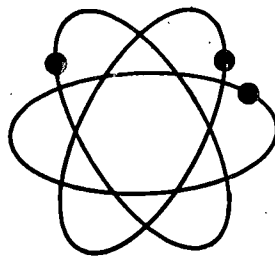


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SURRY UNIT 2, CYCLE 5 CORE PERFORMANCE REPORT



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FUEL RESOURCES DEPARTMENT
Virginia Electric and Power Company

SURRY UNIT 2, CYCLE 5
CORE PERFORMANCE REPORT

BY

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December, 1981

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Section 1

INTRODUCTION AND SUMMARY

On November 7, 1981, after almost fifteen months of operation, Surry Unit 2 completed Cycle 5. Since the initial criticality of Cycle 5 on August 14, 1980, the reactor core produced approximately 82×10^6 MBTU (13,971 Megawatt days per metric ton of contained uranium) which has resulted in the generation of approximately 7.8×10^9 KWHr gross (7.4×10^9 KWHr net) of electrical energy. The purpose of this report is to present an analysis of the core performance for routine operation during Cycle 5. The physics tests that were performed during the startup of this cycle were covered in the Surry 2, Cycle 5 Startup Physics Test Report¹ and, therefore, will not be included here.

The fifth cycle core consisted of six batches of fuel. Two once-burned batches were brought from Cycle 4 (Batches 6A1 and 6B). One twice-burned batch was carried over from Cycles 3 and 4 (Batch 5A). One thrice-burned batch was carried over from Cycles 2, 3, and 4 (Batch 4B2). Two fresh batches (Batches 7A and 7B) were added to the Cycle 5 core. The Surry 2, Cycle 5 core loading map specifying the fuel batch identification, fuel assembly locations, burnable poison locations and source assembly locations is shown in Figure 1.1. Movable detector locations and thermocouple locations are identified in Figure 1.2. Control rod locations are shown in Figure 1.3.

Routine core follow involves the analysis of four principal performance indicators. These are burnup distribution, reactivity

depletion, power distribution, and primary coolant activity. The core burnup distribution is followed to verify both burnup symmetry and proper batch burnup sharing, thereby, ensuring that the fuel held over for the next cycle will be compatible with the new fuel that is inserted. Reactivity depletion is monitored to detect the existence of any abnormal reactivity behavior, to determine if the core is depleting as designed, and to indicate at what burnup level refueling will be required. Core power distribution follow includes the monitoring of nuclear hot channel factors to verify that they are within the Technical Specifications² limits thereby ensuring that adequate margins to linear power density and critical heat flux thermal limits are maintained. Lastly, as part of normal core follow, the primary coolant activity is monitored to verify that the dose equivalent iodine-131 concentration is within the limits specified by the Surry Power Station Technical Specifications, and to assess the integrity of the fuel.

Each of the four performance indicators is discussed in detail for the Surry 2, Cycle 5 core in the body of this report. The results are summarized below:

1. Burnup Follow - The burnup tilt (deviation from quadrant symmetry) on the core was no greater than $\pm 0.14\%$ with the burnup accumulation in each batch deviating from design prediction by less than $\pm 0.6\%$.

2. Reactivity Depletion Follow - The critical boron concentration, used to monitor reactivity depletion, was consistently within $\pm 0.5\%$ $\Delta K/K$ of the design prediction which is well within the $\pm 1\%$ $\Delta K/K$

K/K margin allowed by Section 4.10 of the Technical Specifications.

3. Power Distribution Follow - Incore flux maps taken each month indicated that the assemblywise radial power distributions deviated from the design predictions by an average difference of less than 1.4%. All hot channel factors met their respective Technical Specifications limits.

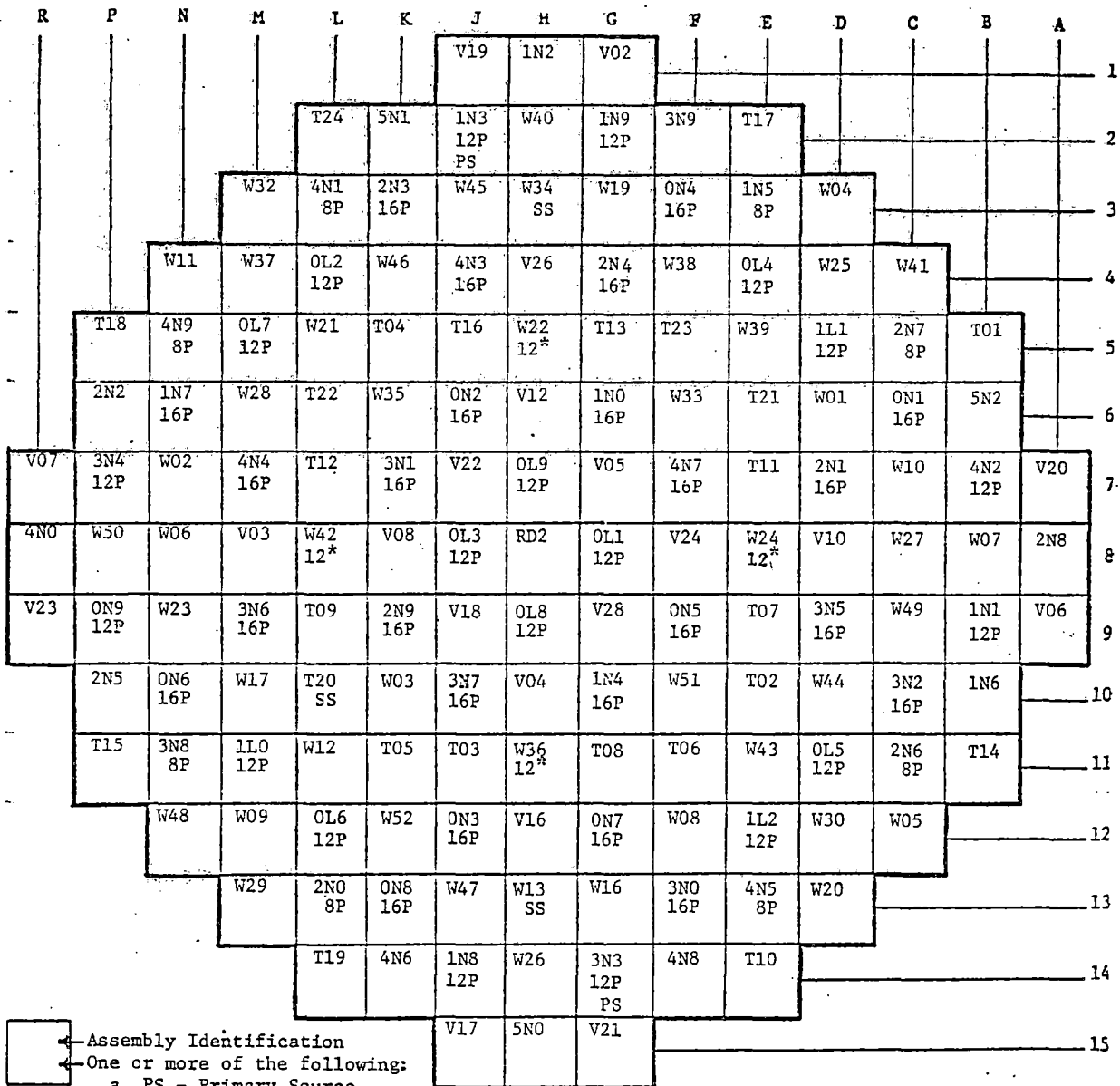
4. Primary Coolant Activity Follow - The dose equivalent iodine-131 activity level in the primary coolant at the end of Cycle 5 was approximately 5.4×10^{-3} micro-Ci/gm. The average dose equivalent iodine-131 value during Cycle 5 was 6.2×10^{-3} micro-Ci/gm. This corresponds to less than 1% of the operating limit for the concentration of radioiodine in the primary coolant.

In addition, the effects of fuel densification were monitored throughout the cycle. No densification effects were observed.

Figure 1.1

SURRY UNIT 2 - CYCLE 5

CORE LOADING



- ← Assembly Identification
- ← One or more of the following:
 - a. PS - Primary Source
 - b. SS - Secondary Source
 - c. xxP - Burnable Poison Assembly (xx-number of rods)
 - d. xx* - Depleted Burnable Poison Assembly (xx-number of rods)

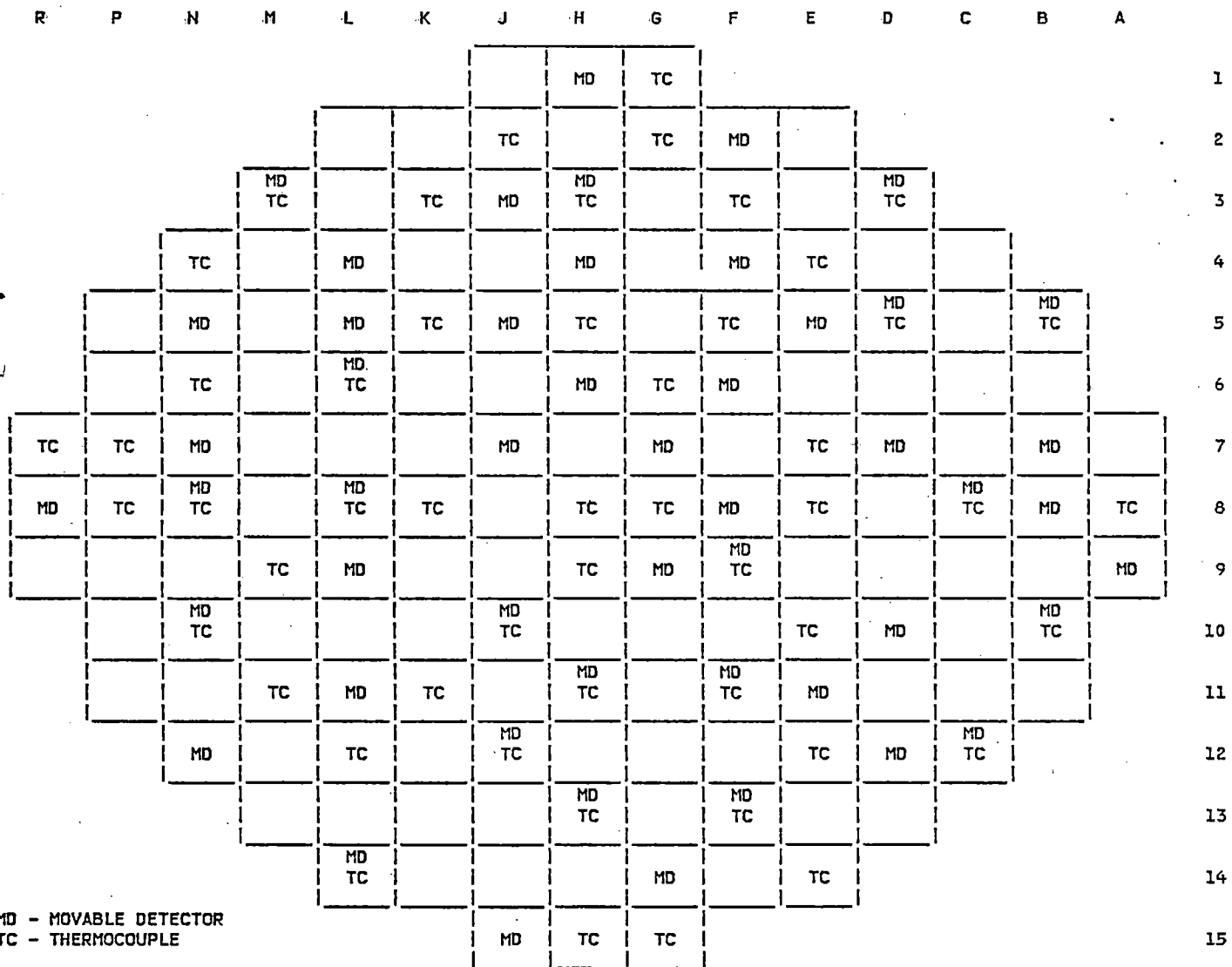
FUEL ASSEMBLY DESIGN PARAMETERS

| | MATCH | | | | | |
|-------------------------------|-------|---------|--|---|--------------------|--|
| | 4B2 | 5A | 6A1 | 6B | 7A | 7B |
| Initial Enrichment (w/o U235) | 3.10 | 3.11 | 2.91 | 3.20 | 3.13 | 3.41 |
| Assembly Type | 17X17 | 15X15 | 15X15 | 15X15 | 15X15 | 15X15 |
| Number of Assemblies | 1 | 24 | 20 | 48 | 12 | 52 |
| Fuel Rods Per Assembly | 264 | 204 | 204 | 204 | 204 | 204 |
| Assembly Identification | RD2 | T01-T24 | V02-V08 V10, V12 V16-V24 V26, V28 | W01-W13 W16, W17 W19-W30 W32-W52 | 0L1-0L9 1L0-1L2 | 0N1-0N9 1N0-1N9 2N0-2N9 3N0-3N9 4N0-4N9 5N0-5N2 |

FIGURE 1.2

SURRY UNIT 2 - CYCLE 5

MOVEABLE DETECTOR AND THERMOCOUPLE LOCATIONS

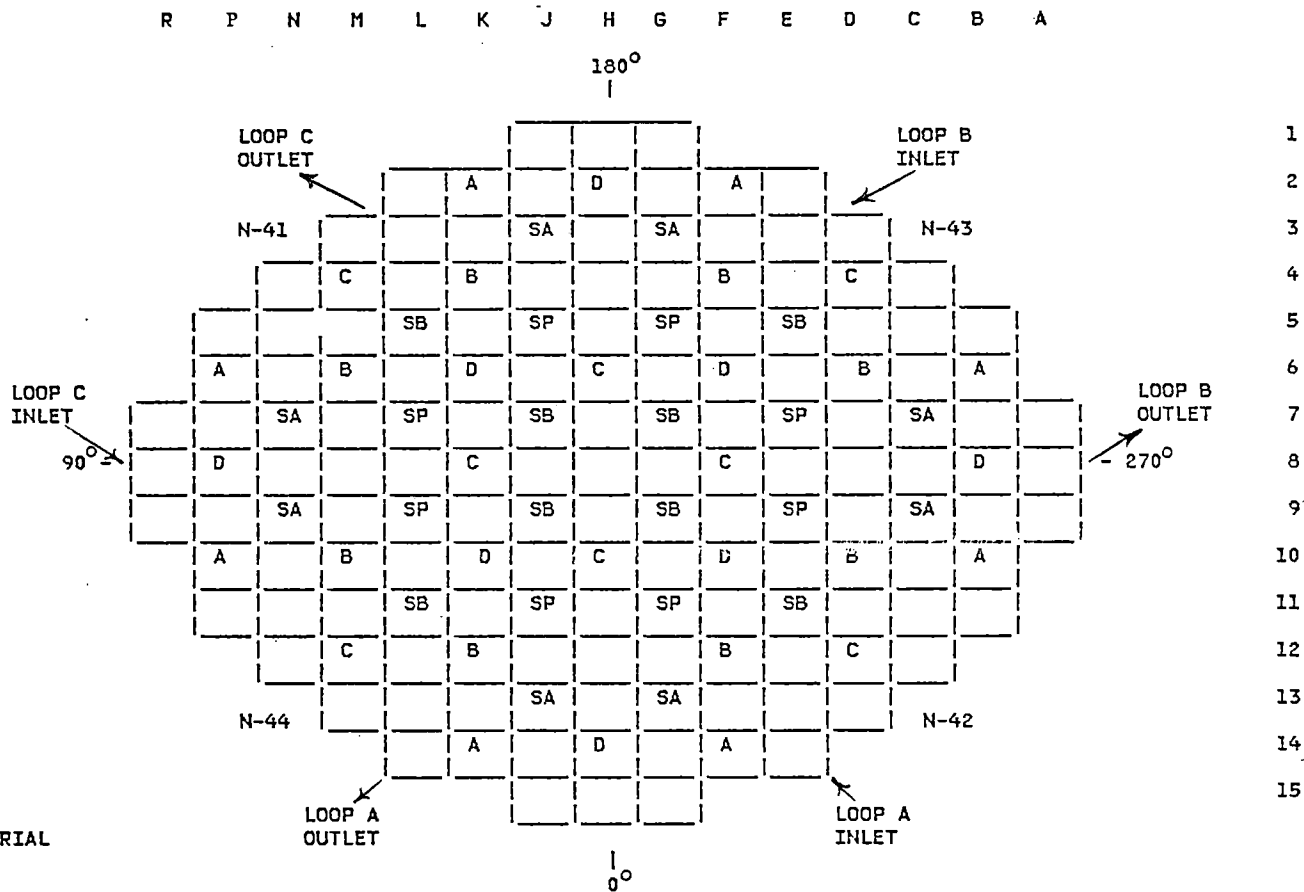


MD - MOVABLE DETECTOR
TC - THERMOCOUPLE

FIGURE 1.3

SURRY UNIT 2 - CYCLE 5

CONTROL ROD LOCATIONS



ABSORBER MATERIAL
AG-IN-CD

| FUNCTION | NUMBER OF CLUSTERS |
|--------------------------|--------------------|
| CONTROL BANK D | 8 |
| CONTROL BANK C | 8 |
| CONTROL BANK B | 8 |
| CONTROL BANK A | 8 |
| SHUTDOWN BANK SB | 8 |
| SHUTDOWN BANK SA | 8 |
| SP (SPARE ROD LOCATIONS) | 8 |

Section 2

BURNUP FOLLOW

The burnup history for the Surry Unit 2, Cycle 5 core is graphically depicted in Figure 2.1. The Surry 2, Cycle 5 core achieved a burnup of 13,971 MWD/MTU. As shown in Figure 2.2, the average load factor for Cycle 5 was 90% when referenced to rated thermal power (2441 MW(t)).

Radial (X-Y) burnup distribution maps show how the core burnup is shared among the various fuel assemblies, and thereby allow a detailed burnup distribution analysis. The NEWTOTE³ computer code is used to calculate these assemblywise burnups. Figure 2.3 is a radial burnup distribution map in which the assemblywise burnup accumulation of the core at the end of Cycle 5 operation is given. For comparison purposes, the design values are also given. Figure 2.4 is a radial burnup distribution map in which the percentage difference comparison of measured and predicted assemblywise burnup accumulation at the end of Cycle 5 operation is also given. As can be seen from this figure, the accumulated assembly burnups were generally within $\pm 3.0\%$ of the predicted values. In addition, deviation from quadrant symmetry in the core, as indicated by the burnup tilt factors, was less than $\pm 0.14\%$.

The burnup sharing on a batch basis is monitored to verify that the core is operating as designed and to enable accurate end-of-cycle batch burnup predictions to be made for use in reload fuel design studies. Batch definitions are given in Figure 1.1. As seen in Figure 2.5, the

batch burnup sharing for Surry Unit 2, Cycle 5 followed design predictions very closely with each batch deviating less than $\pm 0.6\%$ from design; this is considered excellent agreement. Therefore, symmetric burnup in conjunction with the good agreement between actual and predicted assemblywise burnups and batch burnup sharing indicate that the Cycle 5 core did deplete as designed.

FIGURE 2.1

SURRY UNIT 2 - CYCLE 5
CORE BURNUP HISTORY

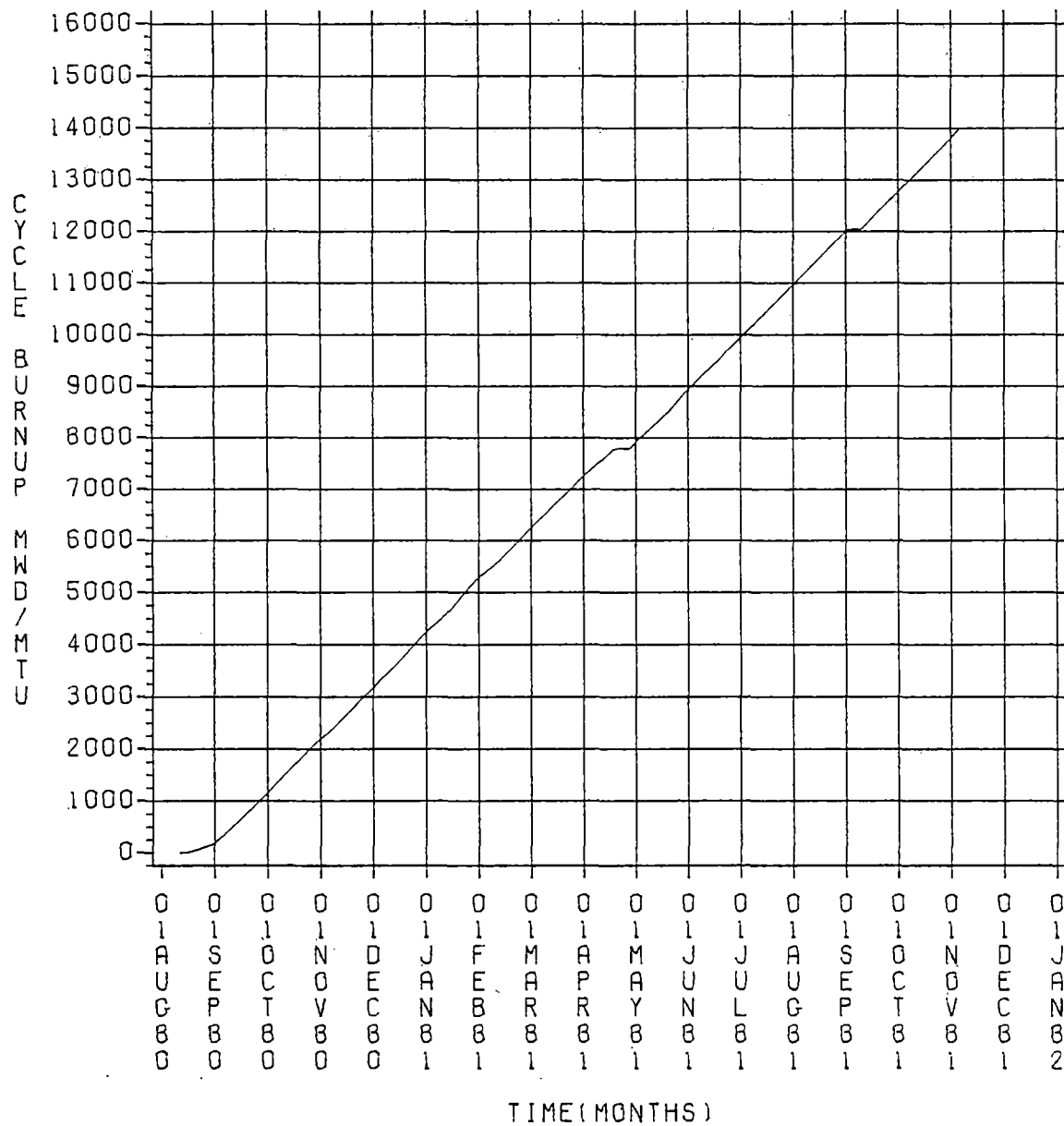
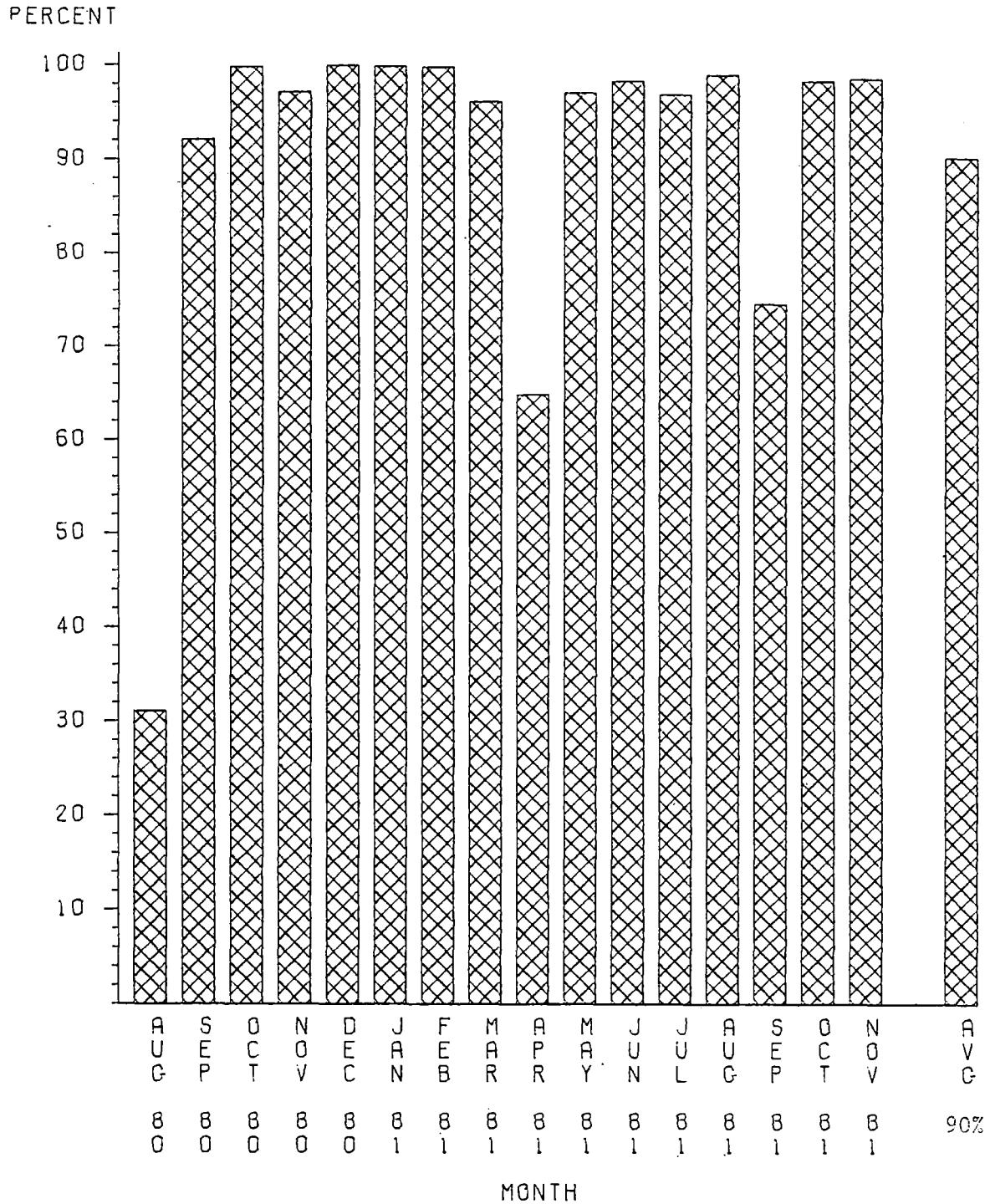


FIGURE 2.2

SURRY 2 - CYCLE 5
MONTHLY AVERAGE LOAD FACTORS



$$\text{LOAD FACTOR} = \frac{\text{THERMAL ENERGY GENERATION IN MONTH (MWHT)}}{\text{AUTHORIZED POWER LEVEL (MWT) X HOURS IN MONTH (EXCLUDES REFUELING OUTAGES)}}$$

FIGURE 2.3
 ASSEMBLYWISE ACCUMULATED BURNUP
 MEASURED AND PREDICTED
 (1000 MWD/MTU)

| | R | P | N | M | L | K | J | H | G | F | E | D | C | B | A | | |
|----|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-----------|----|
| 1 | | | | | | | 22.51 | 10.09 | 22.21 | | | | | | | MEASURED | 1 |
| | | | | | | | 22.36 | 10.20 | 22.36 | | | | | | | PREDICTED | |
| 2 | | | | | 28.36 | 12.96 | 14.51 | 23.23 | 14.48 | 12.92 | 28.27 | | | | | | 2 |
| | | | | | 28.23 | 12.92 | 14.65 | 23.58 | 14.65 | 12.92 | 28.23 | | | | | | |
| 3 | | | | 17.48 | 14.82 | 16.73 | 29.77 | 30.35 | 29.71 | 16.68 | 14.83 | 17.70 | | | | | 3 |
| | | | | 17.46 | 14.70 | 16.65 | 29.96 | 30.89 | 29.96 | 16.65 | 14.70 | 17.46 | | | | | |
| 4 | | | 17.59 | 28.11 | 16.38 | 29.85 | 16.99 | 30.59 | 16.92 | 29.92 | 16.48 | 28.23 | 17.71 | | | | 4 |
| | | | 17.46 | 28.17 | 16.49 | 30.11 | 17.09 | 30.78 | 17.09 | 30.11 | 16.49 | 28.17 | 17.46 | | | | |
| 5 | | 28.48 | 14.75 | 16.36 | 25.98 | 36.88 | 35.90 | 24.99 | 35.69 | 36.62 | 26.20 | 16.50 | 15.19 | 28.60 | | | 5 |
| | | 28.23 | 14.70 | 16.49 | 26.33 | 36.95 | 35.88 | 25.11 | 35.88 | 36.95 | 26.33 | 16.49 | 14.70 | 28.23 | | | |
| 6 | | 12.91 | 16.69 | 29.87 | 36.85 | 26.26 | 17.69 | 31.17 | 17.61 | 26.15 | 36.71 | 30.12 | 16.97 | 13.35 | | | 6 |
| | | 12.92 | 16.65 | 30.11 | 36.95 | 26.60 | 17.66 | 31.24 | 17.66 | 26.60 | 36.95 | 30.11 | 16.65 | 12.92 | | | |
| 7 | 22.43 | 14.58 | 29.71 | 16.97 | 35.80 | 17.59 | 31.56 | 17.65 | 31.59 | 17.62 | 35.38 | 17.00 | 29.96 | 14.75 | 22.45 | | 7 |
| | 22.36 | 14.65 | 29.96 | 17.09 | 35.88 | 17.66 | 31.45 | 17.56 | 31.45 | 17.66 | 35.88 | 17.09 | 29.96 | 14.65 | 22.36 | | |
| 8 | 10.12 | 23.30 | 30.72 | 30.64 | 24.71 | 31.16 | 17.75 | 42.51 | 17.65 | 31.16 | 24.70 | 30.62 | 30.72 | 23.39 | 10.43 | | 8 |
| | 10.20 | 23.58 | 30.89 | 30.78 | 25.11 | 31.24 | 17.56 | 42.27 | 17.56 | 31.24 | 25.11 | 30.78 | 30.89 | 23.58 | 10.20 | | |
| 9 | 22.44 | 14.61 | 29.90 | 17.03 | 35.98 | 17.44 | 31.26 | 17.63 | 31.64 | 17.44 | 35.48 | 17.00 | 29.82 | 14.86 | 22.81 | | 9 |
| | 22.36 | 14.65 | 29.96 | 17.09 | 35.88 | 17.66 | 31.45 | 17.56 | 31.45 | 17.66 | 35.88 | 17.09 | 29.96 | 14.65 | 22.36 | | |
| 10 | | 13.00 | 16.80 | 30.17 | 36.84 | 26.18 | 17.27 | 31.10 | 17.61 | 26.33 | 36.93 | 30.03 | 16.66 | 12.99 | | | 10 |
| | | 12.92 | 16.65 | 30.11 | 36.95 | 26.60 | 17.66 | 31.24 | 17.66 | 26.60 | 36.95 | 30.11 | 16.65 | 12.92 | | | |
| 11 | | 28.34 | 14.96 | 16.45 | 25.88 | 36.65 | 35.70 | 24.58 | 35.85 | 36.96 | 26.11 | 16.61 | 14.97 | 28.21 | | | 11 |
| | | 28.23 | 14.70 | 16.49 | 26.33 | 36.95 | 35.88 | 25.11 | 35.88 | 36.95 | 26.33 | 16.49 | 14.70 | 28.23 | | | |
| 12 | | | 17.75 | 28.40 | 16.18 | 29.68 | 16.72 | 30.38 | 16.87 | 30.25 | 16.59 | 28.37 | 17.64 | | | | 12 |
| | | | 17.46 | 28.17 | 16.49 | 30.11 | 17.09 | 30.78 | 17.09 | 30.11 | 16.49 | 28.17 | 17.46 | | | | |
| 13 | | | | 17.91 | 15.58 | 17.00 | 29.56 | 30.29 | 29.70 | 16.84 | 14.95 | 17.71 | | | | | 13 |
| | | | | 17.46 | 14.70 | 16.65 | 29.96 | 30.89 | 29.96 | 16.65 | 14.70 | 17.46 | | | | | |
| 14 | | | | | 28.61 | 13.63 | 14.70 | 23.30 | 14.61 | 12.99 | 28.45 | | | | | | 14 |
| | | | | | 28.23 | 12.92 | 14.65 | 23.58 | 14.65 | 12.92 | 28.23 | | | | | | |
| 15 | | | | | | | 22.69 | 10.41 | 22.16 | | | | | | | | 15 |
| | | | | | | | 22.36 | 10.20 | 22.36 | | | | | | | | |

R P N M L K J H G F E D C B A

SURRY UNIT 2-CYCLE 5
 BATCH BURNUP SHARING

SYMBOLIC POINTS ARE MEASURED DATA

BATCH : 4B2 5A 6A1 6B 7A 7B
 SYMBOL: DIAMOND STAR PLUS SQUARE TRIANGLE X

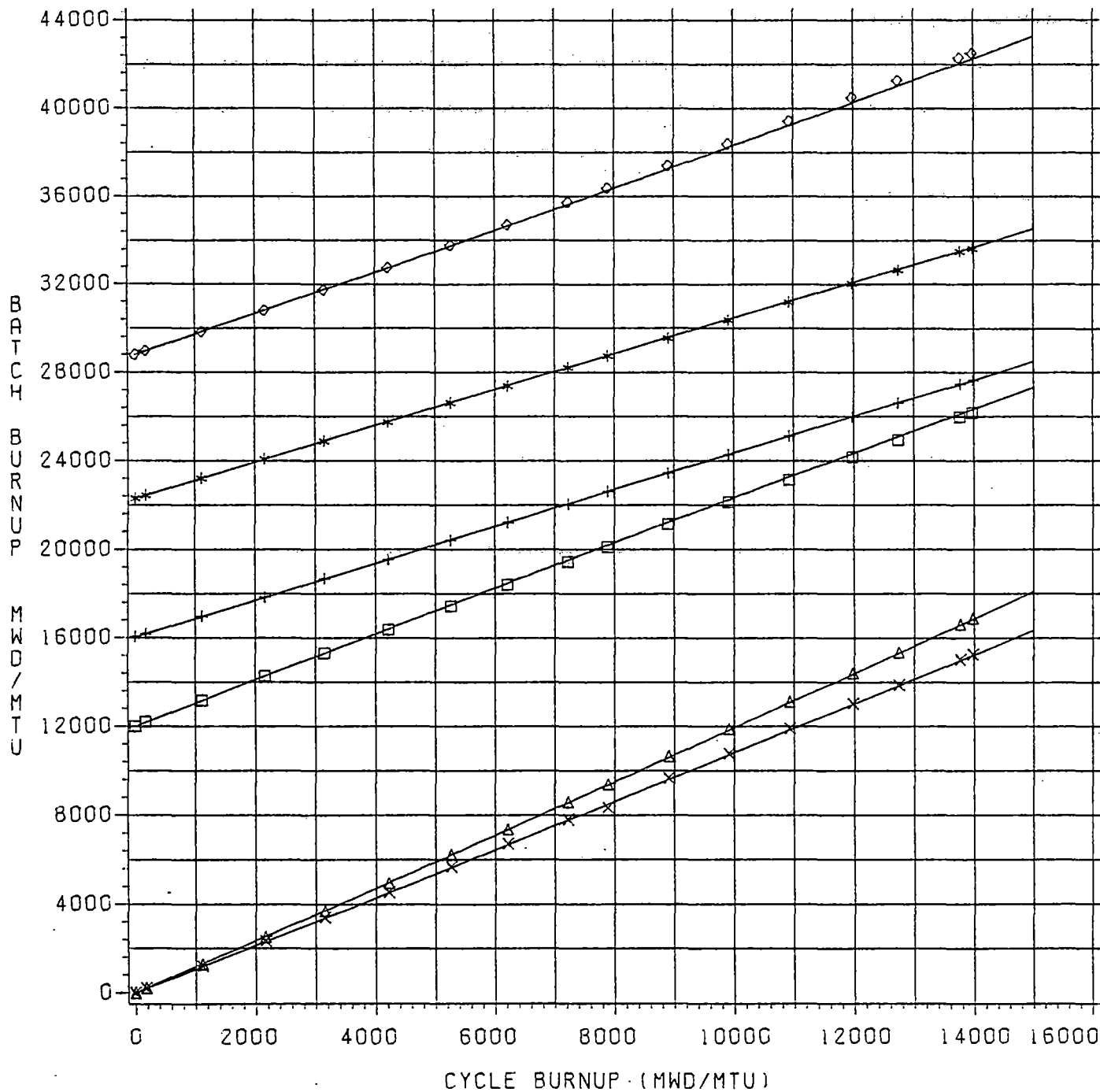


FIGURE 2.4
ASSEMBLYWISE ACCUMULATED BURNUP
COMPARISON OF MEASURED WITH PREDICTED
(1000 MWD/MTU)

| | R | P | N | M | L | K | J | H | G | F | E | D | C | B | A | |
|----|--------------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------|-------------|-------|------------|
| 1 | | | | | | | 22.51 | 10.09 | 22.21 | | | | | | | MEASURED |
| | | | | | | | 0.64 | -1.08 | -0.70 | | | | | | | M/P % DIFF |
| 2 | | | | 28.36 | 12.96 | 14.51 | 23.23 | 14.48 | 12.92 | 28.27 | | | | | | |
| | | | | 0.46 | 0.31 | -1.02 | -1.49 | -1.18 | -0.04 | 0.15 | | | | | | |
| 3 | | | 17.48 | 14.82 | 16.73 | 29.77 | 30.35 | 29.71 | 16.68 | 14.83 | 17.70 | | | | | |
| | | | 0.12 | 0.85 | 0.46 | -0.63 | -1.73 | -0.83 | 0.15 | 0.93 | 1.35 | | | | | |
| 4 | 17.59 | 28.11 | 16.38 | 29.85 | 16.99 | 30.59 | 16.92 | 29.92 | 16.48 | 28.23 | 17.71 | | | | | |
| | 0.75 | -0.19 | -0.70 | -0.88 | -0.54 | -0.61 | -0.97 | -0.63 | -0.11 | 0.23 | 1.41 | | | | | |
| 5 | 28.48 | 14.75 | 16.36 | 25.98 | 36.88 | 35.90 | 24.99 | 35.69 | 36.62 | 26.20 | 16.50 | 15.19 | 28.60 | | | |
| | 0.91 | 0.37 | -0.83 | -1.33 | -0.18 | 0.05 | -0.47 | -0.51 | -0.90 | -0.47 | 0.04 | 3.37 | 1.30 | | | |
| 6 | 12.91 | 16.69 | 29.87 | 36.85 | 26.26 | 17.69 | 31.17 | 17.61 | 26.15 | 36.71 | 30.12 | 16.97 | 13.35 | | | |
| | -0.11 | 0.23 | -0.81 | -0.27 | -1.28 | 0.19 | -0.21 | -0.27 | -1.69 | -0.65 | 0.03 | 1.92 | 3.27 | | | |
| 7 | 22.43 | 14.58 | 29.71 | 16.97 | 35.80 | 17.59 | 31.56 | 17.65 | 31.59 | 17.62 | 35.38 | 17.00 | 29.96 | 14.75 | 22.45 | |
| | 0.31 | -0.51 | -0.83 | -0.70 | -0.22 | -0.38 | 0.35 | 0.48 | 0.43 | -0.25 | -1.39 | -0.49 | -0.01 | 0.66 | 0.37 | |
| 8 | 10.12 | 23.30 | 30.72 | 30.64 | 24.71 | 31.16 | 17.75 | 42.51 | 17.65 | 31.16 | 24.70 | 30.62 | 30.72 | 23.39 | 10.43 | |
| | -0.74 | -1.16 | -0.55 | -0.44 | -1.57 | -0.25 | 1.08 | 0.58 | 0.49 | -0.24 | -1.64 | -0.50 | -0.54 | -0.81 | 2.31 | |
| 9 | 22.44 | 14.61 | 29.90 | 17.03 | 35.98 | 17.44 | 31.26 | 17.63 | 31.64 | 17.44 | 35.48 | 17.00 | 29.82 | 14.86 | 22.81 | |
| | 0.33 | -0.31 | -0.22 | -0.32 | 0.29 | -1.23 | -0.61 | 0.40 | 0.62 | -1.28 | -1.12 | -0.51 | -0.47 | 1.43 | 1.98 | |
| 10 | 13.00 | 16.80 | 30.17 | 36.84 | 26.18 | 17.27 | 31.10 | 17.61 | 26.33 | 36.93 | 30.03 | 16.66 | 12.99 | | | |
| | 0.55 | 0.89 | 0.19 | -0.31 | -1.59 | -2.22 | -0.44 | -0.27 | -1.02 | -0.05 | -0.26 | 0.02 | 0.48 | | | |
| 11 | 28.34 | 14.96 | 16.45 | 25.88 | 36.65 | 35.70 | 24.58 | 35.85 | 36.96 | 26.11 | 16.61 | 14.97 | 28.21 | | | |
| | 0.39 | 1.82 | -0.27 | -1.71 | -0.80 | -0.49 | -2.11 | -0.09 | 0.01 | -0.83 | 0.70 | 1.84 | -0.07 | | | |
| 12 | 17.75 | 28.40 | 16.18 | 29.68 | 16.72 | 30.38 | 16.87 | 30.25 | 16.59 | 28.37 | 17.64 | | | | | |
| | 1.67 | 0.81 | -1.92 | -1.45 | -2.14 | -1.29 | -1.27 | 0.45 | 0.59 | 0.71 | 0.99 | | | | | |
| 13 | 17.91 | 15.58 | 17.00 | 29.56 | 30.29 | 29.70 | 16.84 | 14.95 | 17.71 | | | | | | | |
| | 2.59 | 6.05 | 2.10 | -1.33 | -1.93 | -0.88 | 1.10 | 1.76 | 1.42 | | | | | | | |
| 14 | 28.61 | 13.63 | 14.70 | 23.30 | 14.61 | 12.99 | 28.45 | | | | | | | | | |
| | 1.34 | 5.49 | 0.33 | -1.18 | -0.33 | 0.51 | 0.80 | | | | | | | | | |
| 15 | STANDARD DEV | | | | | | 22.69 | 10.41 | 22.16 | | | | | AVG ABS PCT | | |
| | = 0.86 | | | | | | 1.46 | 2.03 | -0.88 | | | | | DIFF = 0.89 | | |

BURNUP SHARING
(10³ MWD/MTU)

| BATCH | CYCLE 2 | CYCLE 3 | CYCLE 4 | CYCLE 5 | TOTAL |
|--------------|---------|---------|---------|---------|-------|
| 4B2 | 7.28 | 6.61 | 14.91 | 13.71 | 42.51 |
| 5A | - | 6.79 | 15.51 | 11.35 | 33.65 |
| 6A1 | - | - | 16.03 | 11.60 | 27.63 |
| 6B | - | - | 11.93 | 14.24 | 26.17 |
| 7A | - | - | - | 16.85 | 16.85 |
| 7B | - | - | - | 15.25 | 15.25 |
| CORE AVERAGE | | | | 13.97 | |

BURNUP TILT

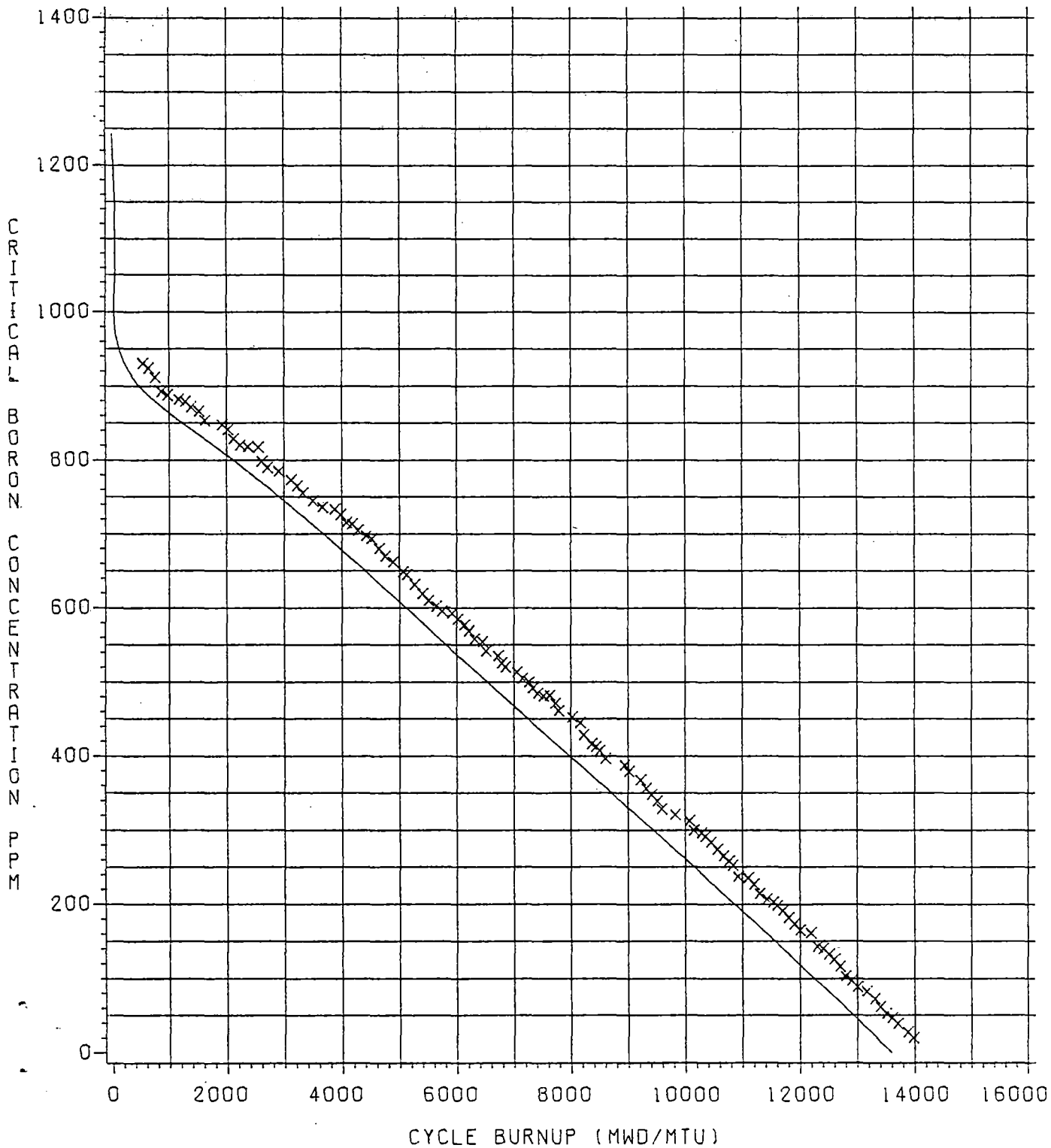
NW- 0.9986
NE- 1.0000
SW- 1.0006
SE- 1.0008

FIGURE 3.1

SURRY UNIT 2-CYCLE 5
CRITICAL BORON CONCENTRATION VS. BURNUP

HFP-ARO

X MEASURED
- PREDICTED



Section 3

REACTIVITY DEPLETION FOLLOW

The primary coolant critical boron concentration is monitored for the purposes of following core reactivity and to identify any anomalous reactivity behavior. The FOLLOW⁴ computer code was used to normalize "actual" critical boron concentration measurements to design conditions taking into consideration control rod position, xenon and samarium concentrations, moderator temperature, and power level. The normalized critical boron concentration versus burnup curve for the Surry 2, Cycle 5 core is shown in Figure 3.1. It can be seen that the measured data compare to within 50 ppm of the design prediction. This corresponds to less than $\pm 0.5\%$ delta K/K which is well within the $\pm 1\%$ delta K/K criterion for reactivity anomalies set forth in Section 4.10 of the Technical Specifications. In conclusion, the trend indicated by the critical boron concentration verifies that the Cycle 5 core depleted as expected without any reactivity anomalies.

Section 4

POWER DISTRIBUTION FOLLOW

Analysis of core power distribution data on a routine basis is necessary to verify that the hot channel factors are within the Technical Specifications limits and to ensure that the reactor is operating without any abnormal conditions which could cause an "uneven" burnup distribution. Three-dimensional core power distributions are determined from movable detector flux map measurements using the INCORE⁵ computer program. A summary of all monthly flux maps taken since the completion of startup physics testing for Surry 2, Cycle 5 is given in Table 4.1. Power distribution maps were generally taken at monthly intervals with additional maps taken as needed.

Radial (X-Y) core power distributions for a representative series of incore flux maps are given in Figures 4.1 through 4.3. Figure 4.1 shows a power distribution map that was taken early in cycle life. Figure 4.2 shows a power distribution map that was taken near mid-cycle burnup. Figure 4.3 shows a map that was taken late in Cycle 5 life. Most of the radial power distributions were taken under equilibrium operating conditions with the unit at approximately full power. In each case, the measured relative assembly powers were generally within 4% of the predicted values. In addition, as indicated by the INCORE tilt factors, the power distributions were essentially symmetric for all cases.

An important aspect of core power distribution follow is the monitoring of nuclear hot channel factors. Verification that these

factors are within Technical Specifications limits ensures that linear power density and critical heat flux limits will not be violated, thereby providing adequate thermal margins and maintaining fuel cladding integrity. The initial Cycle 5 Technical Specifications limit on the axially dependent heat flux hot channel factor, $F-Q(Z)$, was $2.19 \times K(Z)$, where $K(Z)$ is the hot channel factor normalized operating envelope. On July 28, 1980, the Technical Specifications limit for $F-Q(Z)$ was administratively reduced to $2.18 \times K(Z)$ ⁽⁶⁾. The formal Technical Specifications implementation of the 2.18 limit for $F-Q(Z)$ occurred with Technical Specifications Amendment No. 70, dated June 16, 1981⁽⁷⁾. Figure 4.4 is a plot of the $K(Z)$ curve associated with the 2.18 $F-Q(Z)$ limit. This curve is representative of the $K(Z)$ curves used throughout Cycle 5 since $K(Z)$ changes only slightly with changes in the $F-Q(Z)$ limit. The axially dependent heat flux hot channel factors, $F-Q(Z)$, for a representative set of flux maps are given in Figures 4.5 through 4.7. Throughout Cycle 5, the measured values of $F-Q(Z)$ were within the Technical Specifications limit. A summary of the maximum values of all heat flux hot channel factors measured during Cycle 5 is given in Figure 4.8. As can be seen from this figure, there was approximately 27% margin to the limit at the beginning of the cycle, with the margin approximately constant throughout cycle operation.

The value of the enthalpy rise hot channel factor, $F-\Delta H$, which is the ratio of the integral of the power along the rod with the highest integrated power to that of the average rod, is routinely followed. The Technical Specifications limit for this parameter is set such that the critical heat flux (DNB) limit will not be violated. Additionally, the

F-delta H limit ensures that the value of this parameter used in the LOCA-ECCS analysis is not exceeded during normal operation. The Cycle 5 limit on the enthalpy rise hot channel factor was set at $1.55 \times (1+0.2(1-P))$, where P is the fractional power level. The values of the enthalpy rise hot channel factor parameters F-delta H(LOCA,Assy) and F-delta H(LOCA,Rod) were also routinely followed². F-delta H(LOCA,Assy) and F-delta H(LOCA,Rod) represent the enthalpy rise hot channel factor (F-delta H) evaluated for the peak assembly and peak rod in the core, respectively, between the 1.5 ft. and 10.5 ft. levels of the core. The full power limits for F-delta H(LOCA,Assy) and F-delta H(LOCA,Rod) were set at 1.476 and 1.550 respectively. Table 4.2 summarizes the F-delta H(LOCA,Assy) and F-delta H(LOCA,Rod) values measured during Cycle 5 operation. Figures 4.9 through 4.11 show that all measured values for F-delta H, F-delta H(LOCA,Assy) and F-delta H(LOCA,Rod) were within their respective Technical Specifications limits during Cycle 5. The necessity for monitoring F-delta H(LOCA,Assy) and F-delta H(LOCA,Rod) was deleted from the Technical Specifications on June 16, 1981 by Technical Specifications Amendment No. 70⁽⁷⁾.

The Technical Specifications require that target delta flux* values be determined periodically. The target delta flux is the delta flux which would occur at conditions of full power, all rods out, and equilibrium xenon. Therefore, the delta flux is measured with the core

$$*\text{Delta Flux} = (P_t - P_b) \times 100 / 2441$$

where: P_t = Power in top of core (MW(th))

P_b = Power in bottom of core (MW(th))

at or near these conditions and the target delta flux is established at this measured point. Since the target delta flux varies as a function of burnup, the target value is updated monthly. Operational delta flux limits are then established about this target value. By maintaining the value of delta flux relatively constant, adverse axial power shapes due to xenon redistribution are avoided. The plot of the target delta flux versus burnup, given in Figure 4.12, shows the value of this parameter to have been approximately +1% at the beginning of Cycle 5. By the middle of the cycle, the value of delta flux had shifted to approximately -4%, and then returned to approximately -3% by the end of Cycle 5. This power shift can also be observed in the corresponding core average axial power distribution for a representative series of maps given in Figures 4.13 through 4.15. In Map S2-5-13 (Figure 4.13) taken at approximately 450 MWD/MTU, the axial power distribution had a flattened cosine shape with a peaking factor of 1.17. In Map S2-5-24 (Figure 4.14) taken at approximately 6,653 MWD/MTU, the axial power distribution had peaked slightly toward the bottom of the core with an axial peaking factor of 1.13. Finally, in Map S2-5-39 (Figure 4.15) taken at approximately 13,216 MWD/MTU, the axial power distribution remained slightly peaked toward the bottom of the core with an axial peaking factor of 1.13. The history of F-Z during the cycle can be seen more clearly in a plot of F-Z versus burnup given in Figure 4.16.

In conclusion, the Surry 2, Cycle 5 core performed very satisfactorily with power distribution analyses verifying that design predictions were accurate and that the values of the hot channel factors

were within the limits of the Technical Specifications.

TABLE 4.1
SURREY UNIT 2 - CYCLE 5

SUMMARY OF FLUX MAPS FOR ROUTINE OPERATION

| MAP NO. | DATE | BURN UP | | | 1 | | | | 2 | | | | 3 | | 4 | | AXIAL OFF SET (%) | NO. OF THIM BLES |
|---------|----------|---------|-----|-------|---------------------------|-----|-------------|----------------------------|------|-----|---------------|-------------|-------|-------|--------|-----|-------------------|------------------|
| | | MTU | (%) | STEPS | F-Q(T) HOT CHANNEL FACTOR | | | F-DH(N) HOT CHANNEL FACTOR | | | CORE F(Z) MAX | | QPTR | | | | | |
| | | | | | ASSY | PIN | AXIAL POINT | F-Q(T) | ASSY | PIN | F-DH(N) | AXIAL POINT | F(Z) | F(XY) | MAX | LOC | | |
| 13 | 9-12-80 | 450 | 100 | 215 | B06 | DE | 34 | 1.716 | D07 | IH | 1.387 | 23 | 1.171 | 1.352 | 1.0045 | SE | +1.130 | 49 |
| 14 | 9-20-80 | 720 | 98 | 217 | H09 | HG | 34 | 1.710 | H09 | HG | 1.403 | 34 | 1.182 | 1.344 | 1.0035 | NE | -1.519 | 42 |
| 17(5) | 10-17-80 | 1690 | 100 | 217 | J08 | GH | 23 | 1.709 | J08 | GH | 1.394 | 23 | 1.166 | 1.339 | 1.0026 | NE | +1.026 | 48 |
| 18 | 11-12-80 | 2540 | 100 | 214 | J08 | GH | 23 | 1.695 | J08 | GH | 1.402 | 34 | 1.158 | 1.344 | 1.0040 | SE | -0.697 | 49 |
| 19 | 12-15-80 | 3660 | 100 | 212 | D07 | IH | 21 | 1.702 | J08 | GH | 1.412 | 33 | 1.146 | 1.356 | 1.0039 | NE | -0.323 | 49 |
| 22(6) | 1-14-81 | 4704 | 100 | 215 | H09 | HG | 34 | 1.693 | H09 | HG | 1.430 | 44 | 1.135 | 1.367 | 1.0030 | SE | -0.753 | 48 |
| 23 | 2-11-81 | 5619 | 100 | 219 | J08 | GH | 23 | 1.675 | J08 | GH | 1.424 | 44 | 1.129 | 1.372 | 1.0030 | SE | -0.488 | 49 |
| 24 | 3-13-81 | 6653 | 100 | 227 | L13 | MN | 44 | 1.670 | J08 | GH | 1.424 | 45 | 1.133 | 1.372 | 1.0025 | SW | -0.963 | 47 |
| 27(7) | 4-15-81 | 7666 | 100 | 225 | G08 | IH | 45 | 1.691 | J08 | GH | 1.429 | 45 | 1.135 | 1.381 | 1.0046 | SE | -1.725 | 49 |
| 28 | 5-12-81 | 8258 | 100 | 228 | L13 | LL | 46 | 1.773 | L13 | LL | 1.451 | 45 | 1.144 | 1.409 | 1.0023 | SW | -2.935 | 48 |
| 29 | 6-10-81 | 9230 | 100 | 227 | J08 | EH | 46 | 1.710 | J08 | EH | 1.428 | 45 | 1.147 | 1.383 | 1.0019 | SE | -3.436 | 46 |
| 34(8) | 7-19-81 | 10537 | 100 | 214 | H09 | LJ | 47 | 1.720 | J08 | DI | 1.425 | 46 | 1.154 | 1.377 | 1.0019 | NE | -4.504 | 44 |
| 35 | 8-12-81 | 11320 | 100 | 220 | H09 | LJ | 47 | 1.726 | H09 | LJ | 1.424 | 46 | 1.147 | 1.383 | 1.0013 | NE | -4.100 | 44 |
| 36 | 9-16-81 | 12210 | 100 | 216 | F07 | DG | 46 | 1.705 | F07 | DG | 1.421 | 46 | 1.136 | 1.372 | 1.0023 | NE | -3.532 | 45 |
| 39(9) | 10-15-81 | 13216 | 100 | 220 | L13 | LL | 45 | 1.676 | J06 | ID | 1.410 | 53 | 1.133 | 1.366 | 1.0011 | NE | -2.808 | 47 |

NOTES: HOT SPOT LOCATIONS ARE SPECIFIED BY GIVING ASSEMBLY LOCATIONS (E.G. H-8 IS THE CENTER-OF-CORE ASSEMBLY), FOLLOWED BY THE PIN LOCATION (DENOTED BY THE "Y" COORDINATE WITH THE FIFTEEN ROWS OF FUEL RODS LETTERED A THROUGH R AND THE "X" COORDINATE DESIGNATED IN A SIMILAR MANNER). IN THE "Z" DIRECTION THE CORE IS DIVIDED INTO 61 AXIAL POINTS STARTING FROM THE TOP OF THE CORE.

1. F-Q(T) INCLUDES A TOTAL UNCERTAINTY OF 1.08.
2. F-DH(N) INCLUDES A MEASUREMENT UNCERTAINTY OF 1.04.
3. F(XY) IS EVALUATED AT THE MIDPLANE OF THE CORE.
4. QPTR - QUADRANT POWER TILT RATIO.
5. MAPS 15 AND 16 WERE PARTIAL MAPS TAKEN FOR I/E CALIBRATION.
6. MAPS 20 AND 21 WERE PARTIAL MAPS TAKEN FOR I/E CALIBRATION.
7. MAPS 25 AND 26 WERE PARTIAL MAPS TAKEN FOR I/E CALIBRATION.
8. MAPS 30 AND 31 WERE PARTIAL MAPS TAKEN FOR I/E CALIBRATION BUT NOT USED. MAPS 32 AND 33 WERE PARTIAL MAPS TAKEN FOR I/E CALIBRATION.
9. MAPS 37 AND 38 WERE PARTIAL MAPS TAKEN FOR I/E CALIBRATION.

TABLE 4.2

SURRY UNIT 2 - CYCLE 5

SUMMARY TABLE OF LOCA ENTHALPY RISE
HOT CHANNEL FACTORS

| MAP NUMBER | BURNUP (MWD/MTU) | $F_{\Delta H}^N / \text{LOCA}^*_{\text{ASSY}}$ | LOCATION | $F_{\Delta H}^N / \text{LOCA}^*_{\text{ROD}}$ | ASSEMBLY | PIN |
|----------------------|------------------|--|----------|---|----------|-----|
| S2-5-13 | ~450 | 1.279 | F-10 | 1.392 | B-6 | DE |
| S2-5-14 | ~720 | 1.282 | F-10 | 1.395 | H-9 | HG |
| S2-5-17 | ~1690 | 1.263 | F-10 | 1.393 | H-9 | HG |
| S2-5-18 | ~2540 | 1.276 | J-8 | 1.400 | J-8 | GH |
| S2-5-19 | ~3660 | 1.299 | J-8 | 1.409 | J-8 | GH |
| S2-5-22 | ~4704 | 1.332 | H-9 | 1.433 | H-9 | HG |
| S2-5-23 | ~5619 | 1.334 | J-8 | 1.425 | J-8 | GH |
| S2-5-24 | ~6653 | 1.345 | J-8 | 1.425 | J-8 | GH |
| S2-5-27 | ~7666 | 1.357 | J-8 | 1.434 | J-8 | GH |
| S2-5-28 | ~8258 | 1.314 | J-8 | 1.412 | L-13 | LL |
| S2-5-29 [#] | ~9230 | 1.372 | J-8 | 1.434 | J-8 | EH |

* $F_{\Delta H}^N / \text{LOCA}_{\text{ASSY}}$ and $F_{\Delta H}^N / \text{LOCA}_{\text{ROD}}$ are measured between 1.5 feet and 10.5 feet of the core elevation and include an uncertainty of 1.04.

[#]The monitoring of $F_{\Delta H}^N / \text{LOCA}_{\text{ASSY}}$ and $F_{\Delta H}^N / \text{LOCA}_{\text{ROD}}$ was discontinued by Technical Specification Amendment No. 70, dated June 16, 1981.

SURRY UNIT 2 - CYCLE 5

ASSEMBLYWISE POWER DISTRIBUTION

S2-5-13

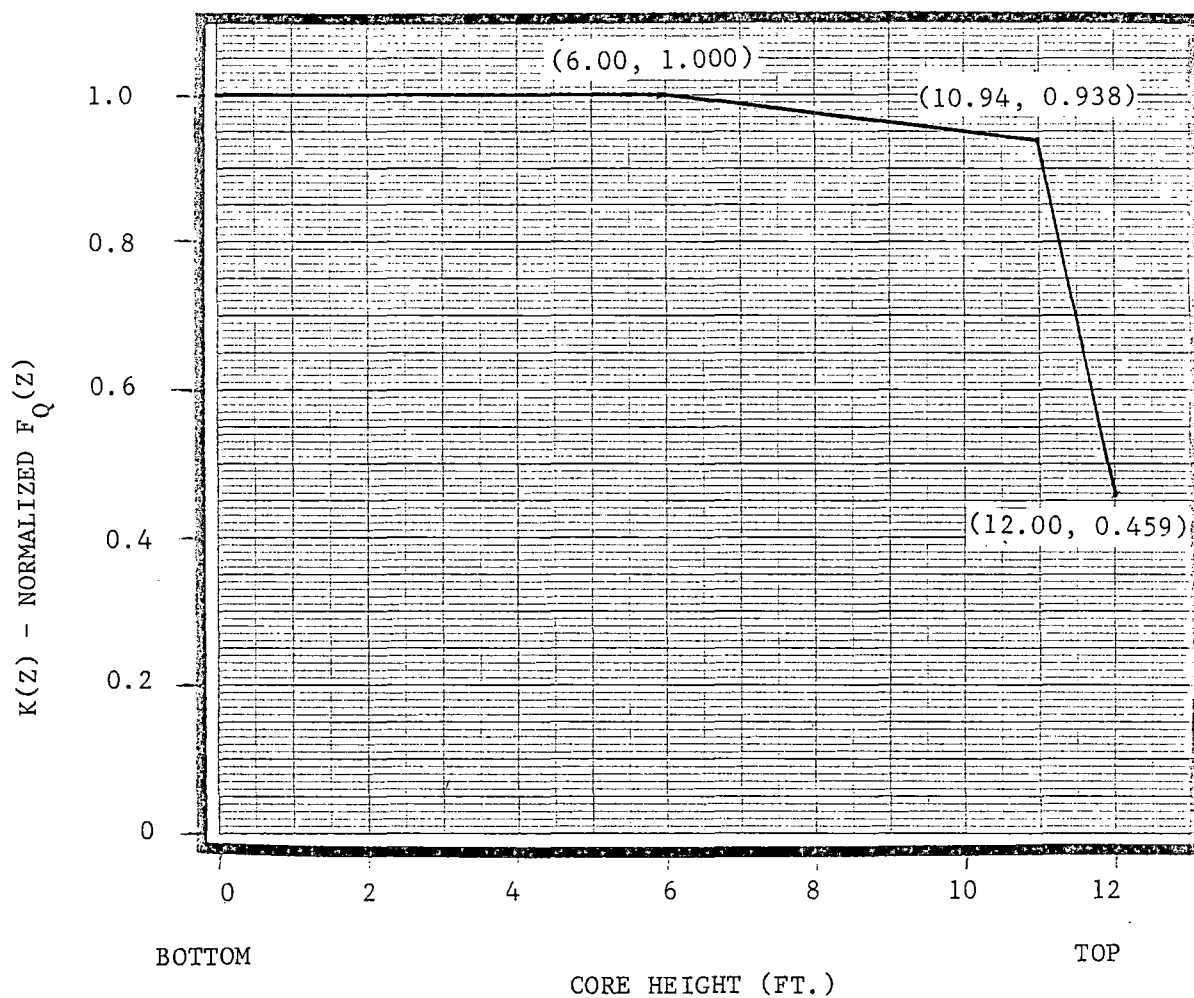
| | R | P | H | M | L | K | J | II | G | F | E | D | C | B | A |
|----------------|----------------|------|------|------|------|------|------|------|------|------|------|------|------|------|------|
| | | | | | | | | | | | | | | | |
| PREDICTED | 0.43 0.75 0.43 | | | | | | | | | | | | | | |
| MEASURED | 0.43 0.75 0.42 | | | | | | | | | | | | | | |
| PCT DIFFERENCE | 0.1 0.1 -0.4 | | | | | | | | | | | | | | |
| | | | | | | | | | | | | | | | |
| | 0.42 | 0.93 | 1.04 | 1.11 | 1.04 | 0.93 | 0.42 | | | | | | | | |
| | 0.41 | 0.93 | 1.04 | 1.10 | 1.03 | 0.92 | 0.41 | | | | | | | | |
| | -0.2 | -0.6 | -0.7 | -0.8 | -1.0 | -1.3 | -0.7 | | | | | | | | |
| | | | | | | | | | | | | | | | |
| | 0.56 | 1.01 | 1.14 | 1.18 | 1.18 | 1.18 | 1.14 | 1.01 | 0.56 | | | | | | |
| | 0.56 | 1.01 | 1.13 | 1.17 | 1.16 | 1.16 | 1.12 | 1.00 | 0.56 | | | | | | |
| | -0.2 | -0.1 | -0.3 | -0.8 | -1.6 | -1.4 | -1.1 | -0.6 | 0.1 | | | | | | |
| | | | | | | | | | | | | | | | |
| | 0.56 | 0.91 | 1.14 | 1.16 | 1.19 | 1.09 | 1.19 | 1.16 | 1.14 | 0.91 | 0.56 | | | | |
| | 0.56 | 0.90 | 1.13 | 1.15 | 1.17 | 1.08 | 1.17 | 1.15 | 1.13 | 0.91 | 0.57 | | | | |
| | 0.4 | -0.5 | -0.4 | -1.1 | -1.3 | -1.5 | -1.4 | -0.9 | -0.4 | 0.2 | 2.7 | | | | |
| | | | | | | | | | | | | | | | |
| | 0.42 | 1.01 | 1.14 | 1.23 | 1.03 | 1.04 | 1.20 | 1.04 | 1.03 | 1.23 | 1.14 | 1.01 | 0.42 | | |
| | 0.42 | 1.02 | 1.13 | 1.20 | 1.01 | 1.02 | 1.18 | 1.02 | 1.01 | 1.21 | 1.15 | 1.05 | 0.44 | | |
| | 0.9 | 0.9 | -0.7 | -2.3 | -1.4 | -1.5 | -1.5 | -1.5 | -1.6 | -1.1 | 0.9 | 4.0 | 6.3 | | |
| | | | | | | | | | | | | | | | |
| | 0.93 | 1.14 | 1.16 | 1.03 | 1.23 | 1.21 | 1.12 | 1.21 | 1.23 | 1.03 | 1.16 | 1.14 | 0.93 | | |
| | 0.94 | 1.14 | 1.15 | 1.02 | 1.21 | 1.19 | 1.11 | 1.19 | 1.19 | 1.02 | 1.17 | 1.17 | 0.98 | | |
| | 0.3 | 0.3 | -0.6 | -0.6 | -1.1 | -1.0 | -1.2 | -1.2 | -2.5 | -0.5 | 0.8 | 3.4 | 5.3 | | |
| | | | | | | | | | | | | | | | |
| | 0.43 | 1.04 | 1.18 | 1.19 | 1.04 | 1.21 | 1.11 | 1.18 | 1.11 | 1.21 | 1.04 | 1.19 | 1.18 | 1.04 | 0.43 |
| | 0.43 | 1.05 | 1.18 | 1.18 | 1.02 | 1.19 | 1.11 | 1.17 | 1.11 | 1.19 | 1.04 | 1.21 | 1.20 | 1.03 | 0.44 |
| | 1.8 | 0.5 | -0.2 | -0.7 | -1.6 | -0.9 | -0.2 | -0.4 | -0.2 | -1.2 | 0.2 | 1.7 | 1.8 | 3.1 | 2.7 |
| | | | | | | | | | | | | | | | |
| | 0.75 | 1.11 | 1.18 | 1.09 | 1.20 | 1.12 | 1.18 | 0.91 | 1.18 | 1.12 | 1.20 | 1.09 | 1.13 | 1.11 | 0.75 |
| | 0.76 | 1.11 | 1.18 | 1.08 | 1.17 | 1.11 | 1.18 | 0.92 | 1.13 | 1.12 | 1.20 | 1.11 | 1.19 | 1.12 | 0.78 |
| | 1.8 | 0.6 | 0.3 | -0.8 | -2.6 | -1.3 | -0.1 | 0.2 | 0.1 | -0.1 | 0.5 | 1.0 | 0.7 | 1.4 | 4.0 |
| | | | | | | | | | | | | | | | |
| | 0.43 | 1.04 | 1.18 | 1.19 | 1.04 | 1.21 | 1.11 | 1.18 | 1.11 | 1.21 | 1.04 | 1.19 | 1.18 | 1.04 | 0.43 |
| | 0.43 | 1.06 | 1.20 | 1.19 | 1.04 | 1.19 | 1.09 | 1.13 | 1.11 | 1.20 | 1.03 | 1.18 | 1.18 | 1.07 | 0.45 |
| | 1.8 | 1.6 | 1.5 | 0.1 | -0.3 | -1.2 | -1.6 | -0.3 | 0.4 | -0.6 | -0.9 | -0.6 | 0.3 | 2.7 | 6.3 |
| | | | | | | | | | | | | | | | |
| | 0.93 | 1.14 | 1.16 | 1.03 | 1.23 | 1.21 | 1.12 | 1.21 | 1.23 | 1.03 | 1.16 | 1.14 | 0.93 | | |
| | 0.96 | 1.16 | 1.16 | 1.01 | 1.20 | 1.18 | 1.10 | 1.19 | 1.22 | 1.02 | 1.15 | 1.13 | 0.95 | | |
| | 2.5 | 2.5 | 0.1 | -1.7 | -1.7 | -1.8 | -2.0 | -0.9 | -0.5 | -0.8 | -1.2 | -0.1 | 1.7 | | |
| | | | | | | | | | | | | | | | |
| | 0.42 | 1.01 | 1.14 | 1.23 | 1.03 | 1.04 | 1.20 | 1.04 | 1.03 | 1.23 | 1.14 | 1.01 | 0.42 | | |
| | 0.43 | 1.03 | 1.14 | 1.18 | 1.00 | 1.02 | 1.15 | 1.02 | 1.02 | 1.22 | 1.15 | 1.02 | 0.42 | | |
| | 2.3 | 2.3 | -0.2 | -3.5 | -2.3 | -2.3 | -3.6 | -2.1 | -0.5 | -0.6 | 0.5 | 1.2 | 1.3 | | |
| | | | | | | | | | | | | | | | |
| | 0.56 | 0.91 | 1.14 | 1.16 | 1.19 | 1.09 | 1.19 | 1.16 | 1.14 | 0.91 | 0.56 | | | | |
| | 0.57 | 0.91 | 1.10 | 1.14 | 1.17 | 1.06 | 1.13 | 1.17 | 1.15 | 0.93 | 0.57 | | | | |
| | 2.2 | -0.1 | -3.5 | -2.1 | -1.6 | -2.7 | -0.9 | 1.0 | 1.2 | 2.5 | 1.8 | | | | |
| | | | | | | | | | | | | | | | |
| | 0.56 | 1.01 | 1.14 | 1.18 | 1.18 | 1.18 | 1.14 | 1.01 | 0.56 | | | | | | |
| | 0.57 | 1.04 | 1.15 | 1.16 | 1.15 | 1.19 | 1.17 | 1.04 | 0.57 | | | | | | |
| | 2.9 | 3.7 | 1.3 | -2.0 | -2.4 | 0.4 | 3.0 | 3.1 | 2.5 | | | | | | |
| | | | | | | | | | | | | | | | |
| | 0.42 | 0.93 | 1.04 | 1.11 | 1.04 | 0.93 | 0.42 | | | | | | | | |
| | 0.43 | 0.97 | 1.03 | 1.11 | 1.05 | 0.95 | 0.43 | | | | | | | | |
| | 3.7 | 4.1 | 1.6 | 0.1 | 0.6 | 2.0 | 3.2 | | | | | | | | |
| | | | | | | | | | | | | | | | |
| STANDARD | 0.43 0.75 0.43 | | | | | | | | | | | | | | |
| DEVIATION | 0.45 0.77 0.43 | | | | | | | | | | | | | | |
| = 1.225 | 4.4 2.9 0.6 | | | | | | | | | | | | | | |
| | | | | | | | | | | | | | | | |
| AVERAGE | PCT DIFFERENCE | | | | | | | | | | | | | | |
| = 1.4 | | | | | | | | | | | | | | | |

SUMMARY

| | | |
|------------------------|-----------------|---------------------|
| MAP NO: S2-5-13 | DATE: 9/12/80 | POWER: 100% |
| CONTROL ROD POSITIONS: | F-Q(T) = 1.716 | QPTR: |
| D BANK AT 215 STEPS | F-DH(N) = 1.387 | NW 0.995 NE 1.002 |
| | F(Z) = 1.171 | ----- ----- |
| | F(XY) = 1.352 | SW 0.998 SE 1.005 |
| BURNUP = 450 MWD/MTU | A.O = 1.13(%) | |

HOT CHANNEL FACTOR NORMALIZED

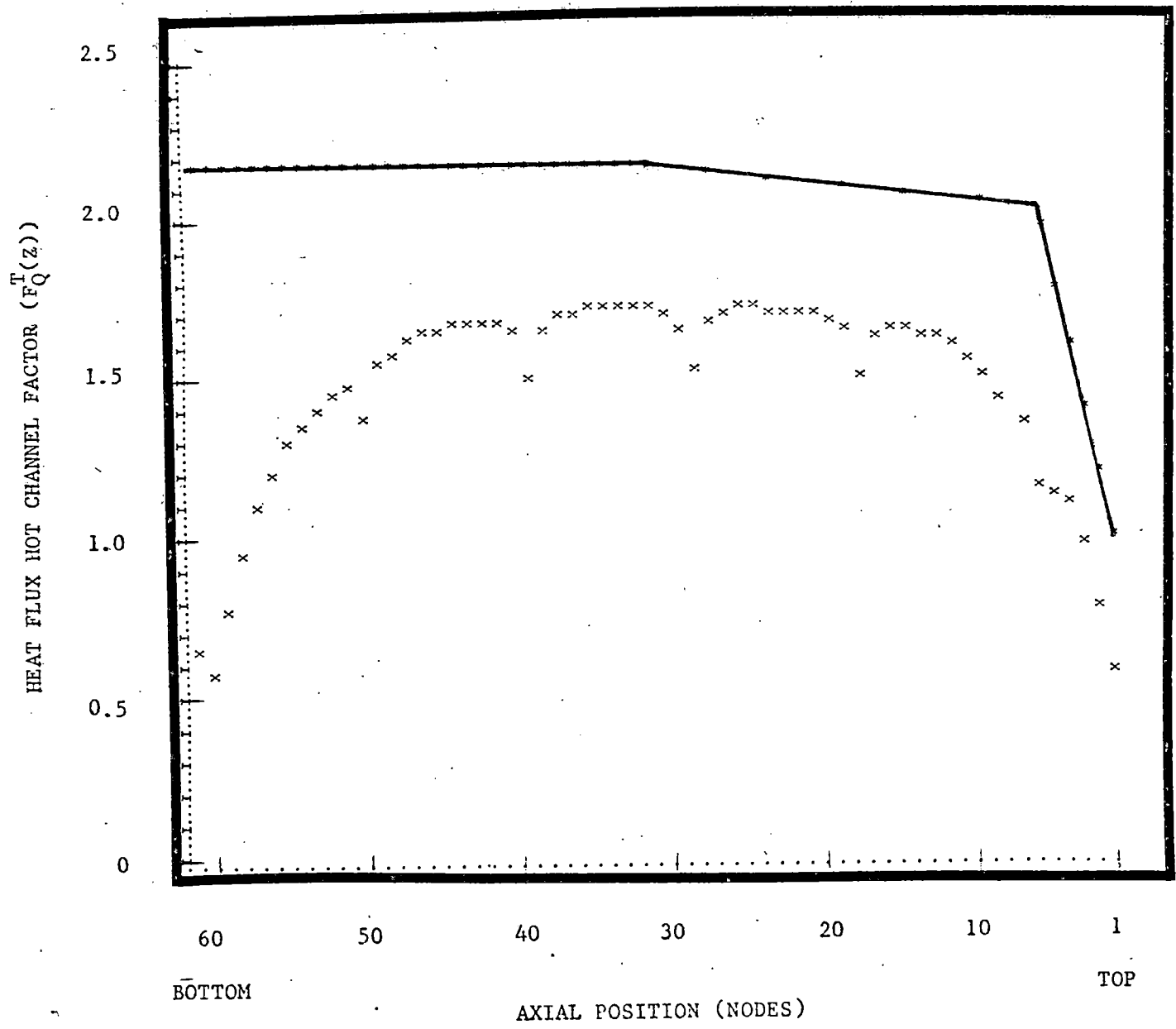
OPERATING ENVELOPE



SURRY UNIT 2 - CYCLE 5

HEAT FLUX HOT CHANNEL FACTOR, $F_Q^T(z)$

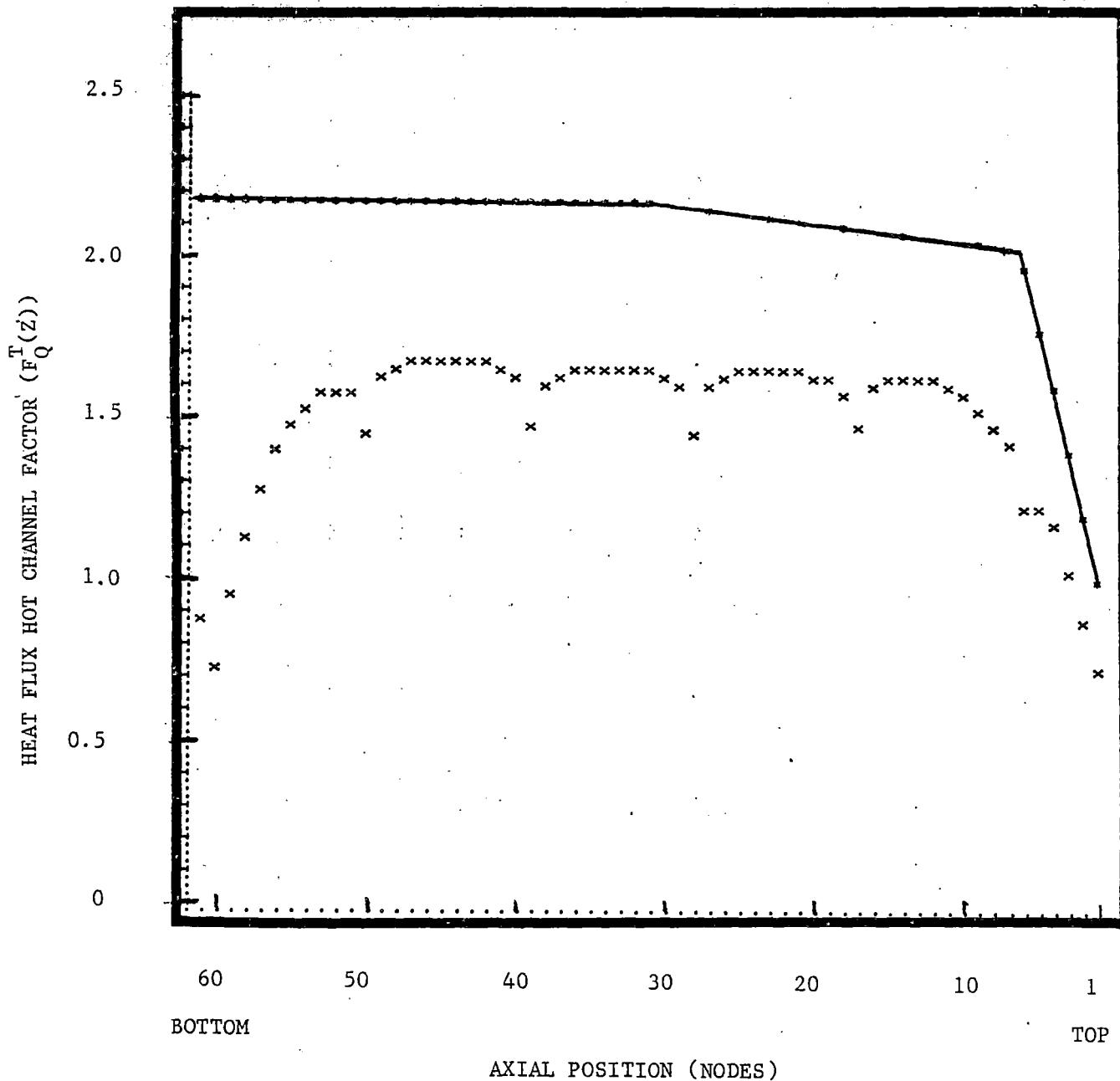
S2 - 5 - 13



SURRY UNIT 2 - CYCLE 5

HEAT FLUX HOT CHANNEL FACTOR, $F_Q^T(Z)$

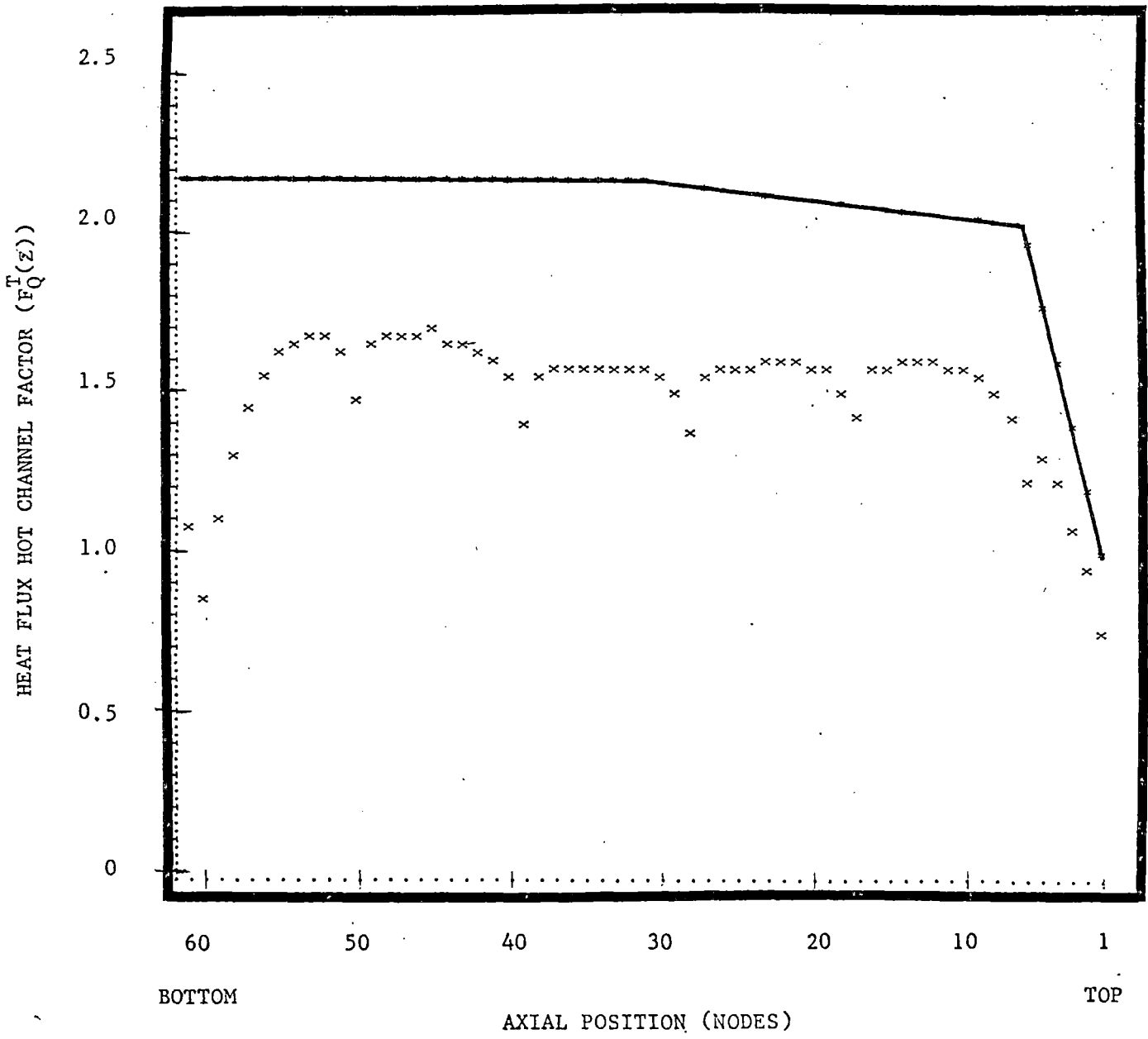
S2 - 5 - 24



SURRY UNIT 2 - CYCLE 5

HEAT FLUX HOT CHANNEL FACTOR, $F_Q^T(z)$

S2 - 5 - 39



SURRY 2' CYCLE 5
MAXIMUM HEAT FLUX HOT CHANNEL FACTOR
VS
BURNUP

FIGURE 4.8

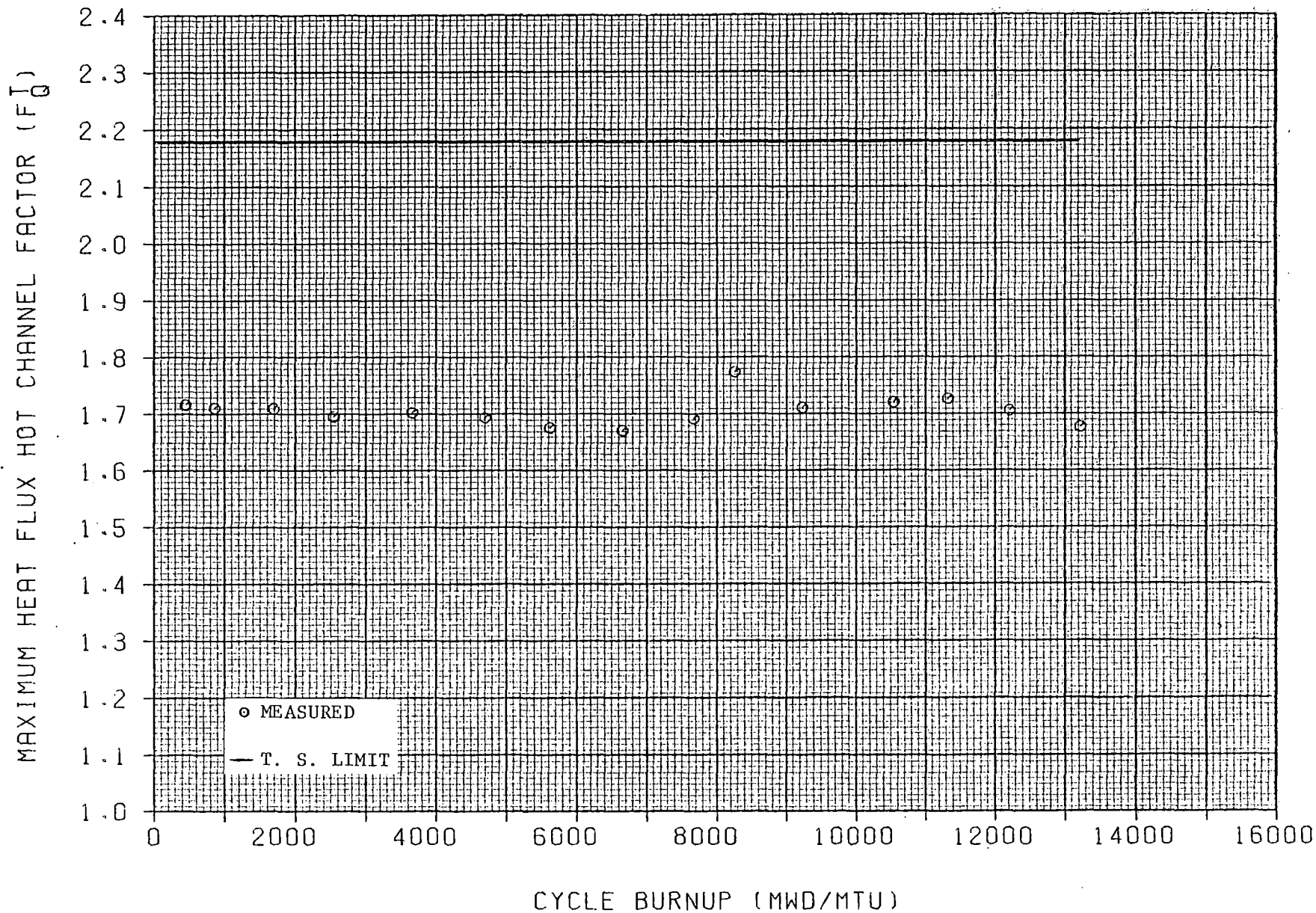
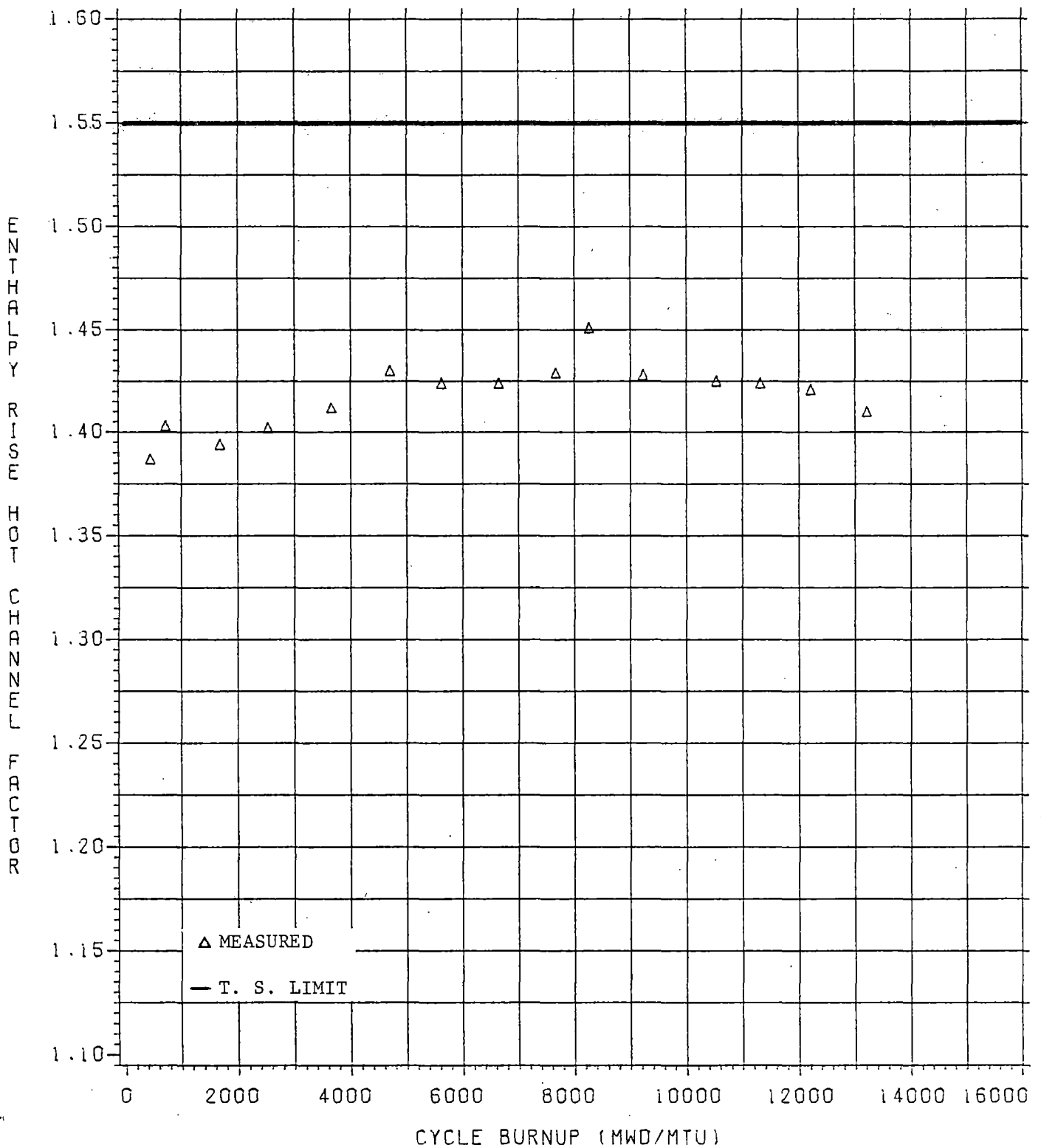


FIGURE 4.9

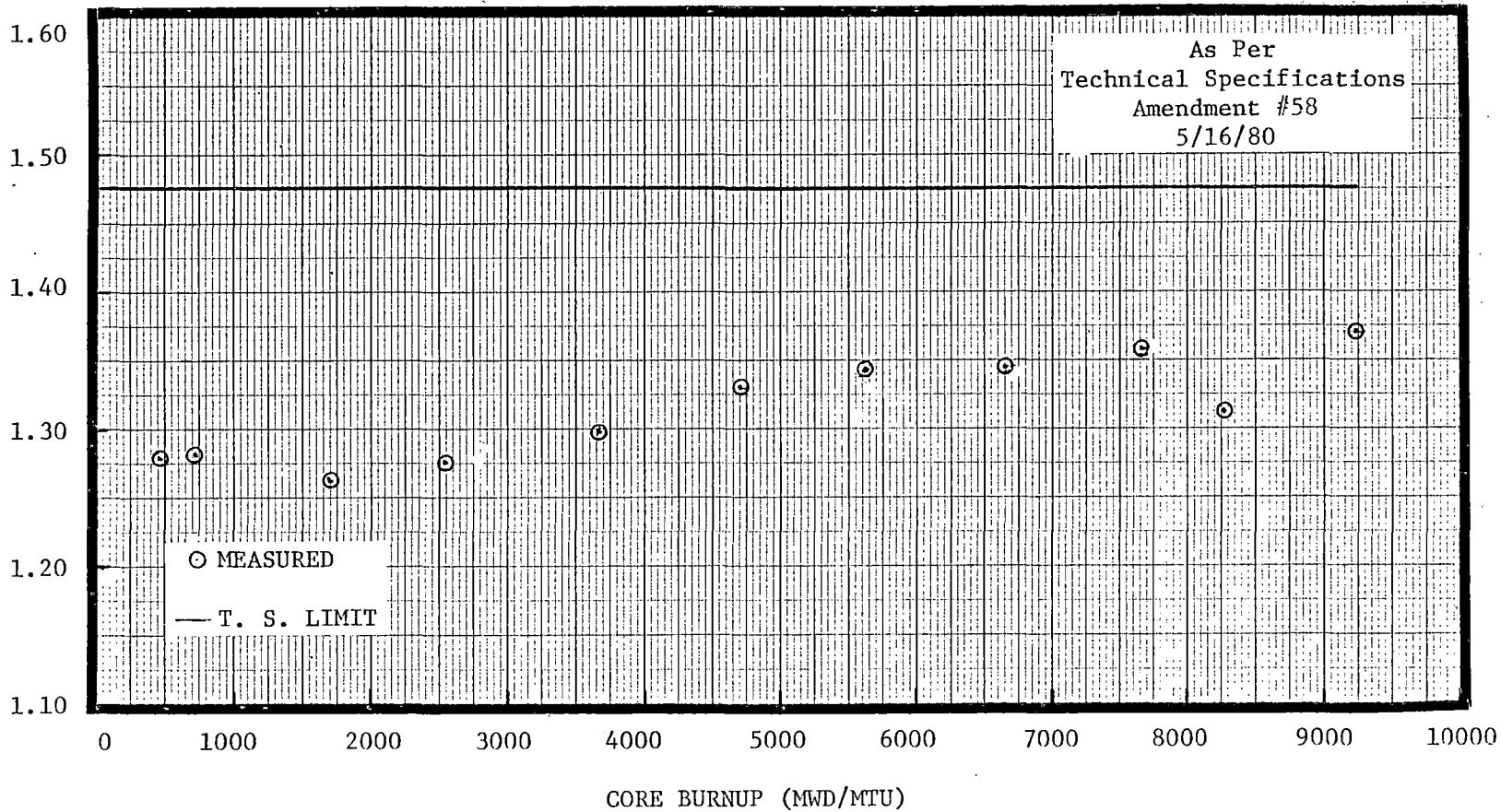
SURRY UNIT 2 - CYCLE 5
 ENTHALPY RISE HOT CHANNEL FACTOR, F-DH(N) VS. BURNUP



SURRY UNIT 2 - CYCLE 5

LOCA ENTHALPY RISE HOT CHANNEL FACTOR - ASSY.
VS.
BURNUP

LOCA ENTHALPY RISE HOT CHANNEL FACTOR - ASSY. ($F_{\Delta H}^{N/LOCA}/ASSY$)



SURRY UNIT 2 - CYCLE 5

LOCA ENTHALPY RISE HOT CHANNEL FACTOR - ROD
VS.
BURNUP

33

LOCA ENTHALPY RISE HOT CHANNEL FACTOR - ROD ($F_{\Delta H}^{N/LOCA}$)

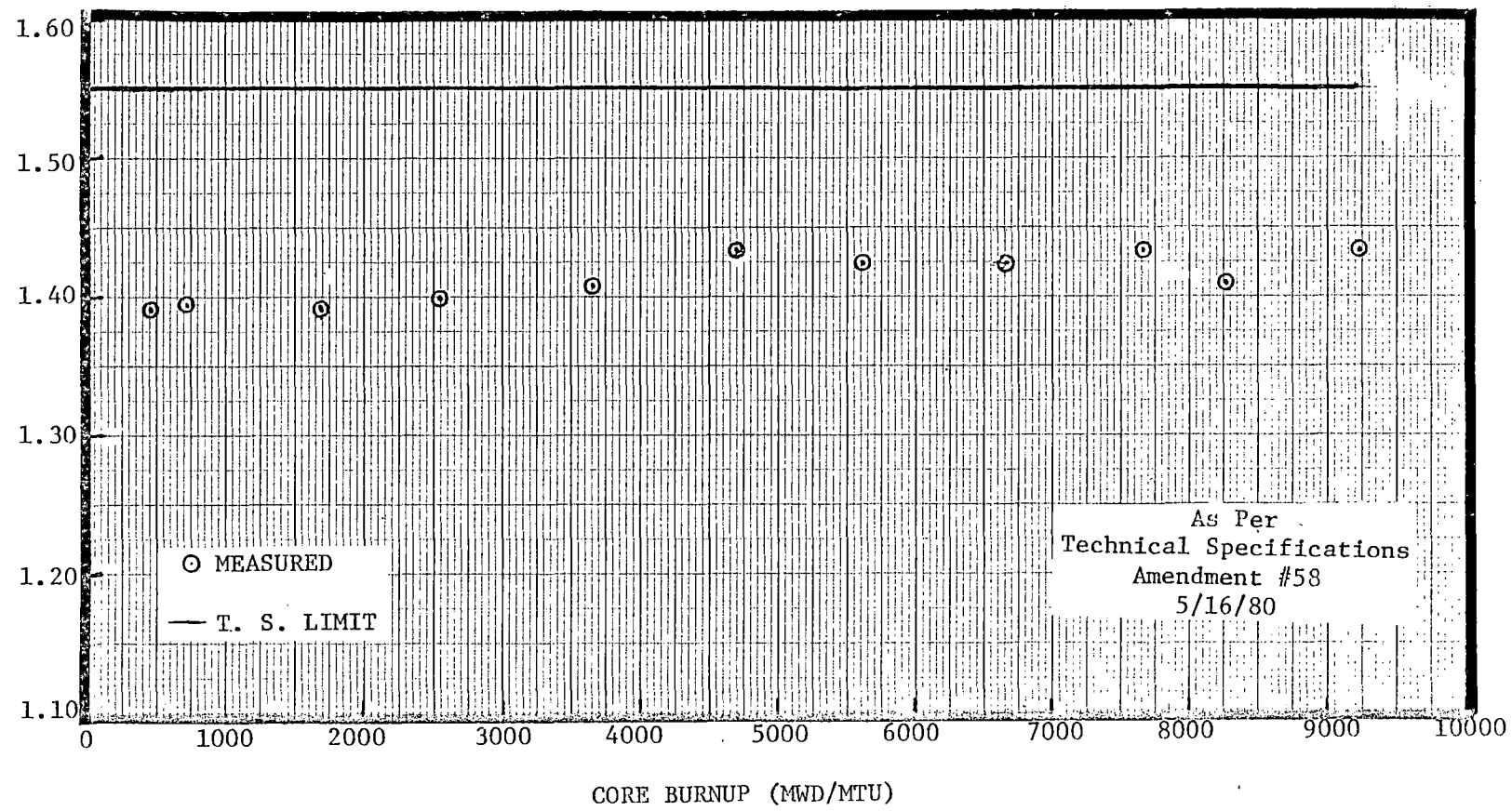
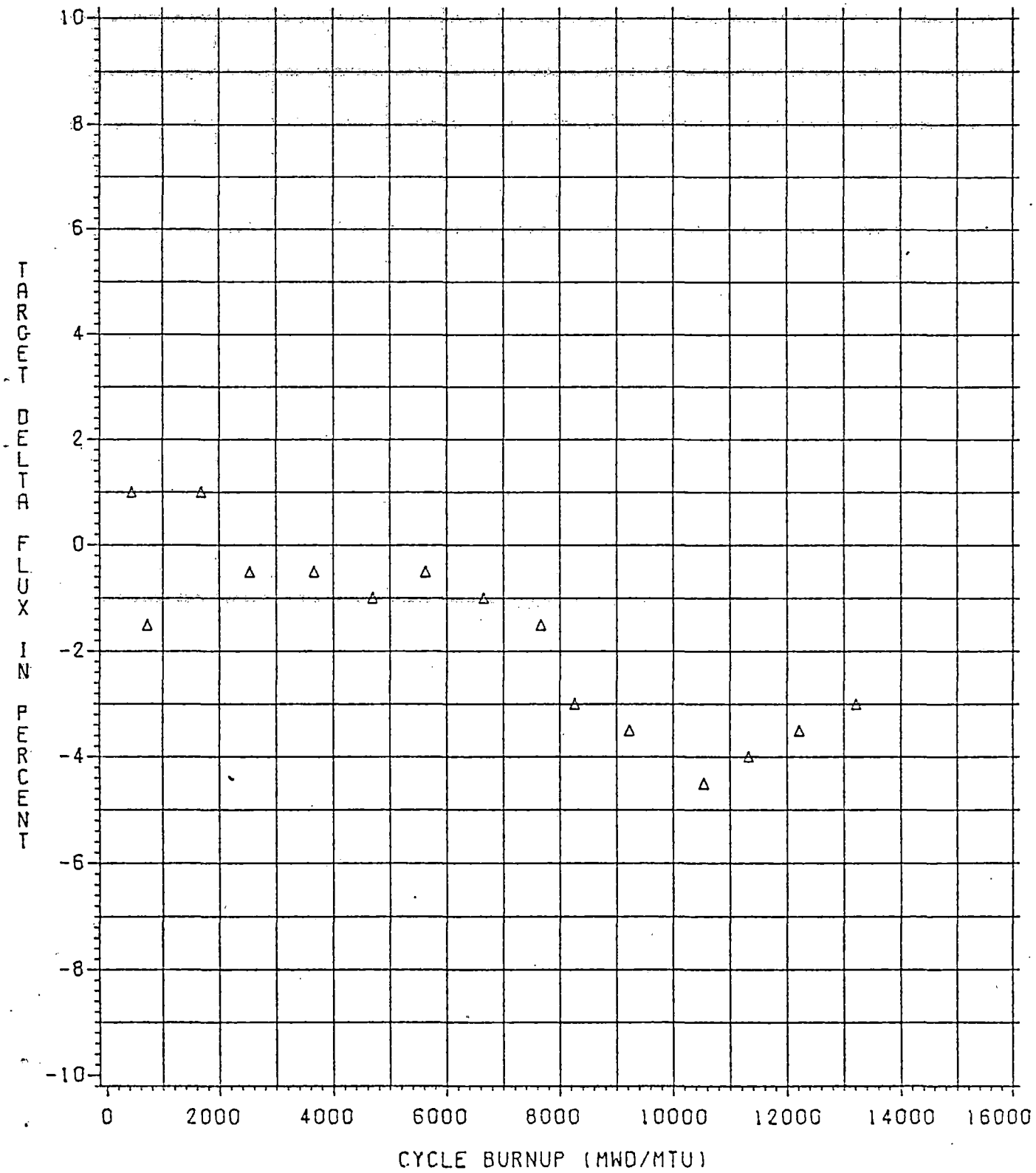


FIGURE 4.12 .

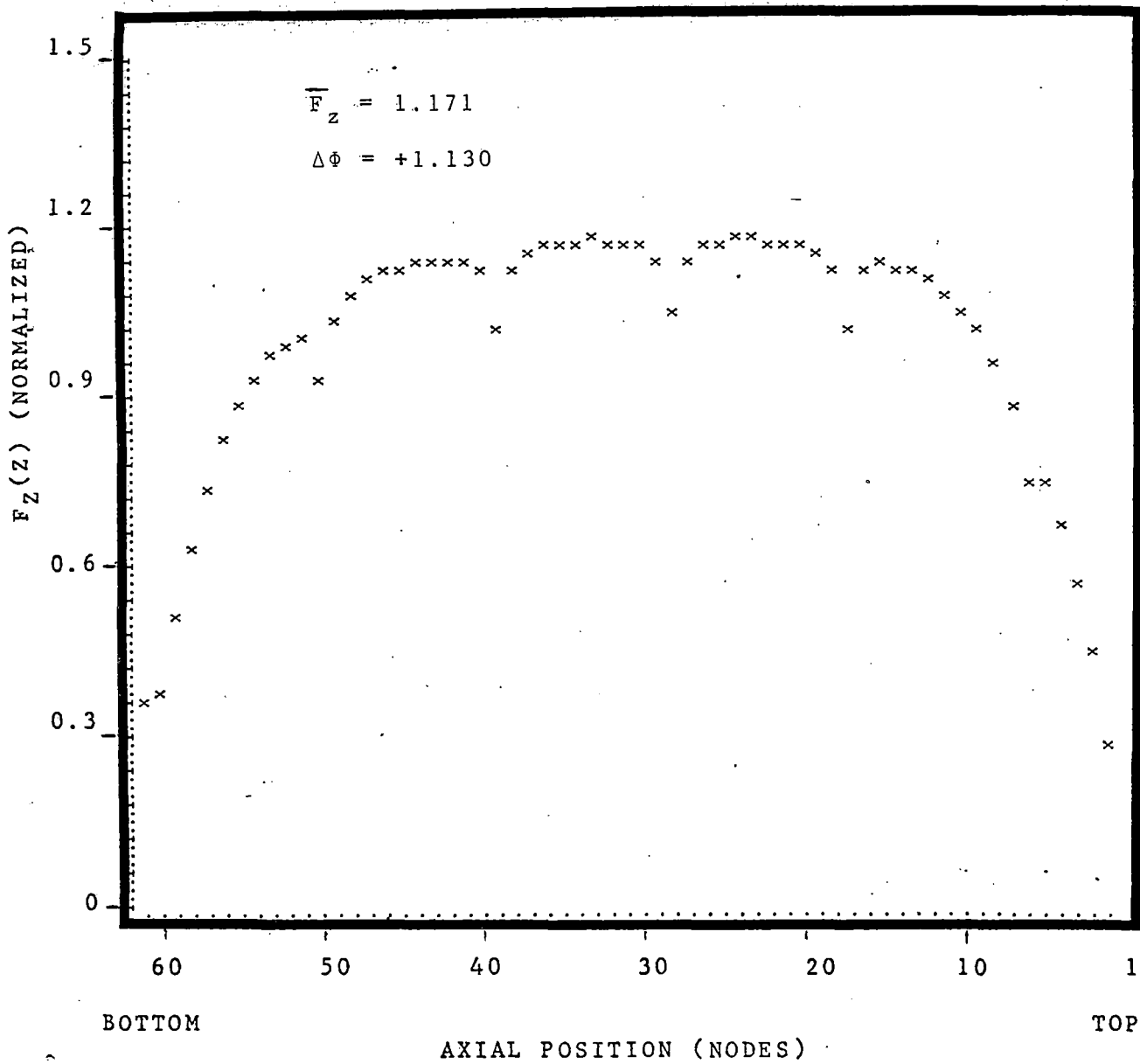
SURRY UNIT 2 - CYCLE 5
TARGET DELTA FLUX VS. BURNUP



SURRY UNIT 2 - CYCLE 5

CORE AVERAGE AXIAL POWER DISTRIBUTION

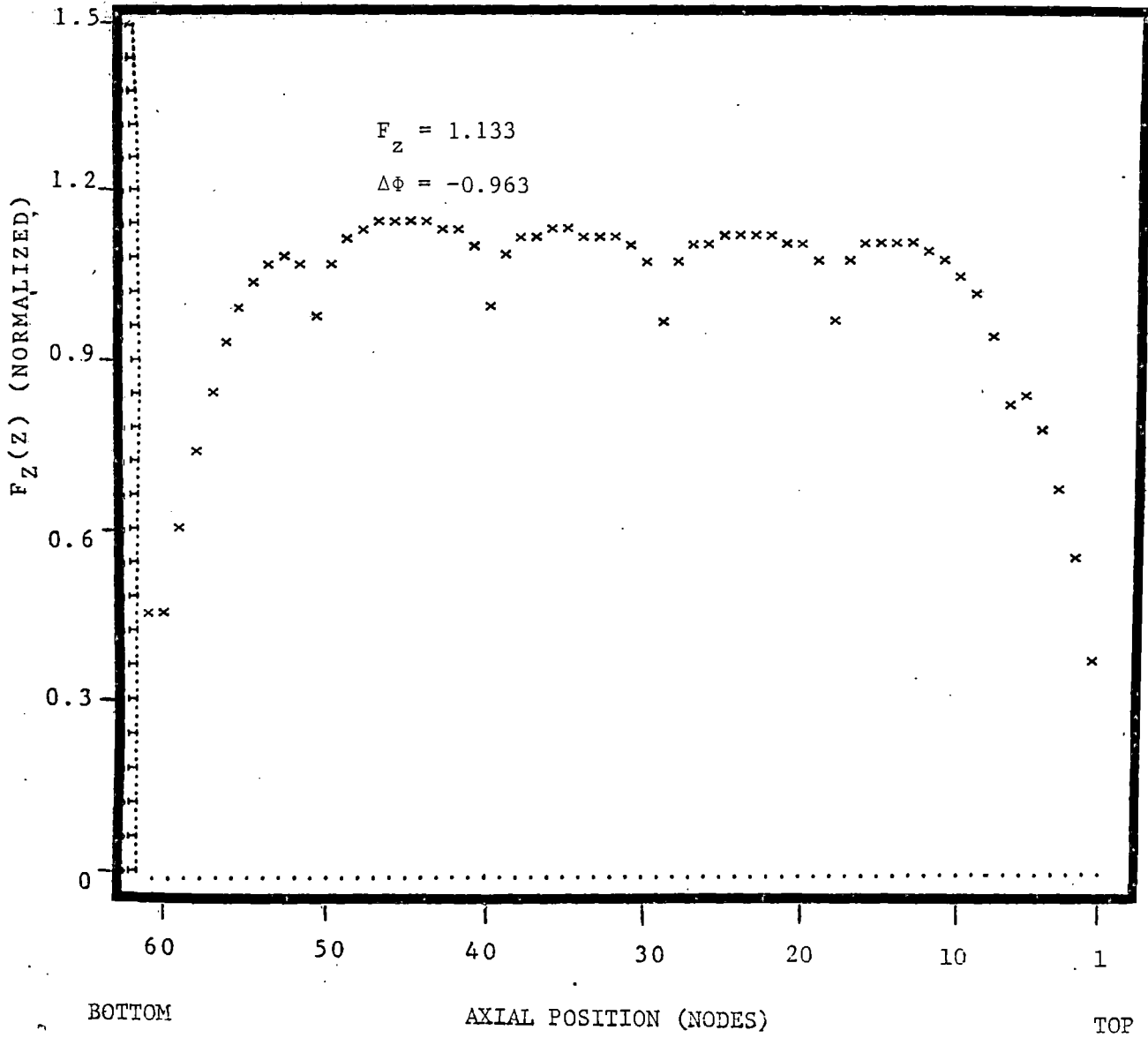
S2 - 5 - 13



SURRY UNIT 2 - CYCLE 5

CORE AVERAGE AXIAL POWER DISTRIBUTION

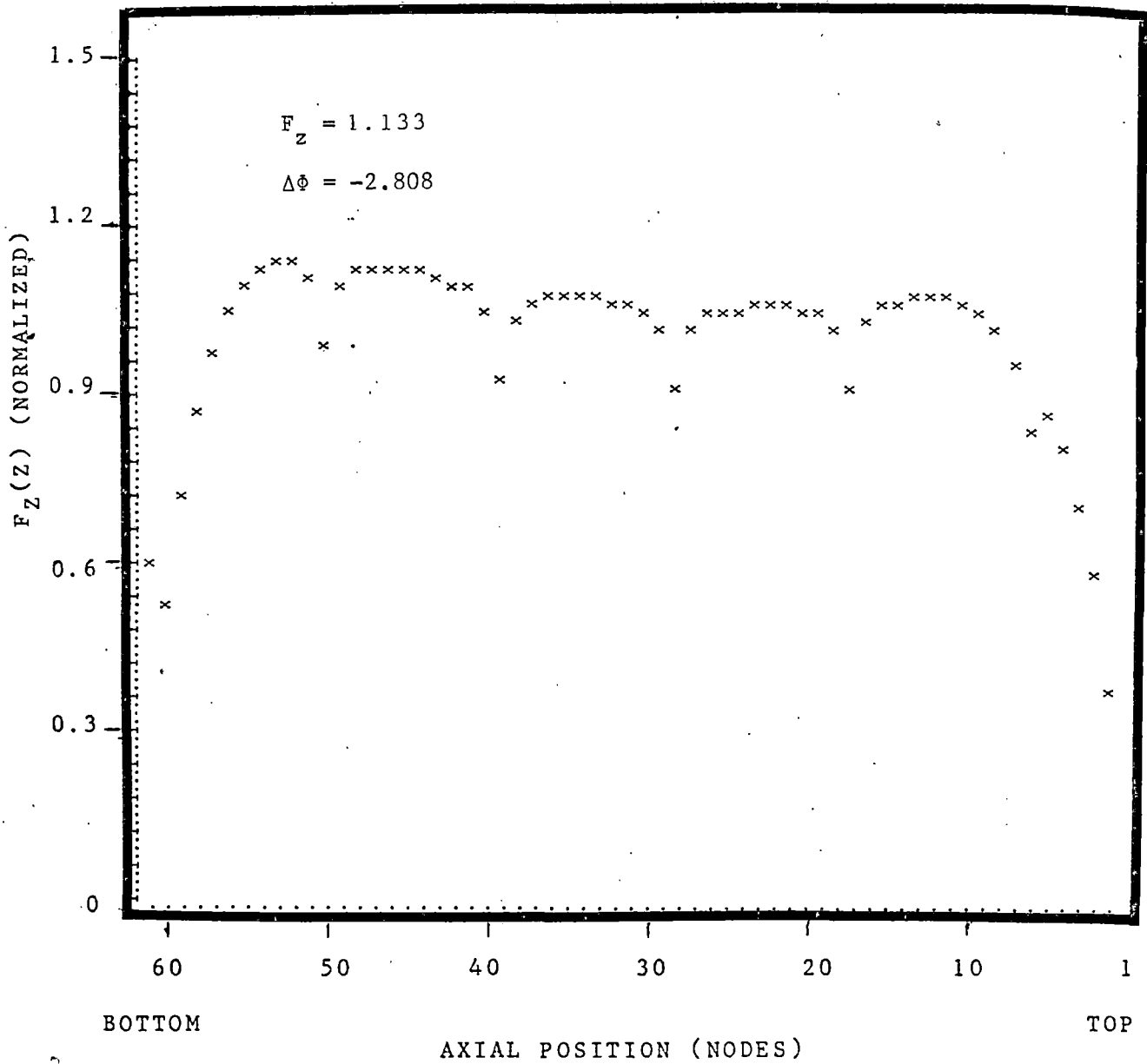
S2 - 5 - 24



SURRY UNIT 2 - CYCLE 5

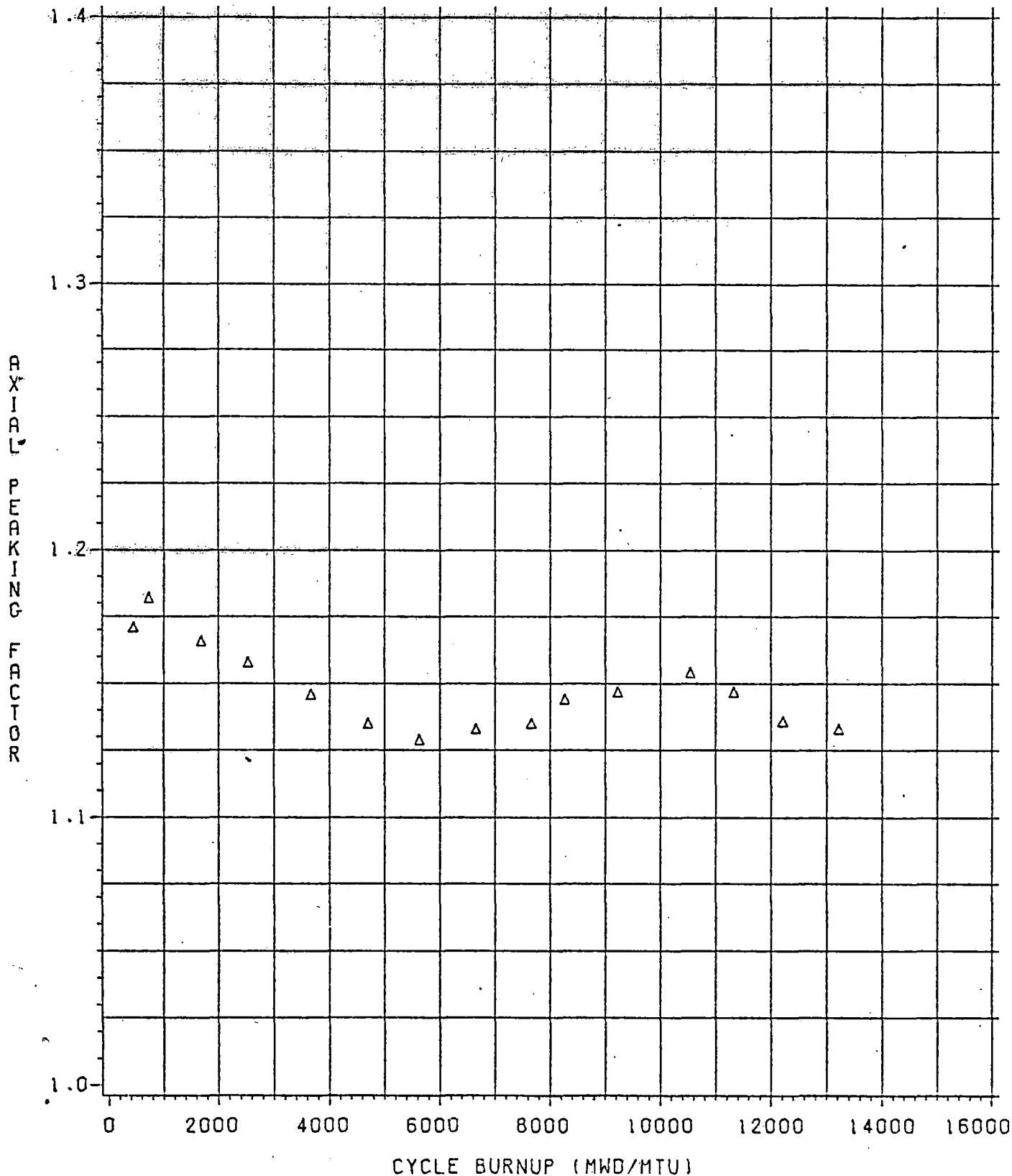
CORE AVERAGE AXIAL POWER DISTRIBUTION

S2 - 5 - 39



SURRY UNIT 2 - CYCLE 5

CORE AVERAGE AXIAL PEAKING FACTOR, F-Z VS. BURNUP



Section 5

PRIMARY COOLANT ACTIVITY FOLLOW

Activity levels of iodine-131 and 133 in the primary coolant are important in core performance follow analysis because they are used as indicators of defective fuel. Additionally, they are also important with respect to the offsite dose calculation values associated with accident analyses. Both I-131 and I-133 can leak into the primary coolant system through a breach in the cladding. As indicated in the Technical Specifications the dose equivalent I-131 concentration in the primary coolant was limited to 1.0 micro-Ci/gm for normal steady state operation during Cycle 5. Figure 5.1 shows the dose equivalent I-131 activity level history for the Surry 2, Cycle 5 core (the letdown flow rate averaged 105 gpm during power operation). The data demonstrates considerable scatter; however, the trend shows that during Cycle 5, the core operated substantially below the 1.0 micro-Ci/gm limit during steady state operation (the spike data is associated with power transients and unit shutdown). Specifically, the average dose equivalent I-131 concentration of 6.2×10^{-3} micro-Ci/gm is less than 1% of the Technical Specifications limit.

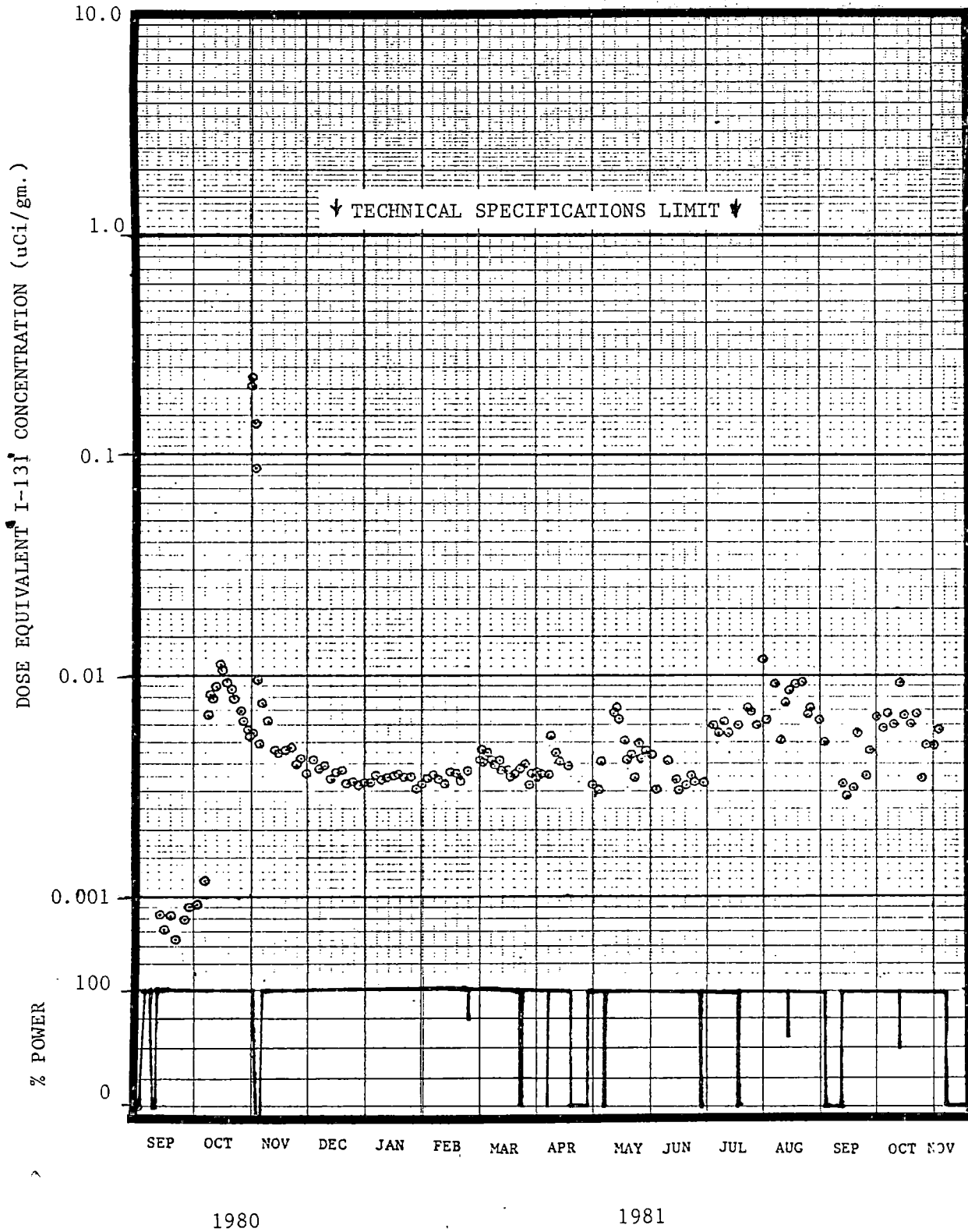
The ratio of the specific activities of I-131 to I-133 is used to characterize the type of fuel defect which may be present in a reactor core. Use of the ratio for this determination is feasible because I-133 has a short half-life (approximately 21 hours) compared to that of I-131 (approximately eight days) so that for pinhole defects where the

diffusion time through the defect is on the order of days, the I-133 decays out leaving the I-131 dominant in activity, thereby causing the ratio to be 0.5 or more. In the case of large leaks, uranium particles in the coolant, and/or "tramp" uranium*, where the diffusion mechanism is negligible, the I-131/I-133 ratio will generally be less than 0.1. Figure 5.2 shows the I-131/I-133 ratio data for the Surry 2, Cycle 5 core. These data indicate that any defects that may have been present during Cycle 5 are quite small.

*"Tramp" uranium consists of small particles of uranium which adhere to the outside of the fuel rod during the manufacturing process.

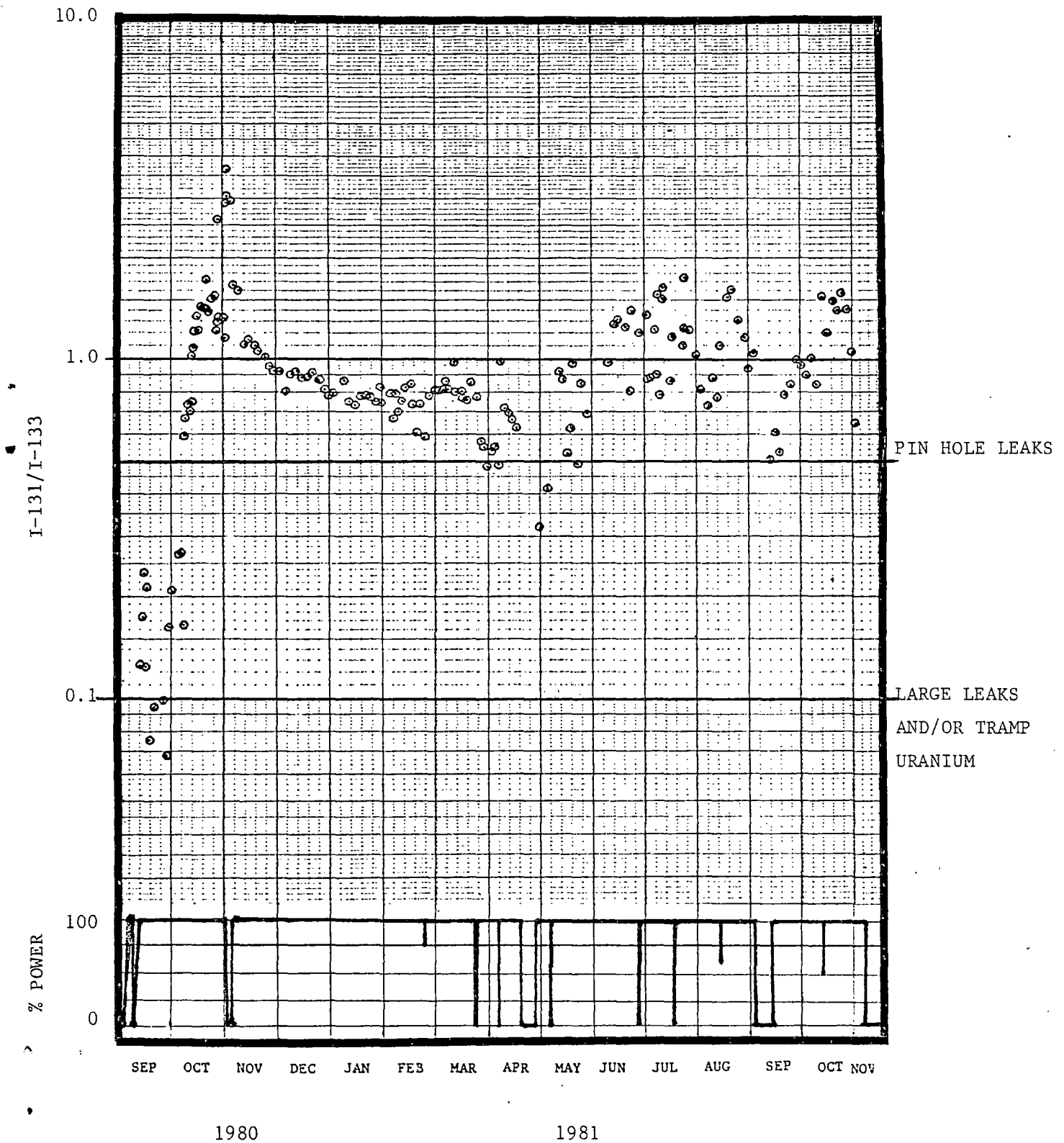
SURRY UNIT 2 - CYCLE 5

DOSE EQUIVALENT I-131 CONCENTRATION vs. TIME



SURRY UNIT 2 - CYCLE 5

I-131/I-133 RATIO vs. TIME



Section 6

CONCLUSIONS

The Surry 2 core has completed Cycle 5 operation. Throughout this cycle, all core performance indicators compared favorably with the design predictions and all core related Technical Specifications limits were met with significant margin. No abnormalities in reactivity, power distribution, or burnup accumulation were detected. In addition, the excellent mechanical integrity of the fuel has not changed significantly throughout Cycle 5 as indicated by the radioiodine analysis.

REFERENCES

- 1) Mr. T. K. Ross and Mr. J. H. Leberstien, "Surry Unit 2, Cycle 5 Startup Physics Test Report," VEP-FRD-37, October, 1980.
- 2) Surry Power Station Unit 1 and 2 Technical Specifications, Sections 3.1.D and 3.12.B.
- 3) Mr. T. K. Ross, "NEWTOTE Code", NFO-CCR-6, Revision 2, Vepco, July, 1981.
- 4) Mr. R. D. Klatt, Mr. W. D. Leggett, III, and Mr. L. D. Eisenhart, "FOLLOW Code," WCAP-7482, February, 1970.
- 5) Mr. W. D. Leggett, III and Mr. L. D. Eisenhart, "INCORE Code," WCAP-7149, December, 1967.
- 6) Letter from Mr. B. R. Sylvia (Vepco) to Mr. H. R. Denton (NRC), "Supplemental Information Concerning Our Assessment of NUREG-0630", July 28, 1980 (Docket No. 50-281).
- 7) Surry Power Station Units 1 and 2, Technical Specifications Amendment Number 70, dated June 16, 1981.