.1 of NUREG 0737 and North Anna 2 License Condition 2.C.21(f) attached are the July 1, 1982 plant specific submittals for the safety and relief valve testing for North Anna 1 and 2 and Surry 1 and 2. Block valve, plant specific information, also scheduled for July 1, 1982 is not available at this time and will be provided as soon as possible but not later than September 1, 1982.

Plant specific pressurizer discharge piping and support evaluation scheduled for July 1, 1982 is incomplete.

We are currently performing two alternate analyses of the North Anna Unit 2 system. The first approach is to analyze the pressurizer safety valve piping with the loop seals drained. This analysis has now been completed and we are evaluating the results.

Due to potential operability problems posed by draining the loop seals, a second analysis has been undertaken in which the loop seals are retained. Currently, generation of time history thermal hydraulic loads and development of structural modeling are underway based on the North Anna Unit 2 piping configuration. Detailed analysis will commence upon completion of these efforts. We presently anticipate this analysis to be completed in November, 1982. In conjunction with this analysis, a review of both North Anna Units 1 and 2, and Surry Units 1 and 2 is being performed to determine the unit with the worst case loadings for the dynamic analysis based on retention of the loop seals. This information will be utilized to envelope the remaining three units in the event that the North Anna Unit 2 analysis indicates the feasibility of retaining the loop seals on North Anna Unit 1, and Surry Units 1 and 2.

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Based on our evaluation of the results of the two analyses outlined above, we anticipate that a course of action will be established for North Anna Unit 2 by January 1, 1983. The outcome of these analyses will also be reviewed for application to North Anna Unit 1, and Surry Units 1 and 2 by January 1, 1983.

Also at this time we are reviewing with Westinghouse, Crosby, and Dresser the necessity of making ring adjustments to the safety valves based on the EPRI test results and our plant specific inlet piping arrangements. At the present, our consultant, Westinghouse Electric Corporation advises us there is no evidence that changes in ring settings must be made; however, if their review indicates adjustments are required, VEPCO will advise NRC of the magnitude of the changes and provide a schedule for making adjustments.

Very truly yours,

R. H. Leasburg

cc: Mr. James P. O'Reilly
Regional Administrator
Office of Inspection and Enforcement
Region II

Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Mr. Robert A. Clark, Chief
Operating Reactors Branch No. 3
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