

APPENDIX A

NOTICE OF VIOLATION

Virginia Electric and Power Company
Surry Power Station

License Nos. DPR-32
DPR-37

Based on the NRC inspection May 5 - June 6, 1980, certain of your activities were apparently not conducted in full compliance with NRC requirements as indicated below. These items have been categorized as described in correspondence to you dated December 31, 1974.

- A. As required by Technical Specification 6.4.D, the detailed written procedures with appropriate check-off lists and instructions provided for preventive or corrective maintenance operations which would have an effect on the safety of the reactor shall be followed.

Contrary to the above, on May 19, 1980, the inspector observed that startup check sheet OP-1B had not been followed prior to Unit 1 startup. The Operating Procedure 1B checklist requires inspections to insure that the containment recirculation sump screens are in place and clear of debris; on the above date, a bucket, mops, rags, plastic bags, a cardboard box and assorted trash was found adjacent to the in-place recirc sump screens of the operating unit.

This is an infraction and applies to Unit 1. A similar item was brought to your attention in our letter dated March 28, 1979.

- B. As required by Criterion III of Appendix B to 10 CFR 50 and Section III of the NPS QA Manual, the design control measures provide assurance and verification of design adequacy by the performance of design reviews, the use of alternate or simplified calculational methods, or by the performance of a suitable testing program. The design change procedure requires an engineering evaluation (safety analysis) to determine the effect on plant operation.

Contrary to the above, the accident reanalysis for Design Change No. 80-519 was not completed and presented to station management until May 28, 1980, even though this modification of the containment recirculation spray system had been completed prior to Unit 1 startup on May 11, 1980. Subsequent review of containment pressure and temperature revealed that Unit 1 had been operated outside the requirements of Technical Specification 3.8 on several occasions between May 11 and May 28, 1980.

This is an infraction and applies to Unit 1.

- C. As required by Technical Specification 3.10.A.6, at least one residual heat removal (RHR) pump and heat exchanger shall be operable to circulate reactor coolant during refueling operations.

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Contrary to the above, Periodic Test 30.2, RHR System MOV Cycling, was not performed to verify that at least one train or subsystem of the RHR system was operable prior to fuel loading on April 19, 1980. One of the four motor operated valves (MOV 2720B) in the RHR system and PT 30.2 was cycled, timed, and documented on April 20, 1980. Documentation of the remaining RHR valves was not provided although station personnel stated that the test was completed on these valves.

This is an infraction.